

Facility: LaSalle County Station		Date of Exam: November 2020															
Tier	Group	RO K/A Category Points												SRO-Only Points			
		K1	K2	K3	K4	K5	K6	A1	A2	A3	A4	G*	Total	A2	G*	Total	
1. Emergency and Abnormal Plant Evolutions	1	3	3	3	N/A			3	4	N/A			4	20	4	3	7
	2	1	2	1	N/A			1	1	N/A			1	7	2	1	3
	Tier Totals	4	5	4	N/A			4	5	N/A			5	27	6	4	10
2. Plant Systems	1	2	2	2	2	2	3	3	3	3	2	2	26	2	3	5	
	2	1	1	1	2	1	1	1	1	1	1	1	12	0	2	3	
	Tier Totals	3	3	3	4	3	4	4	4	4	3	3	38	4	4	8	
3. Generic Knowledge and Abilities Categories					1	2	3	4	10			1	2	3	4	7	
					2	3	2	3				2	2	1	2		

- Note: 1. Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO-only outline sections (i.e., except for one category in Tier 3 of the SRO-only section, the "Tier Totals" in each K/A category shall not be less than two). (One Tier 3 radiation control K/A is allowed if it is replaced by a K/A from another Tier 3 category.)
2. The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by ±1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points, and the SRO-only exam must total 25 points.
3. Systems/evolutions within each group are identified on the outline. Systems or evolutions that do not apply at the facility should be deleted with justification. Operationally important, site-specific systems/evolutions that are not included on the outline should be added. Refer to Section D.1.b of ES-401 for guidance regarding the elimination of inappropriate K/A statements.
4. Select topics from as many systems and evolutions as possible. Sample every system or evolution in the group before selecting a second topic for any system or evolution.
5. Absent a plant-specific priority, only those K/As having an importance rating (IR) of 2.5 or higher. Use the RO and SRO ratings for the RO and SRO-only portions, respectively.
6. Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories.
7. The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A catalog, but the topics must be relevant to the applicable evolution or system. Refer to Section D.1.b of ES-401 for the applicable K/As.
8. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' IRs for the applicable license level, and the point totals (#) for each system and category. Enter the group and tier totals for each category in the table above. If fuel-handling equipment is sampled in a category other than Category A2 or G\* on the SRO-only exam, enter it on the left side of Column A2 for Tier 2, Group 2. (Note 1 does not apply.) Use duplicate pages for RO and SRO-only exams.
9. For Tier 3, select topics from Section 2 of the K/A catalog, and enter the K/A numbers, descriptions, IRs, and point totals (#) on Form ES-401-3. Limit SRO selections to K/As that are linked to 10 CFR 55.43.

G\* Generic K/As

- \* These systems/evolutions must be included as part of the sample (as applicable to the facility) when Revision 3 of the K/A catalog is used to develop the sample plan. They are not required to be included when using earlier revisions of the K/A catalog.
- \*\* These systems/evolutions may be eliminated from the sample (as applicable to the facility) when Revision 3 of the K/A catalog is used to develop the sample plan.

