



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
REGION II
245 PEACHTREE CENTER AVENUE N.E., SUITE 1200
ATLANTA, GEORGIA 30303-1200

November 13, 2020

Ms. Cheryl A. Gayheart
Regulatory Affairs Director
Southern Nuclear Operating Co., Inc.
3535 Colonnade Parkway
Birmingham, AL 35243

SUBJECT: VOGTLE ELECTRIC GENERATING PLANT, UNITS 1 & 2 – INTEGRATED
INSPECTION REPORT 05000424/2020003 AND 05000425/2020003

Dear Ms. Gayheart:

On September 30, 2020, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Vogtle Electric Generating Plant, Units 1 & 2. On October 22, 2020, the NRC inspectors discussed the results of this inspection with Mr. Drayton Pitts and other members of your staff. The results of this inspection are documented in the enclosed report.

No NRC-identified or self-revealing findings were identified during this inspection.

A licensee-identified violation which was determined to be of very low safety significance is documented in this report. We are treating this violation as a non-cited violation (NCV) consistent with Section 2.3.2 of the Enforcement Policy.

If you contest the violation or the significance or severity of the violation documented in this inspection report, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555-0001; with copies to the Regional Administrator, Region II; the Director, Office of Enforcement; and the NRC Resident Inspector at Vogtle Electric Generating Plant, Units 1 & 2.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

/RA/

Alan J. Blamey, Chief
Reactor Projects Branch 2
Division of Reactor Projects

Docket Nos. 05000424 and 05000425
License Nos. NPF-68 and NPF-81

Enclosure:
As stated

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SUBJECT: VOGTLE ELECTRIC GENERATING PLANT, UNITS 1 & 2 – INTEGRATED
INSPECTION REPORT 05000424/2020003 AND 05000425/2020003 dated
November 13, 2020

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U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report

Docket Numbers: 05000424 and 05000425

License Numbers: NPF-68 and NPF-81

Report Numbers: 05000424/2020003 and 05000425/2020003

Enterprise Identifier: I-2020-003-0050

Licensee: Southern Nuclear Operating Co., Inc.

Facility: Vogtle Electric Generating Plant, Units 1 & 2

Location: Waynesboro, GA

Inspection Dates: July 1, 2020 to September 30, 2020

Inspectors: A. Beasten, PhD, Acting Senior Resident Inspector
P. Cooper, Reactor Inspector
S. Downey, Senior Reactor Inspector
W. Pursley, Health Physicist
C. Safouri, Resident Inspector
S. Temple, Resident Inspector

Approved By: Alan J. Blamey, Chief
Reactor Projects Branch 2
Division of Reactor Projects

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting an integrated inspection at Vogtle Electric Generating Plant, Units 1 & 2, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information. A licensee-identified non-cited violation is documented in report section: 71124.01.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

Type	Issue Number	Title	Report Section	Status
LER	05000425/2019-003-00	LER 2019-003-00 for Vogtle Electric Generating Plant, Unit 2, Transformer Deenergization Results in Emergency Diesel Generator and Auxiliary Feedwater Systems Starting	71153	Closed

PLANT STATUS

Unit 1 operated at or near Rated Thermal Power (RTP) for the entire inspection period.

Unit 2 operated at or near RTP until the unit was shutdown for planned refueling outage cycle 21 (2R21) on August 16, 2020. The unit was returned to RTP on September 14, 2020 and remained at or near RTP for the remainder of the inspection period.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

Starting on March 20, 2020, in response to the National Emergency declared by the President of the United States on the public health risks of the coronavirus (COVID-19), resident and regional inspectors were directed to begin teleworking and to remotely access licensee information using available technology. During this time the resident inspectors performed periodic site visits each week, increasing the amount of time on site as local COVID-19 conditions permitted. As part of their onsite activities, resident inspectors conducted plant status activities as described in IMC 2515, Appendix D; observed risk significant activities; and completed on site portions of IPs. In addition, resident and regional baseline inspections were evaluated to determine if all or portion of the objectives and requirements stated in the IP could be performed remotely. If the inspections could be performed remotely, they were conducted per the applicable IP. In some cases, portions of an IP were completed remotely and on site. The inspections documented below met the objectives and requirements for completion of the IP."

REACTOR SAFETY

71111.04 - Equipment Alignment

Partial Walkdown Sample (IP Section 03.01) (5 Samples)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

- (1) Unit 2 Train 'B' Emergency Diesel Generator (EDG) and associated equipment during 2A EDG maintenance outage, on July 21, 2020.
- (2) Unit 2 Train 'B' Residual Heat Removal (RHR) system during Train 'A' RHR system outage, on July 29, 2020.
- (3) Unit 2 Train 'A' and 'B' RHR for Shutdown Cooling alignment during lowered inventory, on August 19, 2020.
- (4) Unit 1/2 Spent Fuel Pool Cooling while Unit 2 was defueled, on August 24, 2020.

- (5) Restoration of Unit 2 Train 'B' of Engineered Safety Features Actuation System (ESFAS) following ESFAS testing, on September 2, 2020.

Complete Walkdown Sample (IP Section 03.02) (1 Sample)

- (1) The inspectors evaluated system configurations during a complete walkdown of the Unit 2 Train 'B' EDG system on July 27, 2020.

71111.05 - Fire Protection

Fire Area Walkdown and Inspection Sample (IP Section 03.01) (6 Samples)

The inspectors evaluated the implementation of the fire protection program by conducting a walkdown and performing a review to verify program compliance, equipment functionality, material condition, and operational readiness of the following fire areas:

- (1) Fire Zones 36/37, Unit 1 Trains 'A' and 'B' Component Cooling Water Pump Rooms, on July 14, 2020.
- (2) Fire Zones 144/162/164, Unit 2 Train 'B' EDG and Cable Tunnel 2T4B, on July 22, 2020.
- (3) Fire Zones 140A/B/C/E, Unit 2 Containment Building, on August 20, 2020.
- (4) Fire Zones 57A/81A/125B/126B/153/178/180, Unit 2 Control Building and Engineered Safety Features (ESF) Chiller Rooms, on September 2, 2020.
- (5) Fire Zones 155/156/157A/157B, Unit 2 Auxiliary Feedwater (AFW) pump houses, on September 15, 2020.
- (6) Fire Zones 91/92/97/98/103, Unit 1 Control Building Level A and 4.16 kV switchgear and remote shutdown rooms, on September 16, 2020.

