



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**

REGION III
2443 WARRENVILLE ROAD, SUITE 210
LISLE, ILLINOIS 60532-4352

February 8, 2021

Mr. Michael Strobe
Site Vice President
NextEra Energy Point Beach, LLC
6610 Nuclear Road
Two Rivers, WI 54241-9516

SUBJECT: POINT BEACH NUCLEAR PLANT – INTEGRATED INSPECTION REPORT
05000266/2020004 AND 05000301/2020004

Dear Mr. Strobe:

On December 31, 2020, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Point Beach Nuclear Plant. On January 12, 2021, the NRC inspectors discussed the results of this inspection with Mr. Bryan Woyak, Safety Assurance and Learning Site Director, and other members of your staff. The results of this inspection are documented in the enclosed report.

No NRC-identified or self-revealing findings were identified during this inspection.

A licensee-identified violation which was determined to be Severity Level IV is documented in this report. We are treating this violation as a non-cited violation (NCV) consistent with Section 2.3.2 of the Enforcement Policy. A traditional enforcement follow-up inspection associated with this violation is not warranted in accordance with Inspection Manual Chapter 0305, "Operating Reactor Assessment Program," because it did not involve willfulness, impeding the regulatory process, or actual safety consequences.

If you contest the violation or the significance or severity of the violation documented in this inspection report, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555-0001; with copies to the Regional Administrator, Region III; the Director, Office of Enforcement; and the NRC Resident Inspector at Point Beach Nuclear Plant.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

/RA/

Néstor J. Félix Adorno, Chief
Branch 4
Division of Reactor Projects

Docket Nos. 05000266 and 05000301
License Nos. DPR-24 and DPR-27

Enclosure:
As stated

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Letter to Michael Strope from Néstor Féliz Adorno dated February 8, 2021.

SUBJECT: POINT BEACH NUCLEAR PLANT – INTEGRATED INSPECTION REPORT
05000266/2020004 AND 05000301/2020004

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**U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report**

Docket Numbers: 05000266 and 05000301

License Numbers: DPR-24 and DPR-27

Report Numbers: 05000266/2020004 and 05000301/2020004

Enterprise Identifier: I-2020-004-0064

Licensee: NextEra Energy Point Beach, LLC

Facility: Point Beach Nuclear Plant

Location: Two Rivers, WI

Inspection Dates: October 01, 2020 to December 31, 2020

Inspectors: K. Barclay, Reactor Inspector
J. Bozga, Senior Reactor Inspector
J. Cassidy, Senior Health Physicist
M. Garza, Emergency Preparedness Inspector
T. Hartman, Senior Resident Inspector
T. McGowan, Resident Inspector
R. Ng, Project Engineer

Approved By: Néstor J. Félix Adorno, Chief
Branch 4
Division of Reactor Projects

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting an integrated inspection at Point Beach Nuclear Plant, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information. A licensee-identified non-cited violation is documented in report section: 71153.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

Type	Issue Number	Title	Report Section	Status
URI	05000266/2020004-01	Letdown Isolation Valve, 1RC-427, Failed to Open from Main Control Room	71111.12	Open
LER	05000301/2020-001-00	Reactor Protection System Channel Failure Results in Operation Prohibited by Technical Specification	71153	Closed

PLANT STATUS

Unit 1 began the inspection period at 97 percent rated thermal power. On October 3, 2020, the plant was shutdown for refueling outage U1R39. On October 26, 2020, Unit 1 was restarted. On October 28, 2020, the unit was synchronized to the grid, and on November 5, 2020, it achieved full power. The unit remained at or near full power throughout the remainder of the inspection period.

Unit 2 began the inspection period at rated thermal power and remained at or near full power throughout the remainder of the inspection period.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection, unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met, consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

Starting on March 20, 2020, in response to the National Emergency declared by the President of the United States on the public health risks of the Coronavirus Disease 2019 (COVID-19), resident inspectors were directed to begin telework and to remotely access licensee information using available technology. During this time, the resident inspectors performed periodic site visits each week; conducted plant status activities as described in IMC 2515, Appendix D, "Plant Status," observed risk-significant activities; and completed on-site portions of IPs. In addition, resident and regional baseline inspections were evaluated to determine if all or portions of the objectives and requirements stated in the IP could be performed remotely. If the inspections could be performed remotely, they were conducted per the applicable IP. In some cases, portions of an IP were completed remotely and on-site. The inspections documented below met the objectives and requirements for completion of the IP.

REACTOR SAFETY

71111.01 - Adverse Weather Protection

Seasonal Extreme Weather Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated readiness for seasonal extreme weather conditions prior to the onset of seasonal cold temperatures.

