



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 19, 2021

Mr. Christopher P. Domingos
Site Vice President
Prairie Island Nuclear Generating Plant
Northern States Power Company - Minnesota
1717 Wakonade Drive East
Welch, MN 55089

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 -
ISSUANCE OF AMENDMENTS NOS. 236 and 224 RE: LOW TEMPERATURE
OVERPRESSURE PROTECTION (EPID L-2020-LLA-0061)

Dear Mr. Domingos:

The U.S. Nuclear Regulatory Commission (NRC, the Commission) has issued the enclosed Amendment No. 236 to Renewed Facility Operating License No. DPR-42 and Amendment No. 224 to Renewed Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Units 1 and 2 (Prairie Island), respectively. The amendments consist of changes to the technical specifications (TSs) in response to your application dated March 30, 2020.

The amendments modify the TSs to remove Note 1 from both TS 3.4.12, "Low Temperature Overpressure Protection (LTOP) - Reactor Coolant System Cold Leg Temperature (RCSCLT) > Safety Injection (SI) Pump Disable Temperature" and LCO 3.4.13, "Low Temperature Overpressure Protection (LTOP) - Reactor Coolant System Cold Leg Temperature (RCSCLT) ≤ Safety Injection (SI) Pump Disable Temperature."

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's monthly *Federal Register* notice.

Sincerely,

/RA/

Robert F. Kuntz, Senior Project Manager
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

Enclosures:

1. Amendment No. 236 to DPR-42
2. Amendment No. 224 to DPR-60
3. Safety Evaluation

cc: Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
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NORTHERN STATES POWER COMPANY

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 236
Renewed License No. DPR-42

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company, a Minnesota Corporation (NSPM, the licensee), dated March 30, 2020, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-42 is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendix A, as revised through Amendment No. 236, are hereby incorporated in the renewed operating license. NSPM shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Nancy L. Salgado, Chief
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License and Technical
Specifications

Date of Issuance: March 19, 2021



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 224
Renewed License No. DPR-60

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company, a Minnesota Corporation (NSPM, the licensee), dated March 30, 2020, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-60 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 224, are hereby incorporated in the renewed operating license. NSPM shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Nancy L. Salgado, Chief
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License and Technical
Specifications

Date of Issuance: March 19, 2021

ATTACHMENT TO LICENSE AMENDMENT NOS. 236 AND 224

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2

RENEWED FACILITY OPERATING LICENSE NOS. DPR-42 AND DPR-60

DOCKET NOS. 50-282 AND 50-306

Replace the following pages of the Renewed Facility Operating License Nos. DPR-42 and DPR-60 with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicate the areas of change.

Renewed Facility Operating License No. DPR-42

REMOVE

INSERT

Page 3

Page 3

Renewed Facility Operating License No. DPR-60

REMOVE

INSERT

Page 3

Page 3

Technical Specifications

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

INSERT

3.4.12-1

3.4.12-1

3.4.13-1

3.4.13-1

- (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NSPM to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, NSPM to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
 - (5) Pursuant to the Act and 10 CFR Parts 30 and 70, NSPM to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility;
 - (6) Pursuant to the Act and 10 CFR Parts 30 and 70, NSPM to transfer byproduct materials from other job sites owned by NSPM for the purpose of volume reduction and decontamination.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

NSPM is authorized to operate the facility at steady state reactor core power levels not in excess of 1677 megawatts thermal.
 - (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 236, are hereby incorporated in the renewed operating license. NSPM shall operate the facility in accordance with the Technical Specifications.
 - (3) Physical Protection

NSPM shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contains

- (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NSPM to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, NSPM to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
 - (5) Pursuant to the Act and 10 CFR Parts 30 and 70, NSPM to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility;
 - (6) Pursuant to the Act and 10 CFR Parts 30 and 70, NSPM to transfer byproduct materials from other job sites owned by NSPM for the purposes of volume reduction and decontamination.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

NSPM is authorized to operate the facility at steady state reactor core power levels not in excess of 1677 megawatts thermal.
 - (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 224, are hereby incorporated in the renewed operating license. NSPM shall operate the facility in accordance with the Technical Specifications.
 - (3) Physical Protection

NSPM shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contains

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.12 Low Temperature Overpressure Protection (LTOP) –Reactor Coolant System Cold Leg Temperature (RCSCLT) > Safety Injection (SI) Pump Disable Temperature

LCO 3.4.12 LTOP shall be provided with:

- a. A maximum of one SI pump capable of injecting into the RCS;
- b. The emergency core cooling system (ECCS) accumulators isolated;
- c. An OPERABLE Over Pressure Protection System (OPPS) with lift setting within the limits specified in the PTLR; and
- d. Two OPERABLE pressurizer power operated relief valves (PORVs).