ES-401		BWR Examination Outline Emergency and Abnormal Plant Evolutions—Tier 1/Group 1 (RO)						Form ES-401-1	
E/APE # / Name / Safety Function	K1	K2	K3	A1	A2	G*	K/A Topic(s)	IR	#
295001 (APE 1) Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4		02					Knowledge of the interrelations between partial or complete loss of forced core flow circulation and the following: Nuclear boiler instrumentation.	3.2	1
295003 (APE 3) Partial or Complete Loss of AC Power / 6			06				Knowledge of the reasons for the following responses or actions as they apply to partial or complete loss of AC power: Containment isolation.	3.7	2
295004 (APE 4) Partial or Total Loss of DC Power / 6				03			Ability to operate and/or monitor the following as they apply to partial or complete loss of DC power: AC Electrical Distribution. <a href="#">KA change to 295004 AA1.02 (3.8)</a>	3.4	3
295005 (APE 5) Main Turbine Generator Trip / 3					06		Ability to determine and/or interpret the following as they apply to main turbine generator trip: Feedwater temperature.	2.6	4
295006 (APE 6) Scram / 1						4.4	Ability to recognize abnormal indications for system operating parameters that are entry-level conditions for Emergency and Abnormal Operating Procedures.	4.5	5
295016 (APE 16) Control Room Abandonment / 7		01					Knowledge of the interrelations between Control Room abandonment and the following: Remote shutdown panel.	4.4	6
295018 (APE 18) Partial or Complete Loss of CCW / 8			07				Knowledge of the reasons for the following responses as they apply to partial or complete loss of component cooling water: Cross-connecting with backup systems.	3.1	7
295019 (APE 19) Partial or Complete Loss of Instrument Air / 8				02			Ability to operate and/or monitor the following as they apply to a partial or complete loss of instrument air: Instrument air system valves. <a href="#">KA change to 295019 AA1.03 (3.0)</a>	3.3	8
295021 (APE 21) Loss of Shutdown Cooling / 4	01						Knowledge of the operational implications of the following concepts as they apply to loss of shutdown cooling: Decay heat.	3.6	9
295023 (APE 23) Refueling Accidents / 8					03		Ability to determine and/or interpret the following as they apply to refueling accidents: Airborne contamination levels.	3.3	10
295024 High Drywell Pressure / 5						2.42	Ability to recognize system parameters that are entry-level conditions for Technical Specifications.	3.9	11
295025 (EPE 2) High Reactor Pressure / 3	06						Knowledge of the operational implications of the following concepts as they apply to high reactor pressure: Pressure effects on reactor water level.	3.5	12
295026 (EPE 3) Suppression Pool High Water Temperature / 5		02					Knowledge of the interrelations between suppression pool high water temperature and the following: Suppression pool spray.	3.6	13
295027 (EPE 4) High Containment Temperature (Mark III Containment Only) / 5							Not sampled, N/A for LaSalle (Mark II Containment).		
295028 (EPE 5) High Drywell Temperature (Mark I and Mark II only) / 5			02				Knowledge of the reasons for the following responses as they apply to high drywell temperature: RPV flooding.	3.5	14
295030 (EPE 7) Low Suppression Pool Water Level / 5				03			Ability to operate and/or monitor the following as they apply to low suppression pool water level: HPCS.	3.4	15
295031 (EPE 8) Reactor Low Water Level / 2					04		Ability to determine and/or interpret the following as they apply to reactor low water level: Adequate core cooling.	4.6	16
295037 (EPE 14) Scram Condition Present and Reactor Power Above APRM Downscale or Unknown / 1						1.31	Ability to locate control room switches, controls, and indications, and to determine that they correctly reflect the desired plant lineup.	4.6	17
295038 (EPE 15) High Offsite Radioactivity Release Rate / 9					04		Ability to determine and/or interpret the following as they apply to high off-site release rate: Source of off-site release.	4.1	18
600000 (APE 24) Plant Fire On Site / 8						1.20	Ability to interpret and execute procedure steps.	4.6	19
700000 (APE 25) Generator Voltage and Electric Grid Disturbances / 6	02						Knowledge of the operational implications of the following concepts as they apply to generator voltage and electric grid disturbances: Over-excitation.	3.3	20
K/A Category Totals:	3	3	3	3	4	4	Group Point Total:		20

