



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 7, 2021

Mr. David P. Rhoades
Senior Vice President
Exelon Generation Company, LLC
President and Chief Nuclear Officer (CNO)
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: BRAIDWOOD STATION, UNITS 1 AND 2 – CORRECTION TO AMENDMENT
NOS. 219 AND 219 REGARDING REVISION OF TECHNICAL
SPECIFICATIONS 5.6.5, “CORE OPERATING LIMITS REPORT (COLR)”
(EPID L-2020-LLA-0038)

Dear Mr. Rhoades:

By letter dated December 28, 2020 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML20317A001) the U.S. Nuclear Regulatory Commission (NRC, the Commission) issued, in part, Amendment No. 219 to Renewed Facility Operating License No. NPF-72 and Amendment No. 219 to Renewed Facility Operating License No. NPF-77 for the Braidwood Station, Units 1 and 2 (Braidwood).

The amendments contained, in part, Technical Specification Page 5.6-4 that included a typographical error in the title of WCAP-16996-P-A, Revision 1, in Technical Specification 5.6.5 b.6. Specifically, the title was identified as WACAP-16996-P-A, Revision 1. The enclosure to this letter provides a corrected copy of Technical Specification Page 5.6-4 to replace the page sent in our December 28, 2020, letter.

D. Rhoades

- 2 -

If you have any questions, please contact me at 301-415-6606 or via e-mail at Joel.Wiebe@nrc.gov

Sincerely,

/RA/

Joel S. Wiebe, Senior Project Manager
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-456 and STN 50-457

Enclosure:
Braidwood, Units 1 and 2, Technical Specification Page 5.6-4

cc: Listserv

BRAIDWOOD, UNITS 1 AND 2, TECHNICAL SPECIFICATION PAGE 5.6-4

EXELON GENERATION COMPANY, LLC

BRAIDWOOD STATION, UNITS 1 AND 2

DOCKET NOS. STN 50-456 AND STN 50-457

Enclosure

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

6. WCAP-16996-P-A, Revision 1, "Realistic LOCA Evaluation Methodology Applied to the Full Spectrum of Break Sizes (FULL SPECTRUM LOCA Methodology)," November 2016.
 7. Not Used.
 8. Not Used.
 9. WCAP-10216-P-A, Revision 1, "Relaxation of Constant Axial Offset Control - F_Q Surveillance Technical Specification," February 1994.
 10. WCAP-8745-P-A, "Design Bases for the Thermal Overpower ΔT and Thermal Overtemperature ΔT Trip Functions," September 1986.
 11. WCAP-14565-P-A, "VIPRE-01 Modeling and Qualification for Pressurized Water Reactor Non-LOCA Thermal-Hydraulic Safety Analysis," October 1999.
 12. WCAP-12610-P-A, "VANTAGE+ Fuel Assembly Reference Core Report," April 1995, (Westinghouse Proprietary).
 13. WCAP-12610-P-A & CENPD-404-P-A, Addendum 1-A, "Optimized ZIRLO™," July 2006, (Westinghouse Proprietary).
- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal mechanical limits, core thermal hydraulic limits, Emergency Core Cooling Systems (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met; and
- d. The COLR, including any midcycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.

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