-----NOTE-----

1. ECCS accumulator may be unisolated when accumulator pressure is less than the maximum RCS pressure for the existing RCS cold leg temperature allowed by the P/T limit curves provided in the PTLR.
-

APPLICABILITY: MODE 4 when any RCS cold leg temperature is \leq the OPPS enable temperature specified in the PTLR and $>$ the SI pump disable temperature specified in the PTLR.

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.13 Low Temperature Overpressure Protection (LTOP) – Reactor Coolant System Cold Leg Temperature (RCSCLT) \leq Safety Injection (SI) Pump Disable Temperature

LCO 3.4.13 LTOP shall be provided with: 1) no SI Pumps capable of injecting into the RCS; 2) the emergency core cooling system (ECCS) accumulators isolated; and 3) one of the following pressure relief capabilities:

- a. An Over Pressure Protection System (OPPS) shall be OPERABLE with two pressurizer power operated relief valves (PORVs) with lift settings within the limits specified in the PTLR; or
- b. The RCS depressurized and an RCS vent of ≥ 3 square inches.

-----NOTES-----

- 1. During reduced inventory conditions an SI pump may be run as required to maintain adequate core cooling and RCS inventory.
- 2. ECCS accumulator may be unisolated when ECCS accumulator pressure is less than the maximum RCS pressure for the existing RCS cold leg temperature allowed by the P/T limit curves provided in the PTLR.

APPLICABILITY: MODE 4 when any RCS cold leg temperature is \leq the SI Pump disable temperature specified in the PTLR,
 MODE 5 when the steam generator (SG) primary system manway and pressurizer manway are closed and secured in position,
 MODE 6 when the reactor vessel head is on and the SG primary system manway and pressurizer manways are closed and secured in position.



UNITED STATES
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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 236 TO RENEWED FACILITY

OPERATING LICENSE NO. DPR-42

AND AMENDMENT NO. 224 TO RENEWED FACILITY

OPERATING LICENSE NO. DPR-60

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2

DOCKET NOS. 50-282 AND 50-306

1.0 INTRODUCTION

By application dated March 30, 2020 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML20090G066), Northern States Power Company, a Minnesota Corporation (NSPM, the licensee), requested changes to the technical specifications (TSs) for Prairie Island Nuclear Generating Plant, Units 1 and 2 (Prairie Island).

The proposed changes would modify the TSs to remove Note 1 from both TS 3.4.12, "Low Temperature Overpressure Protection (LTOP) - Reactor Coolant System Cold Leg Temperature (RCSCLT) > Safety Injection (SI) Pump Disable Temperature" and LCO 3.4.13, "Low Temperature Overpressure Protection (LTOP) - Reactor Coolant System Cold Leg Temperature (RCSCLT) ≤ Safety Injection (SI) Pump Disable Temperature."

2.0 REGULATORY EVALUATION

2.1 Proposed Changes

The current Prairie Island TS 3.4.12, Limiting Condition for Operation (LCO), NOTE 1 states:

Both SI pumps may be run for ≤ 1 hour while conducting SI system testing provided there is a steam or gas bubble in the pressurizer and at least one isolation valve between the SI pump and the RCS [reactor coolant system] is shut.

The current Prairie Island TS 3.4.13 LCO NOTE 1 states:

Both safety injection (SI) pumps may be run for ≤ 1 hour while conducting SI system testing provided there is a steam or gas bubble in the pressurizer, the reactor vessel head is on, and at least one isolation valve between the SI pump and the RCS is shut.