ES-401		BWR Examination Outline Emergency and Abnormal Plant Evolutions—Tier 1/Group 2 (RO)						Form ES-401-1	
E/APE # / Name / Safety Function	K1	K2	K3	A1	A2	G*	K/A Topic(s)	IR	#
295002 (APE 2) Loss of Main Condenser Vacuum / 3									
295007 (APE 7) High Reactor Pressure / 3					02		Ability to determine and/or interpret the following as they apply to high reactor pressure: Reactor power.	4.1	21
295008 (APE 8) High Reactor Water Level / 2									
295009 (APE 9) Low Reactor Water Level / 2									
295010 (APE 10) High Drywell Pressure / 5									
295011 (APE 11) High Containment Temperature (Mark III Containment only) / 5							Not sampled, N/A for LaSalle (Mark II Containment).		
295012 (APE 12) High Drywell Temperature / 5						4.20	Knowledge of the operational implications of EOP warnings, cautions, and notes.	3.8	22
295013 (APE 13) High Suppression Pool Temperature / 5									
295014 (APE 14) Inadvertent Reactivity Addition / 1	05						Knowledge of the operational implications of the following concepts as they apply to inadvertent reactivity addition: Fuel thermal limits.	3.7	23
295015 (APE 15) Incomplete Scram / 1									
295017 (APE 17) Abnormal Offsite Release Rate / 9									
295020 (APE 20) Inadvertent Containment Isolation / 5 & 7		01					Knowledge of the interrelations between inadvertent containment isolation and the following: Main steam system.	3.6	24
295022 (APE 22) Loss of Control Rod Drive Pumps / 1		03					Knowledge of the interrelations between loss of CRD pumps and the following: Accumulator pressures.	3.4	25
295029 (EPE 6) High Suppression Pool Water Level / 5			03				Knowledge of the reasons for the following responses as they apply to high suppression pool water level: Reactor SCRAM.	3.4	26
295032 (EPE 9) High Secondary Containment Area Temperature / 5									
295033 (EPE 10) High Secondary Containment Area Radiation Levels / 9									
295034 (EPE 11) Secondary Containment Ventilation High Radiation / 9									
295035 (EPE 12) Secondary Containment High Differential Pressure / 5				02			Ability to operate and/or monitor the following as they apply to secondary containment high differential pressure: SBTG/FRVS.	3.8	27
295036 (EPE 13) Secondary Containment High Sump/Area Water Level / 5									
500000 (EPE 16) High Containment Hydrogen Concentration / 5									
K/A Category Point Totals:	1	2	1	1	1	1	Group Point Total:		7

ES-401		BWR Examination Outline Plant Systems—Tier 2/Group 1 (RO)											Form ES-401-1		
System # / Name	K1	K2	K3	K4	K5	K6	A1	A2	A3	A4	G*	K/A Topic(s)	IR	#	
203000 (SF2, SF4 RHR/LPCI) RHR/LPCI: Injection Mode												2.39	Knowledge of less than or equal to one hour Technical Specification action statements for systems.	3.9	28
205000 (SF4 SCS) Shutdown Cooling	05												Knowledge of the physical connections and/or cause-effect relationships between shutdown cooling system (RHR shutdown cooling mode) and the following: Component cooling water systems.	3.1	29
206000 (SF2, SF4 HPCIS) High-Pressure Coolant Injection													Not sampled, N/A for LaSalle (BWR-5).		
207000 (SF4 IC) Isolation (Emergency) Condenser													Not sampled, N/A for LaSalle (BWR-5).		
209001 (SF2, SF4 LPCS) Low-Pressure Core Spray		01				02							Knowledge of electrical power supplies to the following: Pump power; Knowledge of the effect that a loss or malfunction of the following will have on the low pressure core spray system: Emergency generators.	3.0	30
209002 (SF2, SF4 HPCS) High-Pressure Core Spray			01				04						Knowledge of the effect that a loss or malfunction of the high pressure core spray system will have on the following: Reactor water level;	3.9	32
													Ability to predict and/or monitor changes in parameters associated with operating the high pressure core spray system controls including: Reactor pressure.	3.3	33
211000 (SF1 SLCS) Standby Liquid Control				07				03					Knowledge of standby liquid control system design feature(s) and/or interlocks which provide for the following: RWCU isolation;	3.8	34
													Ability to (a) predict the impacts of the following on the standby liquid control system; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal operations: AC power failures.	3.2	35
212000 (SF7 RPS) Reactor Protection					02				05				Knowledge of the operational implications of the following concepts as they apply to the reactor protection system: Specific logic arrangements;	3.3	36
													Ability to monitor automatic operations of the reactor protection system including: SCRAM instrument volume level.	3.9	37
215003 (SF7 IRM) Intermediate-Range Monitor						05				03			Knowledge of the effect that a loss or malfunction of the following will have on the intermediate range monitor system: Trip units;	3.1	38
													Ability to manually operate and/or monitor in the control room: IRM range switches.	3.6	39
215004 (SF7 SRMS) Source-Range Monitor							03						Ability to predict and/or monitor changes in parameters associated with operating the source range monitor system controls including: RPS status.	3.4	40
215005 (SF7 PRMS) Average Power Range Monitor/Local Power Range Monitor								02					Ability to (a) predict the impacts of the following on the average power range monitor/local power range monitor system; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Upscale or downscale trips.	3.6	41
217000 (SF2, SF4 RCIC) Reactor Core Isolation Cooling									04				Ability to monitor automatic operations of the reactor core isolation cooling system including: System flow.	3.6	42
218000 (SF3 ADS) Automatic Depressurization										01			Ability to manually operate and/or monitor in the control room: ADS valves.	4.4	43
223002 (SF5 PCIS) Primary Containment Isolation/Nuclear Steam Supply Shutoff												1.28	Knowledge of the purpose and function of major system components and controls.	4.1	44
239002 (SF3 SRV) Safety Relief Valves		01											Knowledge of electrical power supplies to the following: SRV solenoids.	2.8	45
259002 (SF2 RWLCS) Reactor Water Level Control	15												Knowledge of the physical connections and/or cause-effect relationships between Reactor Water Level Control System and the following: Recirculation flow control system	3.2	46