71111.07A - Heat Sink Performance

Annual Review (IP Section 03.01) (1 Sample)

The inspectors evaluated readiness and performance of:

- (1) Unit 2 Train 'B' EDG Jacket Water Heat Exchanger, on August 25, 2020.

71111.07T - Heat Sink Performance

Triennial Review (IP Section 03.02) (4 Samples)

- (1) Unit 2: Piping Penetration Cooler (21561E7002000), Cooled by Service Water - Section 02.02b
- (2) Unit 1: Component Cooling Water 'B' Heat Exchanger (11203E4002), Cooled by Service Water - Section 02.02b
- (3) Unit 2: Nuclear Service Cooling Water Pump Motor Cooler (21202P4006M01), Cooled by Service Water - Section 02.02b
- (4) Unit 1: Residual Heat Removal B Heat Exchanger (11205E6002), Closed Loop Intermediate System - Section 02.02c

71111.08P - Inservice Inspection Activities (PWR)

PWR Inservice Inspection Activities Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated pressurized water reactor non-destructive testing by reviewing the following examinations from September 14 to September 18, 2020:
 1. Liquid Penetrant Examination
 - a. Weld 683246-W2, Containment Spray Pump A valve replacement weld, ASME Class 2. This included a review of associated welding activities.
 - b. Weld 683246-W3, Containment Spray Pump A valve replacement weld, ASME Class 2. This included a review of associated welding activities.
 2. Magnetic Particle Examination
 - a. Weld 21201-B6-002-W26, 6" Auxiliary Feedwater nozzle to shell weld, ASME Class 2
 3. Ultrasonic Examination
 - a. Weld 21201-B6-002-W26, 6" Auxiliary Feedwater nozzle to shell weld, ASME Class 2.
 - b. Weld 21201-V6-002-W01, Pressurizer upper head to shell weld, ASME Class 1. This included the review of relevant indications that the licensee analytically evaluated and accepted for continued service.
 - c. Weld 21204-246-32-RB, 1.5" valve to pipe weld in Safety Injection Cold Leg Loop 4, ASME Class 1.
 - d. Weld 21204-245-34-RB, 3" reducer to branch connection weld in Safety Injection Cold Leg Loop 3, ASME Class 1
 4. Visual Examination
 - a. Bare Metal Visual of Reactor Vessel Head (per Code Case N-729-6)
 - b. VT-2 (System Leakage) of Reactor Coolant Pump 1 Seal Injection from Penetration 54 to Valve 006

The inspectors also evaluated the licensee's boric acid corrosion control program performance.

71111.11Q - Licensed Operator Regualification Program and Licensed Operator Performance

Licensed Operator Performance in the Actual Plant/Main Control Room (IP Section 03.01) (1 Sample)

- (1) The inspectors observed and evaluated licensed operator performance in the Control Room during the reactor shutdown from Mode 1 to Mode 4 to commence the Unit 2 Refueling Outage on August 16, 2020.

Licensed Operator Regualification Training/Examinations (IP Section 03.02) (1 Sample)

- (1) The inspectors observed and evaluated a licensed operator annual regualification exam scenario involving a faulted and ruptured Steam Generator and various transients, on September 22, 2020.

71111.12 - Maintenance Effectiveness

Maintenance Effectiveness (IP Section 03.01) (2 Samples)

The inspectors evaluated the effectiveness of maintenance to ensure the following structures, systems, and components (SSCs) remain capable of performing their intended function:

- (1) Effectiveness of lubrication program to ensure reliability and operability of safety-related equipment for Units 1 and 2, on September 23, 2020.
- (2) Unit 1 Seismic Gap and Hazard Barrier Seals for Cable Tunnel Rooms 1T4A and 1T4B, on September 30, 2020.

71111.13 - Maintenance Risk Assessments and Emergent Work Control

Risk Assessment and Management Sample (IP Section 03.01) (6 Samples)

The inspectors evaluated the accuracy and completeness of risk assessments for the following planned and emergent work activities to ensure configuration changes and appropriate work controls were addressed;

- (1) Unit 2 elevated risk due to Train 'B' Nuclear Service Cooling Water (NSCW) being inoperable during seismic restraint modifications, on July 15, 2020.
- (2) Unit 2 elevated risk due to Train 'A' EDG major outage with use of Risk Informed Completion Time, on July 20, 2020.
- (3) Unit 2 high risk activity for new fuel storage in the Spent Fuel Pool, on August 6, 2020.
- (4) Unit 2 elevated shutdown risk, Yellow, for core cooling and Reactor Coolant System (RCS) inventory due to lowered inventory in the RCS, and for Electrical DC Power due to 2D batteries being out of service during lowered inventory, on August 19, 2020.
- (5) Unit 2 high risk activity for fuel offloading during 2R21 Refueling Outage, on August 21, 2020.
- (6) Unit 1 elevated risk due to swapping from Train 'A' Stator Cooling Water (SCW) Pump to Train 'B' SCW pump for troubleshooting, on September 24, 2020.