71111.04 - Equipment Alignment

Partial Walkdown Sample (IP Section 03.01) (3 Samples)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

- (1) Unit 1 train B containment spray system on November 18, 2020
- (2) Unit 1 train A component cooling water system on November 24, 2020
- (3) Unit 2 125 Volt DC system on December 1, 2020

Complete Walkdown Sample (IP Section 03.02) (1 Sample)

- (1) The inspectors evaluated system configurations during a complete walkdown of the spent fuel pool cooling water system on October 13, 2020.

71111.05 - Fire Protection

Fire Area Walkdown and Inspection Sample (IP Section 03.01) (3 Samples)

The inspectors evaluated the implementation of the fire protection program by conducting a walkdown and performing a review to verify program compliance, equipment functionality, material condition, and operational readiness of the following fire areas:

- (1) fire zones 505, 511, 516, and 520 on October 24, 2020
- (2) fire zone 680 on November 2, 2020
- (3) fire zones 304N, 304S, and 305 on November 5, 2020

71111.08P - Inservice Inspection Activities (PWR)

PWR Inservice Inspection Activities Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors verified the reactor coolant system boundary, steam generator tubes, reactor vessel internals, risk-significant piping system boundaries, and containment boundary were appropriately monitored for degradation and that repairs and replacements were appropriately fabricated, examined and accepted by reviewing the documentation for the following activities from October 6–November 3, 2020:

03.01.a - Nondestructive Examination and Welding Activities.

- ultrasonic examination (UT) of closure head peripheral CRD 22 and 26
- liquid penetrant (PT) examination of integral welded attachment SI-2501R-5-SI10-IWA
- PT examination of 1HX-1B Steam Generator Channel Head Drain
- PT examination of 1HX-1B Steam Generator Channel Head
- pressure boundary field welds of 1B Resistance Temperature Detector (RTD) manifolds (Work Order 40584606)
- pressure boundary field welds of 1AF-106 Valve (Work Order 40575122)
- pressure boundary field welds of 1AF-107 Valve (Work Order 40575123)

03.01.c – Pressurized-Water Reactor Boric Acid Corrosion Control Activities.

- boric acid evaluations for 1HX-1B and 1RC-526B
- boric acid condition reports for boric acid on 1FT-626 and Incore Flux Mapping Thimble Tubes

71111.11Q - Licensed Operator Regualification Program and Licensed Operator Performance

Licensed Operator Performance in the Actual Plant/Main Control Room (IP Section 03.01) (2 Samples)

- (1) The inspectors observed and evaluated licensed operator performance in the Control Room during Unit 1 plant shutdown on October 3, 2020.
- (2) The inspectors observed and evaluated licensed operator performance in the Control Room during Unit 1 reactor startup and turnover on October 26, 2020.

Licensed Operator Regualification Training/Examinations (IP Section 03.02) (1 Sample)

- (1) The inspectors observed and evaluated just-in-time training for Unit 1 reactor and plant startup on October 24, 2020.

71111.12 - Maintenance Effectiveness

Maintenance Effectiveness (IP Section 03.01) (3 Samples)

The inspectors evaluated the effectiveness of maintenance to ensure the following structures, systems, and components (SSCs) remain capable of performing their intended function:

- (1) 1CV-1296, Auxiliary Charging Line Isolation valve
- (2) 1RC-427, RC Loop B Cold Leg to CVCS Letdown Isolation valve failed on October 4, 2020
- (3) Radiation Monitoring System

Quality Control (IP Section 03.02) (1 Sample)

The inspectors evaluated the effectiveness of maintenance and quality control activities to ensure the following SSC remains capable of performing its intended function:

- (1) The inspectors reviewed six work packages covering a variety of maintenance activities on safety related and/or risk significant systems to evaluate whether licensee quality control verifications are properly specified in accordance with the Quality Assurance Program and are implemented as specified.

Aging Management (IP Section 03.03) (1 Sample)

The inspectors evaluated the effectiveness of the aging management program for the following SSCs that did not meet their inspection or test acceptance criteria:

- (1) 2HX-12D component cooling water heat exchanger

71111.13 - Maintenance Risk Assessments and Emergent Work Control

Risk Assessment and Management Sample (IP Section 03.01) (3 Samples)

The inspectors evaluated the accuracy and completeness of risk assessments for the following planned and emergent work activities to ensure configuration changes and appropriate work controls were addressed:

- (1) Unit 1 elevated risk due to short time to boil and reduced inventory on October 8, 2020
- (2) Unit 1 emergent work to lift and reinstall reactor pressure vessel head due to cavity leak on October 9, 2020
- (3) both units elevated risk due to maintenance on a battery charger, on November 1, 2020

71111.15 - Operability Determinations and Functionality Assessments

Operability Determination or Functionality Assessment (IP Section 03.01) (4 Samples)

The inspectors evaluated the licensee's justifications and actions associated with the following operability determinations and functionality assessments:

- (1) Unit 1 train A safety injection pump flow low during testing
- (2) G-02 emergency diesel generator had abnormal air bank pressure drop during start
- (3) foreign material found inside main feed isolation valve actuator 3-way valve
- (4) Unit 2 pressurizer pressure controller experienced abnormal operation

71111.18 - Plant Modifications

Temporary Modifications and/or Permanent Modifications (IP Section 03.01 and/or 03.02) (1 Sample)

The inspectors evaluated the following temporary or permanent modifications:

- (1) EC 295378, Replace 1RC-526A and 1RC-526B with a Pipe Cap

71111.19 - Post-Maintenance Testing

Post-Maintenance Test Sample (IP Section 03.01) (5 Samples)

The inspectors evaluated the following post-maintenance test activities to verify system operability and functionality:

- (1) 2-SOP-AF-001, Auxiliary Feedwater System Operation - Motor Driven, after replacement of AF-4089, 1/2P-53 AFP Mini-Recirc Relief on October 7, 2020
- (2) IT 300D, Main Feedwater Isolation Valve Test Train A Unit 1, after replacement of the main feedwater isolation valve actuator 3-way valve, on October 8, 2020
- (3) IT 03 Train B, Low Head Safety Injection Pumps and Valves Train B Unit 1, after maintenance on 1RH-625, HX-11B RHR HX Outlet Valve, on October 17, 2020
- (4) ORT 59, Train A Spray System CIV Leakage Test Unit 1, after maintenance on 1SI-868A, Containment Spray Nozzle A Header Isolation, on October 22, 2020

- (5) 0-SOP-DC-004, 125 VDC System operating procedure after maintenance on D-106/D-108, on December 7, 2020

71111.20 - Refueling and Other Outage Activities

Refueling/Other Outage Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated U1R39 activities from October 1–November 5, 2020.

71111.22 - Surveillance Testing

The inspectors evaluated the following surveillance tests:

Surveillance Tests (other) (IP Section 03.01) (1 Sample)

- (1) ORT 3A, Safety Injection Actuation with Loss of Engineered Safeguards AC (Train A) Unit 1, on October 5, 2020

Inservice Testing (IP Section 03.01) (1 Sample)

- (1) IT 760, Flow Test of High Head Safety Injection Check Valves (Refueling) Unit 1, on October 9, 2020

Containment Isolation Valve Testing (IP Section 03.01) (1 Sample)

- (1) IT 250, Chemical and Volume Control and Component Cooling System Valves (Cold Shutdown) Unit 1, on October 20, 2020

71114.04 - Emergency Action Level and Emergency Plan Changes

Inspection Review (IP Section 02.01-02.03) (1 Sample)

- (1) The inspectors completed an evaluation of submitted emergency action level and emergency plan changes on December 11, 2020. This evaluation does not constitute NRC approval.

71114.06 - Drill Evaluation

Drill/Training Evolution Observation (IP Section 03.02) (1 Sample)

The inspectors evaluated:

- (1) an emergency coordinator classification evaluation on November 3, 2020

RADIATION SAFETY

71124.01 - Radiological Hazard Assessment and Exposure Controls

Instructions to Workers (IP Section 03.02) (1 Partial)

- (1) (Partial)
The inspectors evaluated radiological protection-related instructions to plant workers. However, the completion of the sample requires direct inspector observations which could not be performed due to COVID-19 restrictions.

OTHER ACTIVITIES – BASELINE

71151 - Performance Indicator Verification

The inspectors verified licensee performance indicators submittals listed below:

MS09: Residual Heat Removal Systems (IP Section 02.08) (2 Samples)

The inspectors verified licensee performance indicators submittal listed below:

- (1) Unit 1 (October 1, 2019–September 30, 2020)
- (2) Unit 2 (October 1, 2019–September 30, 2020)

MS10: Cooling Water Support Systems (IP Section 02.09) (2 Samples)

The inspectors verified licensee performance indicators submittal listed below:

- (1) Unit 1 (October 1, 2019–September 30, 2020)
- (2) Unit 2 (October 1, 2019–September 30, 2020)

BI01: Reactor Coolant System (RCS) Specific Activity Sample (IP Section 02.10) (2 Samples)

- (1) Unit 1 (July 1, 2019–September 30, 2020)
- (2) Unit 2 (July 1, 2019–September30, 2020)

OR01: Occupational Exposure Control Effectiveness Sample (IP Section 02.15) (1 Sample)

- (1) (September 1, 2019–September30, 2020)

PR01: Radiological Effluent Technical Specifications/Offsite Dose Calculation Manual Radiological Effluent Occurrences (RETS/ODCM) Radiological Effluent Occurrences Sample (IP Section 02.16) (1 Sample)

- (1) (September 1, 2019–September 30, 2020)

71152 - Problem Identification and Resolution

Semiannual Trend Review (IP Section 02.02) (1 Sample)

- (1) The inspectors reviewed the licensee’s corrective action program for potential adverse trends in human performance errors that might be indicative of a more significant safety issue.