NSPM proposed a revision to the Prairie Island TSs to remove NOTE 1 from both LCO 3.4.12 and LCO 3.4.13.

2.2 Regulatory Requirements

The categories of items required to be in the TSs are provided in Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.36(c). As required by 10 CFR 50.36(c)(2)(i), the TSs will include LCOs which are the lowest functional capability or performance levels of equipment required for safe operation of the facility. Per 10 CFR 50.36(c)(2)(i), when an LCO of a nuclear reactor is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the TSs until the condition can be met.

The 10 CFR Part 50, Appendix A, "General Design Criteria [GDC] for Nuclear Power Plants," specify minimum requirements for the principal design criteria that apply to water-cooled nuclear power plants. Prairie Island was not licensed to the 10 CFR 50, Appendix A, General Design Criteria (GDC). Prairie Island was designed and constructed to comply with NSPM's understanding of the intent of the Atomic Energy Commission (AEC) General Design Criteria for Nuclear Power Plant Construction Permits, as proposed on July 10, 1967. Since the construction of the plant was significantly completed prior to the issuance of the February 20, 1971, 10 CFR 50, Appendix A, GDC, the plant was not reanalyzed and the Updated Safety Analysis Report (USAR) was not revised to reflect these later criteria. However, the AEC Safety Evaluation Report (September 28, 1972) acknowledged that the principal design criteria as described in the Prairie Island USAR conform to the intent of the 10 CFR 50, Appendix A, GDC.

The applicable principal design criterion (PDC), as documented in PINGP USAR, Section 3.1.2, is:

- PDC 34, "Reactor Coolant Pressure Boundary Rapid Propagation Failure Prevention," which states, in part, "the reactor coolant pressure boundary shall be designed and operated to reduce to an acceptable level the probability of rapidly propagating type failure. Consideration is given (a) to the provisions for control over service temperature and irradiation effects which may require operational restrictions...."

3.0 TECHNICAL EVALUATION

3.1 Low Temperature Overpressurization and Safety Analysis

When the RCS is at low temperatures, especially lower than overpressure protection system (OPPS) enable temperature (analytical limit 225 Fahrenheit (°F), operating limit 310 °F), a low temperature OPPS will be activated to ensure the RCS pressure/temperature (P/T) limits of 10 CFR 50, Appendix G, are not exceeded during pressure transient events to protect the reactor pressure vessel from brittle fracture.

Transients that are capable of overpressurizing the RCS are categorized as either mass or heat input transients. The bounding mass input transient is inadvertent safety injection with injection from one SI pump and three charging pumps, and letdown isolated. The bounding heat input transient is reactor coolant pump (RCP) startup with temperature asymmetry within the RCS or between the RCS and steam generators (SGs).

At about the OPPS enable temperature, the overpressure prevention falls to two OPERABLE power-operated relief valves (PORVs) or to a depressurized RCS and a sufficiently sized RCS vent. Each of these means has a limited overpressure relief capability. LCO 3.4.12 and LCO 3.4.13 provide the requirements for overpressure prevention at the lower temperatures. The LTOP safety analyses described in Prairie Island USAR, Section 4.4.3.3, demonstrate that one PORV can maintain RCS pressure below limits when only one SI pump and all charging pumps are actuated.

3.2 TS LCO 3.4.12, LCO 3.4.13 and NOTE 1

When the plant operates in MODE 4 and any RCS cold leg temperature is less than or equal to the OPPS enable temperature specified in the Pressure Temperature Limits Report (PTLR) and greater than the SI pump disable temperature specified in the PTLR, per LCO 3.4.12 the following are required in order to provide LTOP:

- a. a maximum of one SI pump capable of injecting into the RCS;
- b. the emergency core cooling system (ECCS) accumulators isolated;
- c. an OPERABLE OPPS with lift setting within the limits specified in the PTLR; and
- d. two operable PORVs.

LCO 3.4.13 is applicable in the following conditions:

- a. MODE 4 and when any RCS cold leg temperature is \leq the SI pump disable temperature specified in the PTLR, or
- b. MODE 5 when SG primary system manway and pressurizer manway are closed and secured in position, or
- c. MODE 6 when the reactor vessel head is on and the SG primary system manway and pressurizer manways are closed and secured in position.