71111.15 - Operability Determinations and Functionality Assessments

Operability Determination or Functionality Assessment (IP Section 03.01) (9 Samples)

The inspectors evaluated the licensee's justifications and actions associated with the following operability determinations and functionality assessments:

- (1) Unit 2 Train 'B' RHR Operable but Degraded Non-conforming due to oil loss, on July 28, 2020.
- (2) Unit 2 Main Feedwater Regulating Valve solenoid valves exceeding Environmental Qualification qualified life, on August 4, 2020.
- (3) Unit 1 Train 'A' RHR Pump Motor Cooler Pressure Relief Valve, 1PSV11748, exceeding testing requirements, on August 11, 2020.
- (4) Unit 1 Train 'B' NSCW pump motor cooling outlet spool flange surface outside of procedure limits, on September 14, 2020.
- (5) Unit 1 Train 'A' ESF Chiller compressor oil temperature found high, on September 15, 2020.

- (6) Part 21 Evaluation for Framatome Letter LTR19061 (D26 Relays Only), on September 16, 2020.
- (7) Unit 2 Electrical Containment Penetration #64 leaking by, on September 17, 2020.
- (8) Unit 1 Plant Ventilation Radiation Monitor, 1-RE-12442, functionality challenges while 1-RE-12444 was out of service, on September 17, 2020.
- (9) Unit 1 Train 'A' EDG broken fuel rail hold-down bolt, on September 21, 2020.

71111.18 - Plant Modifications

Temporary Modifications and/or Permanent Modifications (IP Section 03.01 and/or 03.02) (2 Samples)

The inspectors evaluated the following temporary or permanent modifications:

- (1) Unit 2 Solid State Protection System (SSPS) Bridging Strategy, on August 27, 2020.
- (2) Unit 2 Single Point Vulnerabilities (SPVs) reduction from Main Steam Isolation Valve (MSIV) control circuits, on September 17, 2020.

71111.19 - Post-Maintenance Testing

Post-Maintenance Test Sample (IP Section 03.01) (8 Samples)

The inspectors evaluated the following post maintenance test activities to verify system operability and functionality:

- (1) 14980A-2, Unit 2 Train 'A' EDG operability run, following extended maintenance outage, on July 27, 2020.
- (2) Functional Test following Unit 2 Train 'A' RHR system outage, on July 29, 2020.
- (3) Unit 2 Train 'A' and 'B' SSPS General Warning Monitor Circuit and Power Supplies functional testing after Modifications, on August 26, 2020.
- (4) Leak checks following Unit 2 Train 'B' EDG Jacket Water Heat Exchanger inspection, on August 28, 2020.
- (5) Work Order (WO) SNC1115784, for replacement activities on Unit 1 Loop 4 Delta T instrument card, on September 2, 2020.
- (6) Unit 2 Main Generator Protective Relay Replacement Functional Test, on September 24, 2020.
- (7) Operation of Unit 1 Train 'B' NSCW Pump 2 following planned maintenance, on September 29, 2020.
- (8) Functional Test for Train 'A' MSIV following reduction of SPVs from control circuits, on September 30, 2020.

71111.20 - Refueling and Other Outage Activities

Refueling/Other Outage Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated Unit 2 refueling outage (2R21) activities from August 16, 2020 to September 11, 2020.

71111.22 - Surveillance Testing

The inspectors evaluated the following surveillance tests:

Surveillance Tests (other) (IP Section 03.01) (4 Samples)

- (1) 28914-C, Unit 1 Group 'A' Pressurizer Heater Capacity Test, on July 31, 2020.
- (2) 14721C-2, Unit 2 Emergency Core Cooling System Subsystem Flow Balance and Check valve Refueling Inservice Test, on August 19, 2020.
- (3) 14667-2, Unit 2 Train 'B' Diesel Generator and ESFAS Test, on September 1, 2020.
- (4) 14244-2, Unit 2 Main Turbine Safety Injection / P-14 or P-4 Final Trip Device Actuation Test, on September 11, 2020.

Inservice Testing (IP Section 03.01) (1 Sample)

- (1) Unit 2 Train 'B' Motor Driven AFW Pump Inservice Test and Response Time Test, on July 21, 2020.

Containment Isolation Valve Testing (IP Section 03.01) (1 Sample)

- (1) Containment Penetration No. 80 Containment Service Air Supply Line Local Leak Rate Test, on August 12, 2020.

71114.06 - Drill Evaluation

Drill/Training Evolution Observation (IP Section 03.02) (1 Sample)

The inspectors evaluated:

- (1) Licensed Operator Requalification Training exam scenario involving a Reactor Coolant Pump seal failure and various system malfunctions, on September 21, 2020.

RADIATION SAFETY

71124.01 - Radiological Hazard Assessment and Exposure Controls

Radiological Hazard Assessment (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated how the licensee identifies the magnitude and extent of radiation levels and the concentrations and quantities of radioactive materials and how the licensee assesses radiological hazards.

Instructions to Workers (IP Section 03.02) (1 Sample)

- (1) The inspectors evaluated radiological protection-related instructions to plant workers.

Contamination and Radioactive Material Control (IP Section 03.03) (3 Samples)

The inspectors evaluated licensee processes for monitoring and controlling contamination and radioactive material.

- (1) Observed workers exiting the RCA at Unit 2 during a refueling outage.
- (2) Observed licensee surveys of potentially contaminated material leaving the RCA during the Unit 2 refueling outage.

- (3) Evaluated reasons for differences in personnel contamination monitoring equipment count times between plant RCA exit and the Unit 2 personnel hatch.

Radiological Hazards Control and Work Coverage (IP Section 03.04) (5 Samples)

The inspectors evaluated in-plant radiological conditions during facility walkdowns and observation of radiological work activities.

- (1) Unit 2 Reactor Coolant Pump Motor Removal under RWP #20-2200. involved work in area with potential for airborne radioactivity.
- (2) Unit 2 Reactor Coolant Pump Seal Removals under RWP #20-2203 involving a breach of the RCS system and potential for high contamination levels and potential alpha level II controls.
- (3) Unit 2 Reactor Coolant Pump Impeller Removal under RWP #20-2203 involving high potential airborne risks, expected dose rates up to 7 Rem per hour and large dose rate gradients.
- (4) U2 RP entry into Unit 2 Regenerative Heat Exchanger Room for Pre-Job Survey under RWP #20-2108 involving RP surveys in Locked High Radiation Areas with unknown dose rates prior to entry.
- (5) Decon of Unit 2 Reactor Coolant Pump Housing Flange under RWP #20-2109 involving highly contaminated surfaces.

