



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**

REGION I
2100 RENAISSANCE BOULEVARD, SUITE 100
KING OF PRUSSIA, PENNSYLVANIA 19406-2713

April 20, 2021

Mr. Don Moul
Executive Vice President,
Nuclear Division and Chief Nuclear Officer
NextEra Energy Seabrook, LLC
Florida Power & Light Company
Mail Stop: EX/JB
700 Universe Blvd
Juno Beach, FL 33408

SUBJECT: SEABROOK STATION – DESIGN BASIS ASSURANCE INSPECTION
(PROGRAMS) INSPECTION REPORT 05000443/2021010

Dear Mr. Moul:

On March 18, 2021, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Seabrook Station and discussed the results of this inspection with Brian Booth, Site Vice President, and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

X /RA/

Signed by: Melvin K. Gray

Mel Gray, Chief
Engineering Branch 1
Division of Operating Reactor Safety

Docket No. 05000443
License No. NPF-86

Enclosure:
As stated

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SUBJECT: SEABROOK STATION – DESIGN BASIS ASSURANCE INSPECTION
(PROGRAMS) INSPECTION REPORT 05000443/2021010 DATED
APRIL 20 2021

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**U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report**

Docket Number: 05000443

License Number: NPF-86

Report Number: 05000443/2021010

Enterprise Identifier: I-2021-010-0024

Licensee: NextEra Energy Seabrook, LLC

Facility: Seabrook Station

Location: Seabrook, NH

Inspection Dates: March 01, 2021 to March 18, 2021

Inspectors: P. Cataldo, Senior Reactor Inspector
K. Mangan, Senior Reactor Inspector
A. Patel, Senior Reactor Inspector

Approved By: Mel Gray, Chief
Engineering Branch 1
Division of Operating Reactor Safety

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting a design basis assurance inspection (programs) at Seabrook Station, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

None.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

Starting on March 20, 2020, in response to the National Emergency declared by the President of the United States on the public health risks of the coronavirus (COVID-19), regional inspectors were directed to begin telework. Regional based inspections were evaluated to determine if all or a portion of the objectives and requirements stated in the IP could be performed remotely. For the inspection documented below portions of the IP were completed remotely as well as on site and all the objectives and requirements for completion of the IP were met.

REACTOR SAFETY

71111.21N.02 - Design-Basis Capability of Power-Operated Valves Under 10 CFR 50.55a Requirements

POV Review (IP Section 03) (11 Samples)

The inspectors:

- a. Determined whether the sampled POVs are being tested and maintained in accordance with NRC regulations along with the licensee's commitments and/or licensing bases.cope
- b. Determined whether the sampled POVs are capable of performing their design-basis functions.
- c. Determined whether testing of the sampled POVs is adequate to demonstrate the capability of the POVs to perform their safety functions under design-basis conditions.
- d. Evaluated maintenance activities including a walkdown of the sampled POVs (if accessible).

- (1) RC-PCV-456-B, 'B' Power Operated Relief Valve
- (2) SWS-V-V4, Service Water to Secondary Component Cooling Isolation Valve
- (3) CBS-V-8, Containment Recirculation Sump Tank 101A Suction Isolation Valve
- (4) SI-V-139, Chemical and Volume Control/Safety Injection Cold Leg Isolation
- (5) RH-V-36, Residual Heat Removal Train 'B' Discharge to Safety Injection Suction Valve
- (6) SI-V-114, High Pressure Injection Common Cold Leg Discharge Motor Operated Valve
- (7) FW-V-156, Start-Up Feed Pump/Emergency Feed Water Cross Connect Isolation Valve
- (8) MS-V-395, Turbine Driven Emergency Feedwater Pump Common Steam Supply Isolation Valve

- (9) CC-TV-2171-1, Primary Component Cooling Water Heat Exchanger 'A' Outlet Flow Control Valve
- (10) MS-PV-3001, Steam Generator 'A' Atmospheric Relief Valve
- (11) CS-LCV-112-D, Reactor Water Storage Tank to Charging Pump Suction Isolation Valve

INSPECTION RESULTS

No findings were identified.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On March 18, 2021, the inspectors presented the design basis assurance inspection (programs) inspection results to Brian Booth, Site Vice President, and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.21N.02	Calculations	C-S-1-80903	Motor Operated Valve Differential Pressure Calculation	09/25/1998
		C-S-1-80904	Motor Operated Valve Trust, Actuator Capacity and Torque Switch Setpoint Calculation	Rev. 4
		C-S-1-91006	Adequacy of 1-MS-TK243 Air Capacity for 1-MS-V395	Revision 0
		EE-03-005	AOV System Level Design Basis Review	Revision 0
	Corrective Action Documents Resulting from Inspection	2366952		
		2386027		
		2386030		
		2386031		
		2386982		
		2387155		
	Drawings	1-IA-B20644	Instrument Air Miscellaneous Building	Revision 18
	Miscellaneous	73157	Composite Instruction Manual Valves & Actuators	Revision 0
		97771	Instruction Manual for Kerotest Valves	Revision 9
	Procedures	ES 1850 003	MOV Program Procedure	2/10/2004
		ES1850.003	Motor Operated Valve Performance Monitoring	Rev. 18
		ES1850.012	Air Operated Valve Program Procedure	Rev. 9
		OX1436.02	Turbine Driven Emergency Feedwater Pump Quarterly And Monthly Valve Alignment	Revision 32
		STP-133	Stroke and Adjustment of MS-V393, MS-394, and MS-V395	Revision 0
	Work Orders	1134531		
		40625547		