261000 (SF9 SGTS) Standby Gas Treatment			01															Knowledge of the effect that a loss or malfunction of the standby gas treatment system will have on the following: Secondary containment and environment differential pressure. <a href="#">KA change to 261000 K3.03 (3.2)</a>	3.3	47
262001 (SF6 AC) AC Electrical Distribution			03															Knowledge of AC electrical distribution design feature(s) and/or interlocks which provide for the following: Interlocks between automatic bus transfer and breakers.	3.1	48
262002 (SF6 UPS) Uninterruptable Power Supply (AC/DC)							02											Knowledge of the effect that a loss or malfunction of the following will have on the uninterruptable power supply (AC/DC): DC electrical power.	2.8	49
263000 (SF6 DC) DC Electrical Distribution							01											Knowledge of the operational implications of the following concepts as they apply to DC electrical distribution: Hydrogen generation during battery charging.	2.6	50
264000 (SF6 EGE) Emergency Generators (Diesel/Jet) EDG										09								Ability to predict and/or monitor changes in parameters associated with operating the emergency generators controls including: Maintaining minimum load on emergency generator (to prevent reverse power).	3.0	51
300000 (SF8 IA) Instrument Air										01								Ability to (a) predict the impacts of the following on the instrument air system; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal operations: Air dryer and filter malfunctions.	2.9	52
400000 (SF8 CCS) Component Cooling Water											01							Ability to monitor automatic operations of the component cooling water system including: Setpoints on instrument level signals for normal operations, warnings, and trips that are applicable to the CCWS.	3.0	53
510000 (SF4 SWS*) Service Water (Normal and Emergency)																		Not sampled, currently in Rev. 2 of KA catalog.		
K/A Category Point Totals:	2	2	2	2	2	3	3	3	3	2	2							Group Point Total:		26



286000 (SF8 FPS) Fire Protection										12							Ability to (a) predict the impacts of the following on the fire protection system and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Low diesel fuel supply.	3.1	63
288000 (SF9 PVS) Plant Ventilation																			
290001 (SF5 SC) Secondary Containment											01						Ability to monitor automatic operations of the secondary containment including: Secondary containment isolation.	3.9	64
290003 (SF9 CRV) Control Room Ventilation																			
290002 (SF5 RVI) Reactor Vessel Internals				05													Knowledge of reactor vessel and internals design feature(s) and/or interlocks which provide for the following: Natural circulation.	3.3	65
51001 (SF8 CWS*) Circulating Water																	Not sampled, currently in Rev. 2 of KA catalog.		0
K/A Category Point Totals:	1	1	1	2	1	1	1	1	1	1	1	1	1	1	1	1	Group Point Total:		12