High Radiation Area and Very High Radiation Area Controls (IP Section 03.05) (3 Samples)

The inspectors evaluated licensee controls of the following High Radiation Areas and Very High Radiation Areas:

- (1) High Radiation Area controls during the Unit 2 Reactor Coolant Pump Impeller Removal
- (2) Very High Radiation Area controls for the Unit 2 Spent Fuel Pool Transfer Canal
- (3) Unit 2 Chemical Volume Control System Letdown Heat Exchanger Room.

Radiation Worker Performance and Radiation Protection Technician Proficiency (IP Section 03.06) (1 Sample)

- (1) The inspectors evaluated radiation worker and radiation protection technician performance as it pertains to radiation protection requirements.

OTHER ACTIVITIES – BASELINE

71151 - Performance Indicator Verification

The inspectors verified licensee performance indicators submittals listed below:

MS07: High Pressure Injection Systems (IP Section 02.06) (2 Samples)

- (1) Unit 1 (July 1, 2019 – June 30, 2020)
- (2) Unit 2 (July 1, 2019 – June 30, 2020)

MS08: Heat Removal Systems (IP Section 02.07) (2 Samples)

- (1) Unit 1 (July 1, 2019 – June 30, 2020)
- (2) Unit 2 (July 1, 2019 – June 30, 2020)

MS09: Residual Heat Removal Systems (IP Section 02.08) (2 Samples)

- (1) Unit 1 (July 1, 2019 – June 30, 2020)
- (2) Unit 2 (July 1, 2019 – June 30, 2020)

OR01: Occupational Exposure Control Effectiveness Sample (IP Section 02.15) (1 Sample)

- (1) 09/30/2019 to 06/01/2020

71153 - Followup of Events and Notices of Enforcement Discretion

Event Report (IP Section 03.02) (1 Sample)

The inspectors evaluated the following licensee event reports (LERs):

- (1) LER 05000425/2019-003-00, Transformer deenergization results in Emergency Diesel Generator and Auxiliary Feedwater Systems Starting (ADAMS Accession No. ML19338C617). The inspectors determined that it was not reasonable to foresee or correct the cause discussed in the LER therefore no performance deficiency was identified. The inspectors did not identify a violation of NRC requirements.

INSPECTION RESULTS

Licensee-Identified Non-Cited Violation	71124.01
This violation of very low safety significance was identified by the licensee and has been entered into the licensee corrective action program and is being treated as a non-cited violation, consistent with Section 2.3.2 of the Enforcement Policy.	
Violation: Licensee Identified Violation of 10CFR 20.1501(a)(1) which requires in part, surveys necessary to evaluate the magnitude and extent of radiation levels. In addition, 10CFR20.1902 requires posting of areas as a High Radiation Area (HRA) where dose rates exceed 100 mrem in one hour at 30 centimeters. Contrary to these requirements, on September 1, 2020, following a primary resin sluice on Unit 2, a down-posting survey was performed that failed to fully evaluate the magnitude and extent of radiation levels and failed to identify that HRA conditions still existed. Specifically, room 2-AB-86-Mezzanine which contained a portion of the resin line was not surveyed. On September 18, during a routine RP survey, dose rates as high as 8000 mrem per hour contact and 400 mrem per hour at 30 centimeters were discovered on this portion of the resin line, however the room was no longer posted in accordance with 10CFR1902.	
Significance/Severity: Green.	
Corrective Action References: CR10739998	

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On October 22, 2020, the inspectors presented the integrated inspection results to Mr. Drayton Pitts and other members of the licensee staff.
- On July 17, 2020, the inspectors presented the Triennial Heat Sink Inspection results to Drayton Pitts, Site Vice President and other members of the licensee staff.
- On September 1, 2020, the inspectors presented the RP Routine Baseline Inspection 2020003 Exit inspection results to Drayton Pitts, Site Vice President and other members of the licensee staff.
- On September 18, 2020, the inspectors presented the Baseline ISI Exit Meeting inspection results to Drayton Pitts, Site Vice President and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.04	Drawings	1X4DB130	P&I Diagram - Spent Fuel Cooling & Purification System - System No. 1213	50.0
71111.04	Drawings	2X4DB121	P&I Diagram - Safety Injection System - System No. 1204	50.0
71111.04	Drawings	2X4DB122	P&I Diagram - Residual Heat Removal - System No. 1205	54.0
71111.04	Drawings	2X4DB130	P&I Diagram - Spent Fuel Cooling & Purification System - System No. 1213	42.0
71111.04	Drawings	2X4DB170-2	P&I Diagram Diesel Generator System Train B System No. 2403	45.0
71111.04	Miscellaneous	10032-C	Shutdown Defense-In-Depth Assessment	08/24/2020
71111.04	Procedures	11145-2	Diesel Generator Alignment	14.1
71111.04	Procedures	NMP-OS-010	Protected Train/Division and Protected Equipment Program	8.1
71111.04	Procedures	NMP-OS-010-003	Vogtle Protected Equipment Logs	11.0
71111.05	Corrective Action Documents	10574691, 10732376		
71111.05	Fire Plans	92736-1	Zone 36 - Auxiliary Building Level A Fire Fighting Preplan	5.0
71111.05	Fire Plans	92737-1	Zone 37 - Auxiliary Building Level A Fire Fighting Preplan	5.0
71111.05	Fire Plans	92757A-1	Zone 57A - Control Building - Levels B, A, 1, 2, and 3 Fire Fighting Preplan	4.2
71111.05	Fire Plans	92781A-1	Zone 81A - Control Building - Levels B, A, 1, 2, and 3 Fire Fighting Preplan	5.0
71111.05	Fire Plans	92791-1	Zone 91 - Control Building Level A Fire Fighting Preplan	4.1
71111.05	Fire Plans	92792-1	Zone 92 - Control Building Level A Fire Fighting Preplan	3.1
71111.05	Fire Plans	92797-1	Zone 97 - Control Building - Level A Fire Fighting Preplan	3
71111.05	Fire Plans	92798-1	Zone 98 - Control Building - Level A Fire Fighting	4.0