Annual Follow-up of Selected Issues (IP Section 02.03) (2 Samples)

The inspectors reviewed the licensee’s implementation of its corrective action program related to the following issues:

- (1) Action Request (AR) 2369621; Part 21: Nutherm Continuously Energized Eaton D26 Relays; and AR 2373438; Additional Action Required for Part 21 - AR 02369621
- (2) AR 2350545; 2RH-624 Opened with No Flow Response and AR 2361516; 1993 Part 21 Documentation Ambiguous

71153 – Follow-up of Events and Notices of Enforcement Discretion

Event Report (IP Section 03.02) (1 Sample)

The inspectors evaluated the following licensee event reports (LERs):

- (1) LER 05000301/2020-001-00, Reactor Protection System Channel Failure Results in Operation Prohibited by Technical Specifications.
(ADAMS Accession No. ML20163A561)

The inspection conclusions associated with this LER are documented in this report under Inspection Results section.

INSPECTION RESULTS

Unresolved Item (Open)	Letdown Isolation Valve, 1RC-427, Failed to Open from Main Control Room URI 05000266/2020004-01	71111.12
<p><u>Description:</u></p> <p>On October 4, 2020, letdown isolation valve 1RC-427 failed to close remotely from the control room when operators were transitioning reactor unit 1 from hot standby to cold shutdown in accordance with revision 19 of procedure OP 3C Unit 1, “Hot Standby to Cold Shutdown, Unit 1.” This motor-operated valve (MOV) is connected to the reactor coolant system (RCS) and is part of the RCS pressure boundary. The licensee documented this issue in the Corrective Action Program as condition report (AR) 2370332. This AR declared the MOV as functional for the ability to maintain the RCS pressure boundary. On October 11, 2020, the licensee locally and manually throttled the MOV to a mid-position and then was able to successfully stroke the valve remotely using the associated control room switch. On October 21, 2020, the licensee performed revision 24 of procedure IT 320, “CVCS Valves (Cold Shutdown) Unit 1,” to verify the valve stroke time met the inservice testing program requirements. The MOV stroked within the prescribed time band.</p>		

The licensee completed a maintenance rule functional failure (MRFF) evaluation on October 24, 2020, and determined the valve did not have a functional failure. However, the inspectors noted the MRFF evaluation stated the MOV had a function to close for NFPA 805 safe shutdown, abnormal operating procedures (AOPs), and emergency operating procedures (EOPs). The inspectors became concerned because the initial functionality assessment only considered the impact of the MOV failure to the RCS pressure boundary function. In addition, the licensee did not have an action to address these additional functions. Furthermore, the inspectors noted this MOV had experienced similar failures in at least three previous refueling outages. For example, AR 2307067 documented a similar failure during the 2019 refueling outage and referenced similar issues that occurred in 2017 and 2013. The 2019 issue was also corrected by manually and locally placing the valve in a mid-position followed by successful electrical stroking of the valve. Again, the licensee believed the MOV remained functional and only evaluated against the RCS pressure boundary function.

The inspectors questioned the MOV ability to perform its functions to close remotely from the control room and automatically for NFPA 805 safe shutdown, AOPs, and EOPs, when it has been unable to remotely close when transitioning reactor unit 1 from hot standby to cold shutdown in at least four separate occasions. At the end of this inspection period, the licensee agreed the valve was non-functional with respect to NFPA 805 functions and is working on a past functionality review.

Planned Closure Actions: Additional information is needed from the licensee for the NRC to evaluate the following concerns:

- whether or not the maintenance being performed on the valve is appropriate
- if the licensee has adequately evaluated the functionality of 1RC-427

Licensee Actions: On December 23, 2020, the licensee declared 1RC-427 non-functional and implemented hourly fire watches in several fire zones to minimize the likelihood of a fire affecting the 1CV-200A/B/C valves. In addition, if unable to close (or maintain closed) either 1RC-427 or the 1CV-200 valves the licensee has direction to open the breakers for 1CV-200A/B/C to remove power from the valves allowing them to close. The 1CV-200A/B/C are valves located downstream of the flow orifices, which are downstream of 1RC-427. These valves are interlocked with 1RC-427 such that they will receive an automatic close signal if 1RC-427 is not full open. The licensee considers the 1CV-200 valves as a backup to 1RC-427.