ES-401		BWR Examination Outline Emergency and Abnormal Plant Evolutions—Tier 1/Group 1 (SRO)						Form ES-401-1	
E/APE # / Name / Safety Function	K1	K2	K3	A1	A2	G*	K/A Topic(s)	IR	#
295001 (APE 1) Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4									
295003 (APE 3) Partial or Complete Loss of AC Power / 6									
295004 (APE 4) Partial or Total Loss of DC Power / 6					03		Ability to determine and/or interpret the following as they apply to partial or complete loss of DC power: Battery voltage.	2.9	76
295005 (APE 5) Main Turbine Generator Trip / 3						1.7	Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior, and instrument interpretation.	4.7	77
295006 (APE 6) Scram / 1									
295016 (APE 16) Control Room Abandonment / 7					05		Ability to determine and/or interpret the following as they apply to control room abandonment: Drywell pressure.	3.9	78
295018 (APE 18) Partial or Complete Loss of CCW / 8									
295019 (APE 19) Partial or Complete Loss of Instrument Air / 8						4.31	Knowledge of annunciators, alarms, indications, or response procedures.	4.1	79
295021 (APE 21) Loss of Shutdown Cooling / 4									
295023 (APE 23) Refueling Accidents / 8									
295024 High Drywell Pressure / 5									
295025 (EPE 2) High Reactor Pressure / 3					03		Ability to determine and/or interpret the following as they apply to high reactor pressure: Suppression pool temperature.	4.1	80
295026 (EPE 3) Suppression Pool High Water Temperature / 5									
295027 (EPE 4) High Containment Temperature (Mark III Containment Only) / 5							Not sampled, N/A for LaSalle (Mark II Containment).		
295028 (EPE 5) High Drywell Temperature (Mark I and Mark II only) / 5									
295030 (EPE 7) Low Suppression Pool Water Level / 5									
295031 (EPE 8) Reactor Low Water Level / 2						4.8	Knowledge of how abnormal operating procedures are used in conjunction with EOPs.	4.5	81
295037 (EPE 14) Scram Condition Present and Reactor Power Above APRM Downscale or Unknown / 1					07		Ability to determine and/or interpret the following as they apply to SCRAM condition present and reactor power above APRM downscale or unknown: Containment conditions/isolations.	4.2	82
295038 (EPE 15) High Offsite Radioactivity Release Rate / 9									
600000 (APE 24) Plant Fire On Site / 8									
700000 (APE 25) Generator Voltage and Electric Grid Disturbances / 6									
K/A Category Totals:					4	3	Group Point Total:		7



ES-401		BWR Examination Outline Emergency and Abnormal Plant Evolutions—Tier 1/Group 2 (SRO)						Form ES-401-1	
E/APE # / Name / Safety Function	K1	K2	K3	A1	A2	G*	K/A Topic(s)	IR	#
295002 (APE 2) Loss of Main Condenser Vacuum / 3					02		Ability to determine and/or interpret the following as they apply to loss of main condenser vacuum: Reactor power.	3.3	83
295007 (APE 7) High Reactor Pressure / 3									
295008 (APE 8) High Reactor Water Level / 2									
295009 (APE 9) Low Reactor Water Level / 2									
295010 (APE 10) High Drywell Pressure / 5						2.44	Ability to interpret control room indications to verify the status and operation of a system, and understand how operator actions and directives, affect plant and system conditions.	4.4	84
295011 (APE 11) High Containment Temperature (Mark III Containment only) / 5							Not sampled, N/A for LaSalle (BWR-5).		
295012 (APE 12) High Drywell Temperature / 5									
295013 (APE 13) High Suppression Pool Temperature. / 5									
295014 (APE 14) Inadvertent Reactivity Addition / 1									
295015 (APE 15) Incomplete Scram / 1									
295017 (APE 17) Abnormal Offsite Release Rate / 9									
295020 (APE 20) Inadvertent Containment Isolation / 5 & 7									
295022 (APE 22) Loss of Control Rod Drive Pumps / 1									
295029 (EPE 6) High Suppression Pool Water Level / 5									
295032 (EPE 9) High Secondary Containment Area Temperature / 5									
295033 (EPE 10) High Secondary Containment Area Radiation Levels / 9					03		Ability to determine and/or interpret the following as they apply to high secondary containment area radiation levels: Cause of high area radiation.	4.2	85
295034 (EPE 11) Secondary Containment Ventilation High Radiation / 9									
295035 (EPE 12) Secondary Containment High Differential Pressure / 5									
295036 (EPE 13) Secondary Containment High Sump/Area Water Level / 5									
500000 (EPE 16) High Containment Hydrogen Concentration / 5									
K/A Category Point Totals:					2	1	Group Point Total:		3