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			Preplan	
71111.05	Fire Plans	92803-1	Zone 103 - Control Building - Level A Fire Fighting PrePlan	3.0
71111.05	Fire Plans	92825B-1	Zone 125 B Control Building Level 3 Fire Fighting Preplan	5.3
71111.05	Fire Plans	92826B-1	Zone 126B - Control Building - Level 3 Fire Fighting Preplan	3.2
71111.05	Fire Plans	92840A-2	Zone 140A - Containment Building - Levels A, B, 1, 2 and 3 Fire Fighting Preplan	3.1
71111.05	Fire Plans	92840B-2	Zone 140B - Containment Building - Levels A, B, 1, 2 and 3 Fire Fighting Preplan	4.1
71111.05	Fire Plans	92840C-2	Zone 140C - Containment Building - Levels A, B, 1, 2 and 3 Fire Fighting Preplan	2.1
71111.05	Fire Plans	92840E-2	Zone 140E - Containment Building - Levels A, B, 1, 2 and 3 Fire Fighting Preplan	1.1
71111.05	Fire Plans	92844-2	Zone 144 - Diesel Generator Building - Electrical Tunnel - Train B - Fire Fighting Preplan	2.2
71111.05	Fire Plans	92853-2	Zone 153 - Control Building - Level B Fire Fighting Preplan	0.2
71111.05	Fire Plans	92855-2	Zone 155 - Auxiliary Feedwater Pumphouse - Train B Fire Fighting Preplan	0.2
71111.05	Fire Plans	92856-2	Zone 156 - Auxiliary Feedwater Pumphouse Fire Fighting Preplan	0.2
71111.05	Fire Plans	92857A-2	Zone 157A - Auxiliary Feedwater Pumphouse - Train C Fire Fighting Preplan	0.2
71111.05	Fire Plans	92857B-2	Zone 157B - Auxiliary Feedwater Pumphouse - Train C Fire Fighting Preplan	0.2
71111.05	Fire Plans	92862-2	Zone 162 - Diesel Generator Building Fire Fighting Preplan	1.1
71111.05	Fire Plans	92864-2	Zone 164 - Diesel Generator Building - Train B DFO Day Tank Fire Fighting Plan	0.2
71111.05	Fire Plans	92878-2	Zone 178 Control Building Level 3 Fire Fighting Preplan	1.1
71111.05	Fire Plans	92880-2	Zone 180 Control Building Level 3 Fire Fighting	0.2

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			Preplan	
71111.05	Fire Plans	NMP-ES-035-007	Fleet Fire Watch Instruction	5.0
71111.05	Procedures	92040-C	Fire Protection Functionality and LCO Requirements	46.3
71111.05	Work Orders	SNC1114249, SNC782278, SNC902691, SNC993922		
71111.07A	Procedures	83395-C	Heat Exchanger Testing/Maintenance Program	7.10
71111.07A	Procedures	NMP-ES-012	Heat Exchanger Program	12.0
71111.07A	Work Orders	SNC1013636, SNC820593		
71111.08P	Corrective Action Documents	Condition Reports (by CR #)	CR10613570, CR10631016, CR10636903, CR10729805, CR10729936, CR10730672, CR10730754, CR10733292	
71111.08P	Engineering Evaluations	Corrosion Assessment Number 1213-2011-009	Boric Acid Corrosion Assessment for Train A Spent Fuel Pool Heat Exchanger	08/14/2020
71111.08P	Engineering Evaluations	LTR-SDA-19-018-P	Vogtle Unit 2 Pressurizer top Head to Upper Shell Weld ASME Section XI Evaluation of As-Found Indications	Revision 0
71111.08P	Engineering Evaluations	SNC1115524	Evaluation of V2R21 Reactor Head Inspection Results	09/03/2020
71111.08P	Miscellaneous	SG-CDMP-20-18	Steam Generator Degradation Review for Vogtle Unit 2 Cycle 21	Revision 0
71111.08P	NDE Reports	S20V2V045	Visual Examination Report for Vessel Closure Head Exterior	08/30/2020
71111.08P	Procedures	NMP-ES-019-001	Boric Acid Corrosion Control Program Implementation	Version 11.1
71111.08P	Procedures	NMP-ES-019-004	Boric Acid Corrosion Control Program - Corrosion Assessment	Version 5.1
71111.08P	Procedures	NMP-ES-024-202	Visual Examination (VT-2)	Version 7.0
71111.08P	Procedures	NMP-ES-024-208	Visual Examination of Reactor Vessel Head Penetrations and Base Material	Version 6.3
71111.08P	Procedures	NMP-ES-024-401	Magnetic Particle Examination	Version 12.0
71111.08P	Procedures	NMP-ES-024-501	PDI Generic Procedure for Ultrasonic Examination of Austenitic Pipe Welds (Appendix VIII)	Version 7.0