Corrective Action References: ARs 2378631, 2370332, 2307067

Observation: Semi Annual Trend Review	71152
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The inspectors noted a continued trend in the human performance (HU) error rate at the station. The station initiated Condition Report 2375752 to evaluate the HU error rate. During 2020, the station had one "significant" error, six "noteworthy" errors, 117 "minor" errors, and 91 precursor events, as defined by their processes. This trend was similar than the 2019 HU error rate trend. The licensee determined there was no adverse trend and that the error rate was "sustained performance." Nonetheless, the licensee initiated a level 1 assessment (AR 2380282) to determine if any common drivers/gaps existed related to the HU errors. One of the gaps identified was that managers/supervisors did not help the workers understand what HU tools were appropriate for a given task. The message given to the workers was "use your HU tools" without any specifics. The licensee plans to conduct a

HU campaign focusing on HU tool selection and usage. Within this campaign, the safety/HU group plans to provide four monthly HU sharing information updates and perform an effectiveness review in the first quarter of 2022. In addition, since many of the HU errors occurred due to failures to follow procedure use and adherence standards, the station issued an escalated action (Condition Report 2375766) to address the gaps for procedure use standards. This escalation will look into any cultural aspects that may be leading to the performance gaps.

The inspectors did not identify any findings or violations during their review that have not already been addressed.

Licensee-Identified Non-Cited Violation	71153
<p>This violation of very low safety significance was identified by the licensee and has been entered into the licensee corrective action program and is being treated as a non-cited violation, consistent with Section 2.3.2 of the Enforcement Policy.</p>	
<p>Violation: Point Beach Technical Specifications (TS) Limiting Condition for Operation (LCO) 3.3.1, "Reactor Protection System (RPS) Instrumentation," stated that the RPS instrumentation for each function in Table 3.3.1-1 shall be operable. Table 3.3.1-1 included the overtemperature delta temperature function (i.e., Function #5) and required four channels in Modes 1 and 2 or entry to Condition D. When one channel was inoperable, Condition D required the licensee to place the channel in trip within 1 hour or be in Mode 3 within 7 hours.</p>	
<p>Contrary to the above, from April 10 to April 13, 2020, the licensee failed to have operable RPS instrumentation for each function in Table 3.3.1-1. Specifically, one of four channels (i.e., Channel #4) associated with the overtemperature delta temperature function on the Unit 2 reactor was inoperable while in Mode 1 and the licensee did not place the inoperable channel in trip within 1 hour or Unit 2 reactor in Mode 3 within 7 hours. The licensee restored compliance on April 13, 2020, at 10:50 am, when they placed the inoperable channel in trip. This issue is associated with LER 05000301/2020-001-00, Reactor Protection System Channel Failure Results in Operation Prohibited by Technical Specifications.</p>	
<p>Significance/Severity: No Performance Deficiency. Severity Level IV. Traditional Enforcement is being used to disposition this violation with no associated Reactor Oversight Process (ROP) performance deficiency, per NRC Enforcement Policy Sections 2.2.4.d and 3.10. The inspectors concluded this violation was not associated with an ROP performance deficiency because it was not within the licensee's ability to foresee and correct. Specifically, the RPS channel failure was not reasonably evident to the operators. While the channel signal amplitude peaks decreased, the average value of the signal did not change after the failure. Thus, the indication on the operator control boards remained unchanged. Additionally, the inoperable channel module was procured as a safety-related component and functioned properly during its installation and calibration less than a month before its failure.</p>	
<p>Corrective Action References: AR 2352490</p>	

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On January 12, 2021, the inspectors presented the integrated inspection results to Mr. B. Woyak, Safety Assurance and Learning Site Director, and other members of the licensee staff.
- On October 26, 2020, the inspectors presented the radiation protection inspection results to Ms. J. Walters, Radiation Protection Manager, and other members of the licensee staff.
- On November 3, 2020, the inspectors presented the ISI inspection results to Mr. B. Woyak, Safety Assurance and Learning Site Director, and other members of the licensee staff.
- On December 11, 2020, the inspectors presented the Emergency Action Level and Emergency Plan Changes inspection results to Mr. D. Smith, Emergency Preparedness Manager, and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.01	Miscellaneous		List of Work Orders Coded Seasonal Readiness (Winter)	11/10/2020
	Procedures	OP-AA-102-1002	Seasonal Readiness	35
71111.04	Drawings	110E017, Sheet 3	P&ID Safety Injection System	48
		110E018, Sheet 3	P&ID Auxiliary Coolant System	45
		110E018, Sheet 4	P&ID Auxiliary Cooling System	53
		110E018, Sheet 5	P&ID CCW Minimum Flow Recirculation Cross Connect Details, Unit 1	0
	Miscellaneous	1-CL-CC-001	Component Cooling Unit 1	20
		CL 5C	Spent Fuel Pool Cooling and Refueling Water Circulating Pump Normal Operation Valve Lineup	14
		CL 7A	Safety Injection System Checklist Mode 1, 2, 3 Unit 1	40
71111.05	Fire Plans	PFP-0-CB	Pre-Fire Plan Control Building Elev 8 FT, 26 FT, 44 FT, and 66 FT	2
		PFP-0-PAS	Pre-Fire Plan Protected Area South (Inside the Fence)	2
		PFP-1-CONT-FAC	Pre-Fire Plan Unit 1 Containment Building/Facade	1
	Miscellaneous	FPTE 2016-028	PBNP Detailed Fire Modeling Report: Fire Compartment 505 Containment - Unit 1 - EL 8'0"	1
		FPTE 2016-029	PBNP Detailed Fire Modeling Report: Fire Compartment 511 Containment - Unit 1 - EL 21'0"	1
		FPTE 2016-030	PBNP Detailed Fire Modeling Report: Fire Compartment 516 Containment - Unit 1 - EL 46'0"	1
71111.08P	Calculations	LTR-CECO-20-089	Evaluation of Material Removal at Point Beach Unit 1 Steam Generator Channel Head Drain	2
		WCAP-17905-P	Model 44F Replacement Steam Generator Stress Report Addendum 3 for Point Beach Unit 1 Channel Head Bowl Drain Modification	2
		WCAP-17905-P	Model 44F Replacement Steam Generator Stress Report Addendum 3 for Point Beach Unit 1 Channel Head Bowl Drain Modification	3
	Corrective Action Documents	AR 2297583	West NSAL 05-02 Rev 1 OE Review	01/11/2019
		AR 2310042	Support AC-152N-17-H11 Gap Less than Design Drawing	04/11/2019