ES-401	BWR Examination Outline Plant Systems—Tier 2/Group 1 (SRO)											Form ES-401-1		
System # / Name	K1	K2	K3	K4	K5	K6	A1	A2	A3	A4	G*	K/A Topic(s)	IR	#
203000 (SF2, SF4 RHR/LPCI) RHR/LPCI: Injection Mode														
205000 (SF4 SCS) Shutdown Cooling														
206000 (SF2, SF4 HPCIS) High-Pressure Coolant Injection												Not sampled, N/A for LaSalle (BWR-5).		
207000 (SF4 IC) Isolation (Emergency) Condenser												Not sampled, N/A for LaSalle (BWR-5).		
209001 (SF2, SF4 LPCS) Low-Pressure Core Spray								05				Ability to (a) predict the impacts of the following on the LOW PRESSURE CORE SPRAY SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Core spray line break	3.6	89
209002 (SF2, SF4 HPCS) High-Pressure Core Spray														
211000 (SF1 SLCS) Standby Liquid Control														
212000 (SF7 RPS) Reactor Protection														
215003 (SF7 IRM) Intermediate-Range Monitor														
215004 (SF7 SRMS) Source-Range Monitor														
215005 (SF7 PRMS) Average Power Range Monitor/Local Power Range Monitor														
217000 (SF2, SF4 RCIC) Reactor Core Isolation Cooling														
218000 (SF3 ADS) Automatic Depressurization											2.25	Knowledge of the bases in the Technical Specifications for limiting conditions for operations and safety limits.	4.2	86
223002 (SF5 PCIS) Primary Containment Isolation/Nuclear Steam Supply Shutoff														
239002 (SF3 SRV) Safety Relief Valves								01				Ability to (a) predict the impacts of the following on the safety relief valves and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Stuck open vacuum breakers.	3.3	87
259002 (SF2 RWLCS) Reactor Water Level Control														
261000 (SF9 SGTS) Standby Gas Treatment														
262001 (SF6 AC) AC Electrical Distribution											4.50	Ability to verify system alarm setpoints and operate controls identified in the alarm response manual.	4.0	88
262002 (SF6 UPS) Uninterruptable Power Supply (AC/DC)								01				Ability to (a) predict the impacts of the following on the uninterruptable power supply (AC/DC) and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Undervoltage. <a href="#">KA change to 209001 A2.05 (3.6)</a>	2.8	89
263000 (SF6 DC) DC Electrical Distribution														
264000 (SF6 EGE) Emergency Generators (Diesel/Jet) EDG											2.40	Ability to apply Technical Specifications for a system.	4.7	90
300000 (SF8 IA) Instrument Air														
400000 (SF8 CCS) Component Cooling Water														
510000 (SF4 SWS*) Service Water (Normal and Emergency)												Not sampled, currently in Rev. 2 of KA catalog.		
K/A Category Point Totals:								2			3	Group Point Total:		5

ES-401	BWR Examination Outline Plant Systems—Tier 2/Group 2 (SRO)											Form ES-401-1		
System # / Name	K1	K2	K3	K4	K5	K6	A1	A2	A3	A4	G*	K/A Topic(s)	IR	#
201001 (SF1 CRDH) CRD Hydraulic														
201002 (SF1 RMCS) Reactor Manual Control														
201003 (SF1 CRDM) Control Rod and Drive Mechanism														
201004 (SF7 RSCS) Rod Sequence Control												Not sampled, N/A for LaSalle (BWR-5).		
201005 (SF1, SF7 RCIS) Rod Control and Information												Not sampled, N/A for LaSalle (BWR-5).		
201006 (SF7 RWMS) Rod Worth Minimizer														
202001 (SF1, SF4 RS) Recirculation														
202002 (SF1 RSCTL) Recirculation Flow Control								07				Ability to (a) predict the impacts of the following on recirculation flow control system and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Loss of feedwater signal inputs.	3.3	91
204000 (SF2 RWCU) Reactor Water Cleanup														
214000 (SF7 RPIS) Rod Position Information														
215001 (SF7 TIP) Traversing In-Core Probe														
215002 (SF7 RBMS) Rod Block Monitor														
216000 (SF7 NBI) Nuclear Boiler Instrumentation														
219000 (SF5 RHR SPC) RHR/LPCI: Torus/Suppression Pool Cooling Mode														
223001 (SF5 PCS) Primary Containment and Auxiliaries											4.46	Ability to verify that the alarms are consistent with the plant conditions.	4.2	92
226001 (SF5 RHR CSS) RHR/LPCI: Containment Spray Mode											4.46	Ability to verify that the alarms are consistent with the plant conditions. <a href="#">KA change to 223001 G2.4.46 (4.2)</a>	4.2	92
230000 (SF5 RHR SPS) RHR/LPCI: Torus/Suppression Pool Spray Mode														
233000 (SF9 FPCCU) Fuel Pool Cooling/Cleanup														
234000 (SF8 FH) Fuel-Handling Equipment														
239001 (SF3, SF4 MRSS) Main and Reheat Steam														
239003 (SF9 MSVLCS) Main Steam Isolation Valve Leakage Control												Not sampled, N/A for LaSalle (BWR-5).		
241000 (SF3 RTPRS) Reactor/Turbine Pressure Regulating														
245000 (SF4 MTGEN) Main Turbine Generator/Auxiliary														
256000 (SF2 CDS) Condensate														
259001 (SF2 FWS) Feedwater														
268000 (SF9 RW) Radwaste														
271000 (SF9 OG) Offgas														
272000 (SF7, SF9 RMS) Radiation Monitoring														
286000 (SF8 FPS) Fire Protection														
288000 (SF9 PVS) Plant Ventilation														
290001 (SF5 SC) Secondary Containment														
290003 (SF9 CRV) Control Room Ventilation								02				Ability to (a) predict the impacts of the following on the control room HVAC and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Extreme environmental conditions.	3.4	93
290002 (SF4 RVI) Reactor Vessel Internals														
51001 (SF8 CWS*) Circulating Water												Not sampled, currently in Rev. 2 of KA catalog.		