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.08P	Procedures	NMP-ES-024-516	Manual Ultrasonic Examination of Pressure Vessel Welds (Non-Appendix VIII)	Version 6.0
71111.11Q	Procedures	12004-2	Power Operation (Mode 1)	4.1
71111.11Q	Procedures	12005-C	Reactor Shutdown to Hot Standby (Mode 2 to Mode 3)	29
71111.11Q	Procedures	12006RFO-2	Unit Cooldown to Cold Shutdown for Refueling	1.7
71111.12	Corrective Action Documents	123064, 162028, 162032, 395948, 503291, 802581, 841494, 10313228, 10313317, 10315786, 10315787, 10407211, 10407213, 10645149, 10658620, 10659181, 10686716		
71111.12	Corrective Action Documents Resulting from Inspection	10658261		
71111.12	Drawings	AX2D94V030	Waterstops & Waterproofing General Arrangement Plan	12
71111.12	Drawings	AX2D94V031	Waterstops & Waterproofing Sections and Details Sheet 1	13
71111.12	Drawings	AX2D94V033	Waterstops & Waterproofing Sections and Details Sheet 3	12
71111.12	Procedures	20411-C	Control of Lubricants	16.3, 17, 18.1
71111.12	Procedures	NMP-ES-021	Structural Monitoring Program for the Maintenance Rule	11.0
71111.12	Procedures	NMP-ES-027	Maintenance Rule Program	10.2
71111.12	Procedures	NMP-ES-074-006	Fleet Lubrication Instruction	4.0
71111.12	Work Orders	SNC1033792, SNC1057746		
71111.13	Corrective Action Documents	10725636		

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.13	Corrective Action Documents Resulting from Inspection	10722710		
71111.13	Miscellaneous		Phoenix Risk Profile	07/15/2020
71111.13	Miscellaneous		Protecting Equipment Logs	07/15/2020
71111.13	Miscellaneous		Phoenix Risk Model	07/21/2020
71111.13	Miscellaneous		Plan of the Day	07/20/2020
71111.13	Miscellaneous		V2R21 Defense in Depth Assessment Final Report	1
71111.13	Miscellaneous		Phoenix Risk Model	07/20/2020
71111.13	Miscellaneous		Plan of the Day	08/06/2020
71111.13	Miscellaneous	10032-C	Shutdown Defense-In-Depth Assessment (Mode 4, 5, 6, 0)	08/21/2020
71111.13	Miscellaneous	VNP-1-20-003	Degraded SCW Pumps ODMI	
71111.13	Procedures	10032-C	Outage Risk Assessment Monitoring	16
71111.13	Procedures	13419-C	Diesel Generator RICT Support	10.3
71111.13	Procedures	NMP-AD-002	Conduct of Problem Solving and Troubleshooting	13.5
71111.13	Procedures	NMP-AD-002-F04	Troubleshooting Log	1.1
71111.13	Procedures	NMP-DP-001-F01	Operational Risk Awareness Form 1 Risk Activity Evaluation Worksheet	1.0
71111.13	Procedures	NMP-GM-031-003	Risk Management Actions for 10 CFR 50.65(a)(4) and the Risk Informed Completion Time Program	8.0
71111.13	Procedures	NMP-MA-009	Foreign Material Exclusion Program	15.0
71111.13	Procedures	NMP-MA-009-001	Foreign Material Exclusion Program Requirements	10.0
71111.13	Procedures	NMP-OM-002	Shutdown Risk Management	6.0
71111.13	Procedures	NMP-OS-003	Operational Decision Making Issue Evaluation Process	10.0
71111.13	Procedures	NMP-OS-010	Protected Train/Division and Protected Equipment Program	8.1
71111.13	Procedures	NMP-RE-011	Special Nuclear Material Movement	1.0
71111.13	Work Orders	SNC963072		
71111.15	Calculations	X4CPS.0075.183	Qualified Life (QL) Extension for ASCO Solenoid Valves	21

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.15	Corrective Action Documents	10714605, 10727375, 10739502, 277403, 10691621, 10693484, 10727332, 10727518, 277066, 277582, 10265306, 10646853, 10728167, 10708633, 10704418, 10731746, 10720064, 10732422		
71111.15	Engineering Evaluations		2B RHR leak rate calculation	
71111.15	Engineering Evaluations		2B RHR Motor - Oil Level Trend	4
71111.15	Miscellaneous	1-2020-3	Standing Order: Actions Required if overpressure condition (pipe rupture) occurs on NSCW A to RHR A pump motor cooler due to a failure of the relief valve 1PSV11748 to lift	1.0
71111.15	Miscellaneous	AX4AJ04-20000	Installation, Operation, & Maintenance Manual ESF Chillers	9.0
71111.15	Miscellaneous	MIS-17-009	Vogtle Electric Generating Plant Fourth 10-Year Interval Inservice Testing Program	11.0
71111.15	Miscellaneous	MIS-17-027	Vogtle Electric Generating Plant Unit 1 Fourth 10-Year Interval Valve Inservice Testing Basis Document	2.0
71111.15	Miscellaneous	SO 2-2020-4	Standing Order - 2B RHR Oil Leak Monitoring Plan	08/01/2020
71111.15	Procedures	11881-2	Unit 2 Auxiliary Building Rounds	64.0
71111.15	Procedures	24965-C	Containment Electrical Penetration Local Leak Rate Test and Charging Instructions	19
71111.15	Procedures	34316-C	Operation of DRMS Plant Vent Effluent Monitor (RE-12442)	40
71111.15	Procedures	GEN-87	Operator Lubricant Addition List	6.0
71111.15	Procedures	NMP-AD-012	Operability Determinations and Functionality Assessments	13.3
71111.15	Procedures	NMP-AD-012-GL04	Mission Times	1.1