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		AR 2370320	1RC-526B Liquid Penetrant Indications	10/04/2020
		AR 2371547	Unit 1 B SG Channel Head Liquid Penetrant Indications	10/13/2020
	Corrective Action Documents Resulting from Inspection	AR 2371957	NRC Walkdown Discovered Boric Acid on Components	10/16/2020
		AR 2373417	FSAR Table A.5-3 Changed in Error (Legacy)	10/27/2020
		AR 2373596	File Copy of Specification 955381 Rev 2 Missing Even # Pages	10/28/2020
		AR 2373861	Stress Report for Unit 1 SGs Does Not Document SSE Results	10/30/2020
	Engineering Changes	295371	Steam Generator Channel Head Drain Weld Removal to Address Field Indications (U1R39)	2
	Miscellaneous	Design Specification No. 955381	NextEra Energy Point Beach Unit 1 Model 44F Replacement Steam Generators	4
		Repair/Replacement Activity No. 2020-029		10/12/2020
		Repair/Replacement Activity No. 2020-031		10/14/2020
	NDE Reports	1RC-526B	Liquid Penetrant Examination of 1HX-1B Steam Generator Channel Head Drain	10/04/2020
		2020U1VT-009	RPV Closure Head (BACC Outage Visual)	10/15/2020
		2020U1VT-010	RPV Bottom Mounted Instrumentation	10/07/2020
		Unit 1 Steam Generator B	Lower Bowl Drain Surface Area	10/16/2020
		Unit 1 Steam Generator B	Liquid Penetrant Examination of Unit 1 Steam Generator B Bowl Drain	10/12/2020
	Procedures	NDE 120	Manual Ultrasonic Examination of Control Rod Drive and Instrument Nozzle Housing Welds	0
		NDE 401	Visible Dye Penetrant Examination Expanded Temperature Applications (60F TO 350F)	10
		NDE 451	Visible Dye Penetrant Examination Temperature Applications 40F to 125F	32
		NDE 757	Visual Examination for Leakage of Pressure Vessel Penetrations	11

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
	Work Orders	40713753-20	HX-1B SG Channel Head DRAIN	10/12/2020
		40744957-04	1HX-1B Surface Anomaly on Steam Generator B Lower	10/16/2020
71111.11Q	Corrective Action Documents Resulting from Inspection	AR 2373292	OP-1B Step Marked N/A After Subsequent Steps Performed	10/26/2020
	Miscellaneous	PBN LOC Mod 1-5 JITT	Simulator Exercise Guide - Mode 1 thru 5 JITT	0
	Procedures	OP 1B	Reactor Startup	80
		OP 3A Unit 1	Power Operation to Hot Standby Unit 1	21
OP 3B		Reactor Shutdown	48	
71111.12	Corrective Action Documents	AR 2307067	1RC-427, RC Loop to CVCS Letdown, Does Not Close	03/24/2019
		AR 2355570	2HX-12D As-Found Coating Inspection	05/05/2020
		AR 2359423	RMS System Exceeds Maximum Number of Functional Failures	06/10/2020
		AR 2370332	1RC-427 Will Not Stroke Shut From 1C04 in Control	10/04/2020
		AR 2371361	1CV-1296 Failed Primary Boundary Leak Test	10/12/2020
		AR 2372314	1CV-1296 Actuator is Out of Alignment, Causing Packing Leakage	10/19/2020
		AR 2372321	AR for Excessive Leakage on ORT 46	10/20/2020
		AR 2372690	1CV-1296 Exceeds Admin Limit of ORT 46	10/21/2020
		AR 2372766	Replace 1CV-01296 During Next Outage	10/22/2020
	AR 2373171	Active Boric Acid Leak on 1CV-1296	10/26/2020	
	Corrective Action Documents Resulting from Inspection	AR 2377247	RMP 9006-6 Component Cooling Water Pump Overhaul Missing QC	12/03/2020
		AR 2378631	Screening Adequacy Questioned for 1RC-427 MOV Fail to Close	12/16/2020
	Miscellaneous		Maintenance Rule (a)(1) Action Plan - Radiation Monitoring System	10/09/2020
	Procedures	EN-AA-206	Renewed License Process	11
		LR-AMP-021-OCCW	Open Cycle Cooling (Service) Water System Surveillance Program Basis Document for License Renewal	10
		NP 11.1.14	Inspection Planning	11