LCO 3.4.13 requires the following to provide LTOP:

- 1) no SI Pumps capable of injecting into the RCS;
- 2) the ECCS accumulators isolated; and

- 3) One of the following pressure relief capabilities:
 - a. An OPPS shall be operable with two PORVs with lift settings within the limits specified in the PTLR; or
 - b. The RCS depressurized and an RCS vent of ≥ 3 square inches.

The requirements of LCOs 3.4.12 and 3.4.13 ensure that mass and heat input transients in excess of analysis assumptions do not occur.

These two LCOs require the reactor vessel be protected by maintaining its pressure and temperature within the plants' PTLR so that the integrity of the reactor coolant pressure boundary will not be compromised by violating the P/T limits of 10 CFR 50, Appendix G. LTOP is most critical during shutdown when the RCS is water solid, and a mass or heat input transient can cause a very rapid increase in RCS pressure when little or no time allows operator action to mitigate the event.

NOTE 1 of both LCO 3.4.12 and LCO 3.4.13 allow operation of both SI pumps for ≤ 1 hour for conducting SI system testing providing there is a steam or gas bubble in the pressurizer and at least one isolation valve between the SI pump and the RCS is shut.

TS Bases for NOTE 1 states that the purpose of this note is to permit the conduct of the integrated SI test and other SI system tests and operations that may be performed in MODE 4 for LCO 3.4.12 and MODES 4, 5 or 6 for LCO 3.4.13. In this case, pressurizer level is maintained at less than 50 percent and a positive means of isolation is provided between the SI pumps and the RCS to prevent fluid injection to the RCS.

Existing NOTE 1 states that the conduct of SI system testing under the low RCS temperature condition is allowable, however, subject to certain conditions to prevent the reactor pressure vessel from low RCS temperature overpressurization. The NRC staff determined the following from its evaluation of the proposed TS change:

1. The SI testing under low RCS temperature condition is permitted by existing NOTE 1. However, such testing is not required by an associated ACTION statement or Surveillance Requirement. The SI system, as an ECCS subsystem, is required by TS 3.5.2 and TS 3.5.3 to be OPERABLE with associated ACTION statements and Surveillance Requirements.
2. The proposed removal of NOTE 1 from TS LCO 3.4.12 and LCO 3.4.13 requirements is consistent with the Prairie Island design and preserves the applicable Prairie Island USAR LTOP analysis because removal of the SI testing allowed by existing NOTE 1 to these LCOs does not alter any assumptions of RCS OPPS performance in the USAR Section 4.3.3.3 LTOP analysis. Therefore, the proposed changes will not adversely affect the Prairie Island OPPS capability to perform a required safety function.

Based on 1 above, the proposed TS change (i.e., removal of NOTE 1) is acceptable because TS 3.5.2 and TS 3.5.3 continue to require that the SI system be tested periodically for operability and required functional performance. Based on 2 above, the proposed TS change will continue to meet PDC 34 because the removal of SI testing under conditions allowed by NOTE 1 will not adversely affect the LTOP requirements for Prairie Island.

The changes continue to provide assurance that the RCS can be maintained below P/T limits during transient events to protect the reactor pressure vessel from brittle fracture.

3.3 Technical Evaluation Conclusion

Based on the above evaluation, the NRC staff finds that the proposed TS change will continue to meet the applicable regulatory requirements in 10 CFR 50.36(c)(2). Because the removal of the note does not disturb the remaining TS requirements to maintain LTOP protection, the lowest functional capability and performance levels of equipment required for safe operation of the facility is maintained. Therefore, these LCOs, as revised, continue to meet the requirements of 10 CFR 50.36(c)(2)(i). The NRC staff, therefore, concludes that the TS change proposed by this license amendment request is acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State official was notified of the proposed issuance of the amendments on January 7, 2020. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change the requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding published in the *Federal Register* on May 19, 2020 (85 FR 29985). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Peng, NRR

Date of Issuance: March 19, 2021

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 -
ISSUANCE OF AMENDMENTS RE: LOW TEMPERATURE OVERPRESSURE
PROTECTION (EPID L-2020-LLA-0061) DATED MARCH 19, 2021

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