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.15	Procedures	NMP-EP-305-GL03	VEGP Equipment Important to the EP Function	6.2
71111.15	Procedures	NMP-ES-013	Inservice Testing Program	8.1
71111.15	Procedures	NMP-ES-013-006	IST Ten Year Interval Update	2.1
71111.15	Procedures	NMP-ES-013-010	IST Relief Valve Program Development and Maintenance	1.0
71111.15	Procedures	NMP-ES-016	Environmental Qualification Program	11.1
71111.15	Work Orders	SNC1096401, SNC1096402, SNC1096403, SNC1096404		
71111.18	Engineering Changes	SNC106213450.59	50.59 Evaluation of Eliminate SPVs from MSIV control circuits for Normally Energized MDR relays and micro switches	
71111.18	Engineering Changes	SNC1062134COMPACTS	Risk Mitigation plan for elimination of SPVs from MSIV control circuits for Normally Energized MDR relays and micro switches	
71111.18	Engineering Changes	SNC1062134DCP	Eliminate SPVs from MSIV control circuits for Normally Energized MDR relays and micro switches	
71111.18	Engineering Changes	SNC1062134WKCOMP	Implementing Work Order for elimination of SPVs from MSIV control circuits for Normally Energized MDR relays and micro switches	
71111.18	Engineering Changes	SNC95902550.59	10 CFR 50.59 Screening: U2 SSPS Bridging Strategy: Low Margin, Increase UVTA Coil Ckt Voltage	0.0
71111.18	Engineering Changes	SNC959025AD	Applicability Determination: U2 SSPS Bridging Strategy: Low Margin, Increase UVTA Ckt Voltage	1.0
71111.18	Engineering Changes	SNC959025DCP	U2 SSPS Bridging Strategy	0
71111.18	Engineering Changes	SNC959025FFP6	Detailed Fire Protection Program Change Evaluation: U2 SSPS Bridging Strategy	0.0
71111.18	Engineering Changes	SNC959025FPP5	Fire Protection Program Impact Screening: U2 SSPS Bridging Strategy	0.0
71111.18	Engineering Evaluations	SNC1062134RISK	Risk Assessment Form for elimination of SPVs from MSIV control circuits for Normally Energized MDR	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			relays and micro switches	
71111.18	Miscellaneous	EQ3663	Seismic Qualification Data Summary / Seismic and Dynamic Qualification Summary	1.0
71111.18	Miscellaneous	EQ3666	Seismic Qualification Data Summary / Seismic and Dynamic Qualification Summary	1.0
71111.18	Miscellaneous	PWROG-14063-P	SSPS Conceptual General Warning Monitor Circuit Summary Report	0
71111.18	Miscellaneous	SNC959025RISK	Design Change/Modification - Risk Assessment Form: SNC959025	0.0
71111.19	Corrective Action Documents	10726268, 10730390, 10735813, 10736030, 10736032, 10735143		
71111.19	Miscellaneous	DCP SNC959025	Unit 2 SSPS General Warning Monitor Circuit (GWMC) Site Acceptance Test	1.0
71111.19	Procedures	11885A-2	Diesel Generator 2A Operating Log	5.3
71111.19	Procedures	13150B-1	Train B Nuclear Service Cooling Water System	13
71111.19	Procedures	14420-2	Solid State Protection System Train A Operability Test	52.1
71111.19	Procedures	14980A-2	Diesel Generator 2A Operability Test	31.1
71111.19	Procedures	22408-C	7300 Circuit Board Removal, Inspection, Cleaning, and Reinstallation	44.2
71111.19	Procedures	24813-1	Delta T/T Avg Loop 4 Protection Channel IV 1T-441 Channel Operational Test and Channel Calibration	48
71111.19	Work Orders	SNC1056311, SNC1056316, SNC1060825, SNC872956DCP, SNC1070289, SNC1085621, SNC1091321, SNC1091322, SNC1115784, SNC928745, SNC931262		
71111.22	Corrective Action	10637850, 10722762,		

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
	Documents	10729784, 10731967		
71111.22	Drawings	1X3D-AA-E01A	One Line Diagram - 480V Switchgear 1NB01 - 1-1805-S3-B01	15.0
71111.22	Drawings	1X3D-AA-F13A	One Line Diagram - 480V Pressurizer Heater Panels - 1NBPC, 1NBPB1, 1NBPB2, 1NBPB3	7.0
71111.22	Miscellaneous	NUREG-0737	Clarification of TMI Action Plan Requirements	11/1990
71111.22	Procedures	13430-1	480V AC Non 1E Electrical Distribution System	16.2
71111.22	Procedures	14244-2	Main Turbine SI/P-14 or P-4 Final Trip Device Actuation Test	3
71111.22	Procedures	14380-2	Containment Penetration No. 80 Containment Service Air Supply Line Local Leak Rate Test	11
71111.22	Procedures	14545B-2	Motor Driven Auxiliary Feedwater Pump B Operability Test	1.2
71111.22	Procedures	14721C-2	ECCS Subsystem Flow Balance and Checkvalve Refueling Inservice Test	9
71111.22	Procedures	14807B-2	Train B Motor Driven Auxiliary Feedwater Pump / Check Valve Inservice and Response Time Test	5.1
71111.22	Procedures	18031-1	Loss of Class 1E Elect Systems	1.2
71111.22	Procedures	19011-1	ES - 1.1 SI Termination	1.1
71111.22	Procedures	28914-C	Pressurizer Heater Capacity Test	17
71124.01	ALARA Plans	RWP 20-2203	20-2203 Remove and Replace RCP#1 Pump	08/03/2020
71124.01	Corrective Action Documents	10733187	Key to LHRA lock 02 was lost in containment	08/25/2020
71124.01	Corrective Action Documents Resulting from Inspection	10734162	Relocation of dosimetry due to potential gradient dose rates not evaluated in accordance with RP procedural requirements.	08/31/2020
71124.01	Corrective Action Documents Resulting from Inspection	10734865	ALARA plan for the RCP pump replacement did not discuss contingencies for the pump binding as it did in 1998	08/31/20
71124.01	Corrective Action Documents Resulting from	10734866	Permanent High Radiation Area postings in U2 containment have not been Updated	08/31/20