Inspection Procedure	Type	Designation	Description or Title	Revision or Date	
	Work Orders	NP 11.1.15	Quality Control Procedures	5	
		WO 40659838	2HX-012D GL 89-13 Open/Inspect/Clean/Close HX	05/12/2020	
		WO 40635290 01	2R-1 Remove and Reinstall RV Head	04/04/2020	
		WO 40658841 07	HX-12B Contingency to Repair Coating if Required	04/30/2020	
		WO 40659838 20	2HX-012D Contingency to Weld Repair Heat Exchanger	05/07/2020	
		WO 40677714 01	P-035B-E/Small Oil Leak on Valve Cover for P-035B-E	08/29/2019	
		WO 40687597 01	3Y Mechanical Maintenance	06/11/2020	
		WO 40693366 01	M-3-5-16-W21/Repair Fire Penetration	01/07/2020	
71111.13	Corrective Action Documents Resulting from Inspection	WO 40711598 01	2P-11A CCW Pump Inboard Seal Oil Leak	03/27/2020	
		AR 2377034	D-105 Battery Did Not Have Local GE Sign	12/01/2020	
		Miscellaneous		PBNP Shutdown Safety Assessment and Fire Inspection Checklist	10/09/2020
				PBNP Shutdown Safety Assessment and Fire Inspection Checklist	12/01/2020
	PBNP Shutdown Safety Assessment and Fire Inspection Checklist		10/08/2020		
	Procedures	NP 10.3.7	On-Line Safety Assessment	43	
71111.15	Corrective Action Documents	AR 2370318	1CS-256A, FM Found Inside the Valve and Exhaust Muffler	10/04/2020	
		AR 2370982	1SI-899A Flow Low Out of Limits In IT-760	10/09/2020	
		AR 2371249	G-02 EDG Air Bank Pressure Drop	10/11/2020	
		AR 2374863	G-02 Air Start Motor No Residual Oil	11/08/2020	
		AR 2379147	Pressurizer Pressure Controller Abnormal Operation	12/23/2020	
71111.18	Calculations	2020-0008	Unit 1 Steam Generator Drain Line Nozzle Loads	0	
	Corrective Action Documents	AR 2371710	U1 SG B Bowl Drain Pipe Internal Pit	10/15/2020	
		AR 2372091	Difficulty Welding Drain Cap on Unit 1 B SG	10/18/2020	
	Corrective Action Documents Resulting from Inspection	AR 2371782	U1 SG Drain Line Stress Analysis May Not be Bounding	10/15/2020	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
	Engineering Changes	295378	Replace 1RC-526A and 1RC-526B With A Pipe Cap	0
71111.19	Procedures	0-SOP-DC-004	125 VDC System, Bus D-04 & Components	24
		0-SOP-DC-005	125 VDC System, Swing Buses & Components	25
		IT 03 Train B	Low Head Safety Injection Pumps and Valves Train B Unit 1	9
		ORT 59	Train A Spray System CIV Leakage Test Unit 1	36
	Work Orders	WO 40657564	1SI-868A Exceeds Administrative Leakage Limit	10/22/2020
		WO 40662959	AF-4089 Replace Relief Valve Require Duel Unit TSAC	10/06/2020
		WO 40666356	1CS-00256A / Replace Seat and Seal Material Per EC 294033	10/08/2020
		WO 40666758	D-106/D-108, Bat Disch & Charger Maint and SR Tests	0
	WO 40731898	1RH-625/Part 21 Woodruff Key Replacement	10/17/2020	
71111.20	Corrective Action Documents Resulting from Inspection	AR 2370236	B RCP Pump Cubicle Lower Level Area Oil Present	10/03/2020
		AR 2371957	NRC Walkdown Discovered Boric Acid on Components	10/16/2020
		AR 2373033	Unit 1 Containment Walkdown Identified the Following 3 Items	10/24/2020
	Procedures	1RMP 9096-1	Reactor Vessel Head Removal and Installation Using Biach Tensioning System	27
		NP 8.4.7	Control of Safe Load Path and Rigging Manual	15
		OP 1B	Reactor Startup	80
		OP 1C Unit 1	Startup to Power Operation Unit 1	43
		OP 3C Unit 1	Hot Standby to Cold Shutdown Unit 1	19
	SLP 1	Safe Load Path and Rigging Manual - Items Lifted by Containment Polar Crane Unit 1	30	
71111.22	Corrective Action Documents	AR 2369297	Found Flow Transmitter Out of Tolerance During Calibration	09/24/2020
		AR 2370982	1SI-889A Flow Low Out of Limits in IT 760	10/09/2020
	Corrective Action Documents Resulting from Inspection	AR 2372028	U1 ORT 3A Documentation and Signoff Inconsistencies	10/17/2020
	Procedures	IT 250	Chemical and Volume Control and Component Cooling System Valves (Cold Shutdown) Unit 1	28

