Security Related Information - Withhold Under 10 CFR 2.390 This letter is decontrolled when separated from Enclosure 2



Tennessee Valley Authority, Sequoyah Nuclear Plant, P.O. Box 2000, Soddy Dalsy, Tennessee 37384

October 28, 2020

10 CFR 50.4 10 CFR 50.71(e) 10 CFR 54.37(b)

ATTN: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555-0001

Sequoyah Nuclear Plant, Units 1 and 2

Renewed Facility Operating License Nos. DPR-77 and DPR-79

NRC Docket Nos. 50-327 and 50-328

Subject: Updated Final Safety Analysis Report Amendment 29

Reference: TVA letter to NRC, Updated Final Safety Analysis Report Amendment 28," dated

June 5, 2019

Consistent with the requirements of 10 CFR 50.4(b)(6) and 10 CFR 50.71(e), enclosed for Sequoyah Nuclear Plant (SQN) Units 1 and 2, are two compact discs (CD-ROM) that contain Amendment 29 of the Updated Final Safety Analysis Report (UFSAR).

The Tennessee Valley Authority (TVA) has marked this Amendment for redaction as indicated by Regulatory Issue Summary 2015-17, "Review and Submission of updates to Final Safety Analysis Reports, Emergency Preparedness Documents, and Fire Protection Documents," dated December 23, 2015, using the criteria provided by the Nuclear Regulatory Commission (NRC) in SECY-04-191.

Enclosure 1 to this letter is a copy redacted according to the marking for NRC Public Use. Enclosure 2 provides the Non-Public Use version, which is marked, but not redacted. TVA has marked the information as "Security-Related Information - Withhold Under 10 CFR 2.390."

Internally to TVA, access to the UFSAR will not be restricted.

A brief description and summary of the 10 CFR 50.59 safety evaluations of changes, tests, and experiments, reportable pursuant to 10 CFR 50.59 is being submitted under separate cover letter.

D030 A053 NRR

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Since the referenced letter, in accordance with 54.37(b), TVA has revised Appendix A of SQN UFSAR Section 13.9 to include newly identified systems, structures, and components that would have been subject to an aging management review or evaluation of time-limited aging analyses. Specifically, Main Feedwater mechanical control components are newly identified components that meet the scope of 10 CFR 54.4(a)(2). The summary information provided in Section 13.9, Appendix A for the newly identified components is consistent with the requirements of 10 CFR 54.21(d).

There are no new regulatory commitments contained in this letter. If you have any questions concerning this submittal, please contact Mr. Andrew McNeil, SQN Licensing Manager (acting), at (423) 843-8098.

I certify that I am duly authorized by TVA, and that, to the best of my knowledge and belief, the information contained herein accurately presents changes made since the previous submittal, necessary to reflect information and analyses submitted to the Commission or prepared pursuant to Commission requirements.

Respectfully,

Scott Hunnewell

Interim Site Vice President Sequoyah Nuclear Plant

Enclosures:

- 1. Redacted Updated Final Safety Analysis Report Amendment 29 (Public Use Only)
- 2. Updated Final Safety Analysis Report Amendment 29 (Non-Public Use)

cc (w/ Enclosures):

NRC Regional Administrator – Region II NRC Senior Resident Inspector – Sequoyah Nuclear Plant

NRC Project Manager - Sequoyah Nuclear Plant

AREVA Nuclear Power, Inc. Attn: Mr. Bob Clarke 3315 Old Forest Road OF-11 Lynchburg, Virginia 24501

ENCLOSURE 1

SEQUOYAH NUCLEAR PLANT REDACTED UPDATED FINAL SAFETY ANALYSIS REPORT AMENDMENT 29 (PUBLIC USE ONLY) (PROVIDED ON ENCLOSED COMPACT DISC)

ENCLOSURE 2

SEQUOYAH NUCLEAR PLANT
UPDATED FINAL SAFETY ANALYSIS REPORT AMENDMENT 29
(NON-PUBLIC USE)
(PROVIDED ON ENCLOSED COMPACT DISC)