



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 6, 2021

Mr. Daniel G. Stoddard
Senior Vice President and Chief Nuclear Officer
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: NORTH ANNA POWER STATION, UNIT NOS. 1 AND 2, SURRY POWER STATION UNIT NOS. 1 AND 2 – CORRECTION TO ISSUANCE OF AMENDMENT TO REVISE TECHNICAL SPECIFICATIONS TO ALLOW USAGE OF A SMALL BREAK LOSS OF COOLANT ACCIDENT METHODOLOGY (EPIDS L-2018-LLA-0195 AND L-2018-LLA-0215)

Dear Mr. Stoddard:

On March 19, 2021 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML20325A088), the U.S. Nuclear Regulatory Commission (NRC) issued Amendment Nos. 287 and 270 to Renewed Facility Operating License Nos. NPF-4 and NPF-7 for the North Anna Power Station (North Anna), Unit Nos. 1 and 2, respectively, and Amendment Nos. 303 and 303 to Renewed Facility Operating License Nos. DPR-32 and DPR-37, for the Surry Power Station (Surry), Unit Nos. 1 and 2, respectively. The amendments revised the North Anna and Surry Technical Specifications to include Framatome Topical Report EMF-2328(P)(A), Revision 0, "PWR [pressurized water reactor] Small Break LOCA [loss-of-coolant accident] Evaluation Model, S-RELAP5 Based," as supplemented by the North Anna plant-specific application report ANP-3467P, Revision 0, "North Anna Fuel-Vendor Independent Small Break LOCA Analysis Licensing Report," and Surry plant-specific report ANP-3676P, "Surry Fuel-Vendor Independent Small Break LOCA Analysis Licensing Report," in the list of methodologies approved for reference in the Core Operating Limits Report (COLR). The amendments also removed obsolete COLR references that supported use of a fuel product that is not planned for future use.

Subsequent to the issuance of the amendments, the NRC recognized it made an administrative error by not including the revised page 3 for NPF-4, NPF-7, DPR-32, and DPR-37 attached to the license amendment.

The NRC staff has determined that these errors were made inadvertently and has corrected the errors. The corrections do not change any of the conclusions associated with the issuance of Amendment Nos. 287 and 270 to Renewed Facility Operating License Nos. NPF-4 and NPF-7 for North Anna, Unit Nos. 1 and 2, respectively, and Amendment Nos. 303 and 303 to Renewed Facility Operating License Nos. DPR-32 and DPR-37, for Surry, Unit Nos. 1 and 2, respectively, and do not affect the associated notice to the public. Further, the error did not affect the proposed finding that the amendments involve no significant hazards consideration published in the *Federal Register* on October 9, 2018

(83 FR 50697) for Surry and December 4, 2018 (83 FR 62609) for North Anna. Accordingly, Enclosures 1 and 2 to this letter provide corrected pages for Amendment No. 287 and 270 to Renewed Facility Operating License Nos. NPF-4 and NPF-7 for North Anna, Unit Nos. 1 and 2, respectively. Enclosures 3 and 4 to this letter provide corrected pages for Amendment Nos. 303 and 303 to Renewed Facility Operating License Nos. DPR-32 and DPR-37, for Surry, Unit Nos. 1 and 2, respectively.

If you have any questions, please contact me at 301-415-4032, or via e-mail at Ed.Miller@nrc.gov.

Sincerely,

/RA/

G. Edward Miller, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-338, 50-339, 50-280, and 50-281

Enclosure:

1. Revised pages for Amendment No. 287 to NPF-4
2. Revised pages for Amendment No. 270 to NPF-7
3. Revised pages for Amendment No. 303 to DPR-32
4. Revised pages for Amendment No. 303 to DPR-37

cc: Listserv

ENCLOSURE 1

Corrected Pages for Amendment No. 287 to NPF-4

- (2) Pursuant to the Act and 10 CFR Part 70, VEPCO to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Updated Final Safety Analysis Report;
 - (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material, without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or component; and
 - (5) Pursuant to the Act and 10 CFR Parts 30 and 70, VEPCO to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I; Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

VEPCO is authorized to operate the North Anna Power Station, Unit No. 1, at reactor core power levels not in excess of 2940 megawatts (thermal).
 - (2) Technical Specifications

Technical Specifications contained in Appendix A, as revised through Amendment No. 287 are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

ENCLOSURE 2

Corrected Pages for Amendment No. 270 to NPF-7

- (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material, without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or component; and
- (5) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations as set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

VEPCO is authorized to operate the facility at steady state reactor core power levels not in excess of 2940 megawatts (thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 270 are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Additional Conditions

The matters specified in the following conditions shall be completed to the satisfaction of the Commission within the stated time periods following the insurance of the condition or within the operational restrictions indicated. The removal of these conditions shall be made by an amendment to the renewed license supported by a favorable evaluation by the Commission:

- a. If VEPCO plans to remove or to make significant changes in the normal operation of equipment that controls the amount of radioactivity in effluents from the North Anna Power Station, the

ENCLOSURE 3

Corrected Pages for Amendment No. 303 to DPR-32

3. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

- A. Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 2587 megawatts (thermal).

- B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 303 are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

- C. Reports

The licensee shall make certain reports in accordance with the requirements of the Technical Specifications.

- D. Records

The licensee shall keep facility operating records in accordance with the requirements of the Technical Specifications.

- E. Deleted by Amendment 65

- F. Deleted by Amendment 71

- G. Deleted by Amendment 227

- H. Deleted by Amendment 227

- I. Fire Protection

The licensee shall implement and maintain in effect the provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report and as approved in the SER dated September 19, 1979, (and Supplements dated May 29, 1980, October 9, 1980, December 18, 1980, February 13, 1981, December 4, 1981, April 27, 1982, November 18, 1982, January 17, 1984, February 25, 1988, and

ENCLOSURE 4

Corrected Pages for Amendment No. 303 to DPR-37

- E. Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such by product and special nuclear materials as may be produced by the operation of the facility.
- 3. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:
 - A. Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power Levels not in excess of 2587 megawatts (thermal)
 - B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 303 are hereby incorporated in this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.
 - C. Reports

The licensee shall make certain reports in accordance with the requirements of the Technical Specifications.
 - D. Records

The licensee shall keep facility operating records in accordance with the Requirements of the Technical Specifications.
 - E. Deleted by Amendment 54
 - F. Deleted by Amendment 59 and Amendment 65
 - G. Deleted by Amendment 227
 - H. Deleted by Amendment 227

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