K/A Category Point Totals:								2			1	Group Point Total:			3
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Facility: LaSalle County Station		Date of Exam: November 2020				
Category	K/A #	Topic	RO		SRO-only	
			IR	#	IR	#
1. Conduct of Operations	2.1.38	Knowledge of the station's requirements for verbal communications when implementing procedures.	3.7	66		
	2.1.44	Knowledge of RO duties in the control room during fuel handling such as responding to alarms from the fuel handling area, communication with the fuel storage facility, systems operated from the control room in support of fueling operations and supporting instrumentation.	3.9	67		
	2.1.5	Ability to use procedures related to shift staffing, such as minimum crew complement, overtime limitations, etc.			3.9	94
	2.1.23	Ability to perform specific system and integrated plant procedures during all modes of plant operation.			4.4	95
	Subtotal			2		2
2. Equipment Control	2.2.14	Knowledge of the process for controlling equipment configuration or status.	3.9	68		
	2.2.22	Knowledge of limiting conditions for operations and safety limits.	4.0	69		
	2.2.43	Knowledge of the process used to track inoperable alarms.	3.0	70		
	2.2.17	Knowledge of the process for managing maintenance activities during power operations, such as risk assessments, work prioritization, and coordination with the transmission system operator.			3.8	96
	2.2.21	Knowledge of pre- and post-maintenance operability requirements.			4.1	97
	Subtotal			3		2
3. Radiation Control	2.3.5	Ability to use radiation monitoring systems, such as fixed radiation monitors and alarms, portable survey instruments, personnel monitoring equipment, etc.	2.9	71		
	2.3.12	Knowledge of radiological safety principles pertaining to licensed operator duties, such as containment entry requirements, fuel handling responsibilities, access to locked high-radiation areas, or aligning filters.	3.2	72		
	2.3.13	Knowledge of radiological safety procedures pertaining to licensed operator duties, such as response to radiation monitor alarms, containment entry requirements, fuel handling responsibilities, access to locked high-radiation areas, aligning filters, etc.			3.8	98
	2.3.					
	2.3.					
	2.3.					
	Subtotal			2		1
4. Emergency Procedures/Plan	2.4.14	Knowledge of general guidelines for EOP usage.	3.8	73		
	2.4.34	Knowledge of RO tasks performed outside the main control room during an emergency and the resultant operational effects. <a href="#">KA change to 2.4.37 (3.0)</a>	4.2	74		
	2.4.35	Knowledge of local auxiliary operator tasks during an emergency and the resultant operational effects. <a href="#">KA change to 2.4.12 (4.0)</a>	3.8	75		
	2.4.5	Knowledge of the organization of the operating procedures network for normal, abnormal, and emergency evolutions.			4.3	99
	2.4.29	Knowledge of the Emergency Plan.			4.4	100
	Subtotal			3		2
Tier 3 Point Total				10		7