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
	Inspection			
71124.01	Corrective Action Documents Resulting from Inspection	10734869	RP supervisory oversight required for high risk work may be more effective in central monitoring than at the job when telemetry and video capabilities are available.	08/31/20
71124.01	Corrective Action Documents Resulting from Inspection	10734871	Organizational knowledge gap in the appropriate criteria for relocation of dosimetry since the NISPs were issued.	08/31/20
71124.01	Procedures	NM-HP-306	Radiological Job Coverage	Version 1.0
71124.01	Procedures	NMP-HP-108	Issuance, Use, and Collection of Personnel Dosimetry	Version 2.7
71124.01	Procedures	NMP-HP-108-002	Use of EDE (Effective Dose Equivalent) Methodologies	Version 3.1
71124.01	Procedures	NMP-HP-204	ALARA Planning and Job Review	Version 5.5
71124.01	Procedures	NMP-HP-300	Radiation and Contamination Surveys	Version 5.4
71124.01	Radiation Surveys	228197	HI_STORM SURFACE DOSE RATES (C) MPC #489	08/17/2019
71124.01	Radiation Surveys	228198	HI_STORM SURFACE DOSE RATES (C) MPC #489	08/17/2019
71124.01	Radiation Surveys	228388	HI_STORM SURFACE DOSE RATES (C) MPC #490	08/26/2019
71124.01	Radiation Surveys	228389	HI_STORM SURFACE DOSE RATES (C) MPC #490	08/26/2019
71124.01	Radiation Surveys	228946	HI_STORM SURFACE DOSE RATES (C) MPC #544	09/14/2019
71124.01	Radiation Surveys	228947	HI_STORM SURFACE DOSE RATES (C) MPC #539	09/21/2019
71124.01	Radiation Surveys	237531	Plant Vogtle Quadrant 4 Elevation 197	08/16/2020
71124.01	Radiation Surveys	237538	Plant Vogtle Level A Lower Overview (2RXAL)	08/16/2020

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71124.01	Radiation Surveys	237856	Plant Vogtle Level A Lower Overview	08/16/2020
71124.01	Radiation Surveys	238058	Plant Vogtle RCP Motor (2RXA15)	08/23/2020
71124.01	Radiation Surveys	238069	RCP #1 Shroud (rev#2) (2RXB5)	08/23/2020
71124.01	Radiation Surveys	238070	RCP #1 Seal Plate (2RXB9)	08/23/2020
71124.01	Radiation Surveys	238113	Plant Vogtle Regenerative Heat Exchanger Room	08/24/2020
71124.01	Radiation Surveys	238263	Reactor Coolant Pump Impeller (1RXA-19)	08/27/2020
71124.01	Radiation Surveys	2H19	Vogtle Boundary Report from Environmental Laboratory	07/12/2019 - 12/31/2019
71124.01	Radiation Work Permits (RWPs)	20-0148	Unit 1 Remove and Replace Pump on RCP#1 and All Associated Work in Unit 2 Containment	Revision 0
71124.01	Radiation Work Permits (RWPs)	20-2003	2R21 Refueling Outage Work Activities to Include CMs/PMs, Surveillance, Calibration and Misc/Support Work Activities in High Radiation Areas not Covered Under Other RWPs.	Revision 0
71124.01	Radiation Work Permits (RWPs)	20-2109	Decon Activities in Locked High Radiation Areas in U2 CNMT	Revision 0
71124.01	Radiation Work Permits (RWPs)	20-2202	Remove and Replace RCP #1 Pump to Include All Associated Work for Assembly and Reassembly in Unit 2 Containment	Revision 0
71124.01	Radiation Work Permits (RWPs)	20-2202	Remove and Replace RCP #1 Pump to Include All Associated Work for Assembly and Reassembly in Unit 2 Containment	
71124.01	Radiation Work Permits (RWPs)	20-2203	RP Surveys and Job Coverage In Locked High Radiation Areas in U2 Containment	Revision 0
71124.01	Radiation Work Permits (RWPs)	20-2505	Under Rx Vessel Work Including DMIMS, Cavity Sump CM/PM, Liner Plate Inspection,/Repair and All Associated Work	Revision 0
71124.01	Self-	NMP-HP-305	Vogtle Alpha Plant Characterization Study	04/24/2020

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
	Assessments			
71151	Corrective Action Documents	10730772	Individual Received Unanticipated Dose Rate Alarm	08/16/2020
71151	Corrective Action Documents Resulting from Inspection	10741335, 278373		
71151	Miscellaneous	Access Control Alarms Report	Access Control Alarms Report	01/16/2019 to 08/21/2020
71151	Procedures	MSPI-VOGTLE	NRC Mitigating System Performance Index (MSPI) Basis Document	10
71151	Procedures	MSPI-VOGTLE	NRC Mitigating System Performance Index (MSPI) Basis Document	9
71151	Procedures	NMP-ES-076	Mitigating System Performance Index Program (MSPI)	3.1
71151	Procedures	NMP-ES-076-001	Mitigating System Performance Index Implementation Instruction	4.1
71153	Corrective Action Documents	10724196, 10650853, 10653959, 10653949, 276893, 256995, 1057750, 1054522		
71153	Drawings	AX3D-AA-A03A	Vogtle - Wilson - Master One Line Diagram	20.0
71153	Miscellaneous		Plant Wilson Services Agreement Between Georgia Power Company and Southern Nuclear Operating Company, Inc.	
71153	Miscellaneous	NUMARC 93-01	Nuclear Energy Institute - Industry Guideline for Monitoring the Effectiveness of Maintenance at Nuclear Power Plants	4A
71153	Miscellaneous	RG 1.160	Monitoring the Effectiveness of Maintenance at Nuclear Power Plant	3
71153	Procedures	13418A-2	Standby Auxiliary Transformer Unit Two Train A Operations	6.2
71153	Procedures	13418A-2	Standby Auxiliary Transformer Unit Two Train A	7.1

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			Operations	
71153	Procedures	14230-2	Offsite AC Circuit Verification and Capacity/Capability Evaluation	28
71153	Procedures	NMP-GM-021-003	Plant Vogtle 1-2 Switchyard Access and Maintenance Controls	2.0
71153	Procedures	NMP-GM-021-003	Plant Vogtle 1-2 Switchyard Access and Maintenance Controls	1.1
71153	Procedures	NMP-OS-010-003	Vogtle Protected Equipment Logs	11.0
71153	Procedures	NMP-OS-010-003	Vogtle Protected Equipment Logs	10.0