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		IT 760	Flow Test of High Head Safety Injection Check Valves (Refueling) Unit 1	17
		ORT 3A Unit 1	Safety Injection Actuation with Loss of Engineered Safeguards AC (Train A) Unit 1	50
71114.04	Miscellaneous	4Q19 EP-5.0	10 CFR 50.54(q)(2) - Revising the Emergency Plan Evaluation Form	10/09/2019
		4Q19-EPIP 1-3-1	10 CFR 50.54 (q)(2) Revising the Emergency Plan Evaluation Form	10/09/2019
		4Q19-EPIP 1-3-2	10 CFR 50.54 (q)(2) Revising the Emergency Plan Evaluation Form	10/09/2019
		4Q19-EPIP 4-2	10 CFR 50.54 (q)(2) Revising the Emergency Plan Evaluation Form	10/09/2019
71114.06	Miscellaneous	PBN EPR 203 003D	EC DEP Opportunity - September 2020	09/09/2020
71124.01	ALARA Plans	ALARA Package No: R021 RWP 20-1062	Pre-Job ALARA Review - Emergent 1RC-526B Steam Generator Bowl Drain Repair	10/07/2020
	Radiation Surveys	PBPROD-M-20201003-33	Primary Manway Platform B Side - Insulation Removal	10/03/2020
		PBPROD-M-20201003-7	Primary Manway Platform B Side - Initial Entry	10/03/2020
		PBPROD-M-20201005-1	Primary Manway Platform - B Side	10/05/2020
		PBPROD-M-20201005-37	Unit 1 CTMT Primary Manway Platform, B Side Contamination Survey	10/05/2020
	Radiation Work Permits (RWPs)	20-1062	RCS Leak Repair	02
71151	Miscellaneous		MSPI Margin Reports; Units 1 and 2	10/01/2019-09/30/2020
			MSPI Derivation Reports; Residual Heat Removal, Cooling Water; Units 1 and 2	10/01/2019-09/30/2020
		NP 5.2.16 Attachment B	PI Data Collection, Review and Approval; RCS Activity-Fuel Performance	Various
		NP 5.2.16 Attachment B	PI Data Collection, Review and Approval; ODCM Rad. Eff. Occurrences	Various

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		NP 5.2.16 Attachment B	PI Data Collection, Review and Approval; Occ. Exp. Cont. Effectiveness	Various
	Procedures	PBN-BFJR-18-054	MSPI Basis Document for PBNP	26
71152	Corrective Action Documents	AR 2350536	Unit 2 High RHR Flow	03/30/2020
		AR 2350547	2RH-624 Opened with No Flow Response	03/30/2020
		AR 2362363	Inspect 2RH-624 Shaft to Lever Arm Keyways	07/10/2020
		AR 2369621	Part 21: Nutherm Continuously Energized Eaton D26 Relays	09/29/2020
		AR 2373438	Additional Action Required for Part 21 - AR 2369621	10/27/2020
		AR 2375752	Adverse Trend: HU Issues in 2020	11/16/2020
		CAP 14698	Sheared Woodruff Key in 2-RH00624	10/22/1991
	Corrective Action Documents Resulting from Inspection	AR 2359462	MRFF Needed for AR 2350547	06/10/2020
		AR 2361516	1993 Part 21 Documentation Ambiguous	07/01/2020
	Drawings	H-11051 Sheet 1	8IN. 300# Type 7613 AOV W/Extension	1
	Miscellaneous		Trend ARs 06-02-2020 thru 11-30-2020	12/01/2020
	Work Orders	WO 346830	2RH-00624; Disassemble and Inspect Valve	04/25/2008
		WO 391386	2RH-00624: License Renewal Follow-up Inspection	03/27/2011
WO 40712503		2RH-624 Opened with No Flow Response	04/01/2020	
71153	Corrective Action Documents	AR 2346369	Mode Panel Switches on New NUSI TMD Modules Faulty	02/27/2020
		AR 2351694	2TI-407A Blue Channel Setpoint 1 Failure	04/07/2020
		AR 2352490	Replace Setpoint 1 Yellow Channel (2TM-404B)	04/13/2020