



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
REGION I
2100 RENAISSANCE BOULEVARD, SUITE 100
KING OF PRUSSIA, PENNSYLVANIA 19406-2713

November 10, 2020

Mr. Bryan C. Hanson
Senior Vice President
Exelon Generation Company, LLC
President and Chief Nuclear Officer
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT – INTEGRATED
INSPECTION REPORT 05000333/2020003

Dear Mr. Hanson:

On September 30, 2020, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at James A. FitzPatrick Nuclear Power Plant. On October 22, 2020, the NRC inspectors discussed the results of this inspection with Mr. Pat Navin, Site Vice President, and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

X /RA/

Signed by: Erin E. Carfang
Erin E. Carfang, Chief
Reactor Projects Branch 1
Division of Reactor Projects

Docket No. 05000333
License No. DPR-59

Enclosure:
As stated

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DATE	11/09/2020	11/10/2020	11/10/2020		

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**U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report**

Docket Number: 05000333

License Number: DPR-59

Report Number: 05000333/2020003

Enterprise Identifier: I-2020-003-0019

Licensee: Exelon Generation Company, LLC

Facility: James A. FitzPatrick Nuclear Power Plant

Location: Oswego, NY

Inspection Dates: July 1, 2020 to September 30, 2020

Inspectors: E. Miller, Senior Resident Inspector
J. England, Resident Inspector
H. Anagnostopoulos, Senior Health Physicist
E. Andrews, Health Physicist
M. Henrion, Health Physicist
C. Lally, Senior Project Engineer
M. Patel, Senior Reactor Inspector
R. Rolph, Resident Inspector
J. Rady, Emergency Preparedness Inspector
B. Sienel, Resident Inspector
A. Turilin, Reactor Inspector

Approved By: Erin E. Carfang, Chief
Reactor Projects Branch 1
Division of Reactor Projects

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting an integrated inspection at James A. FitzPatrick Nuclear Power Plant, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

None.

PLANT STATUS

FitzPatrick began the inspection period at rated thermal power. On July 25, 2020, FitzPatrick began end of cycle coastdown. On August 12, 2020, operators reduced reactor power to 85 percent to perform control rod hydraulic control unit maintenance. Following repairs to the control rod hydraulic control unit, operators restored reactor power to maximum achievable thermal power the same day. On September 5, 2020, operators reduced reactor power to 80 percent due to degraded feedwater heater performance. Following removal of a condensate booster pump from service to stabilize feedwater heating, operators maintained power at 80 percent until September 13, 2020. On September 13, 2020, operators removed FitzPatrick from service to commence refueling outage 24 (J1R24). FitzPatrick remained offline for the remainder of the inspection period.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards. Starting on March 20, 2020, in response to the National Emergency declared by the President of the United States on the public health risks of the coronavirus (COVID-19), resident and regional inspectors were directed to begin telework and to remotely access licensee information using available technology. During this time the resident inspectors performed periodic site visits each week, increasing the amount of time on site as local COVID-19 conditions permitted. As part of their onsite activities, resident inspectors conducted plant status activities as described in IMC 2515, Appendix D, observed risk significant activities, and completed on site portions of IPs. In addition, resident and regional baseline inspections were evaluated to determine if all or portion of the objectives and requirements stated in the IP could be performed remotely. If the inspections could be performed remotely, they were conducted per the applicable IP. In some cases, portions of an IP were completed remotely and on site. The inspections documented below met the objectives and requirements for completion of the IP.

REACTOR SAFETY

71111.04 - Equipment Alignment

Partial Walkdown Sample (IP Section 03.01) (5 Samples)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

- (1) 'A' emergency diesel generator on July 8, 2020
- (2) 'A' residual heat removal service water loop 'A' on July 23, 2020
- (3) 'A' residual heat removal system in shutdown cooling mode on September 15, 2020
- (4) Decay heat removal standby diesel generator on September 19, 2020
- (5) Decay heat removal system on September 20, 2020

Complete Walkdown Sample (IP Section 03.02) (2 Samples)

- (1) The inspectors evaluated system configurations during a complete walkdown of the reactor building closed loop cooling system on August 5, 2020.
- (2) The inspectors evaluated system configurations during a complete walkdown of the standby gas treatment system on August 21, 2020.

71111.05 - Fire Protection

Fire Area Walkdown and Inspection Sample (IP Section 03.01) (6 Samples)

The inspectors evaluated the implementation of the fire protection program by conducting a walkdown and performing a review to verify program compliance, equipment functionality, material condition, and operational readiness of the following fire areas:

- (1) East diesel fire pump room, elevation 255', fire area/zone IB/FP-3, on July 7, 2020
- (2) Reactor building, elevation 344' and 369', fire area/zone IX/RB-1A, on July 15, 2020
- (3) Turbine building south, elevation 252', fire area/zone IE/TB-1 on September 17, 2020
- (4) Reactor building, elevation 242' & 227', fire area/zone XVII/RB-1E, on September 19, 2020
- (5) Drywell, elevation 268', fire area/zone XIV/PC-1 on September 21, 2020
- (6) Turbine building, elevation 252' and 272', fire area/zone IE/TB-1, OR-1 on September 24, 2020

Fire Brigade Drill Performance Sample (IP Section 03.02) (1 Sample)

- (1) The inspectors evaluated the onsite fire brigade performance during an unannounced fire drill in the auxiliary boiler room on August 3, 2020.

71111.07A - Heat Sink Performance

Annual Review (IP Section 03.01) (1 Sample)

The inspectors evaluated readiness and performance of:

- (1) Reactor building closed loop cooling heat exchanger 'C'

71111.08G - Inservice Inspection Activities (BWR)

BWR Inservice Inspection Activities Sample - Nondestructive Examination and Welding Activities (IP Section 03.01) (1 Sample)

- (1) The inspectors verified that the reactor coolant system boundary, reactor vessel internals, risk-significant piping system boundaries, and containment boundary are appropriately monitored for degradation and that repairs and replacements were appropriately fabricated, examined and accepted by reviewing the following activities from September 21, 2020 to October 8, 2020:
 - Ultrasonic examination of residual heat removal valve 81A to reducing tee weld overlay on weld 24-10-130 (J1R24-VEN-001)

- Ultrasonic examination of dissimilar metal weld overlay of CRD N-9-C1 nozzle to cap (ISI-UT-20-004)
- Ultrasonic examination of recirculation system valve to elbow weld overlay on weld 28-02-2-92 (ISI-UT-20-015)
- Ultrasonic examination of core spray safe end to nozzle dissimilar metal weld N-5A-SE (APR-R24-02)
- Ultrasonic examination of recirculation system nozzle N-1B-IR inner radius (J1R24-VEN-005)
- Liquid penetrant examination of nuclear boiler instrumentation nozzle N12A-SE (ISI-PT-20-001)
- Liquid penetrant examination of reactor core isolation cooling turbine steam inlet isolation valve 13 MOV-131 welds W2 and W4 (B20PT026)
- Magnetic particle examination of reactor core isolation cooling turbine steam inlet isolation valve 13MOV-131 welds W1 and W5 (B20MT046)
- Welding activities associated with the replacement of reactor core isolation cooling turbine steam inlet isolation valve 13MOV-131 welds W1, W2, W4 and W5 (4937146)
- Visual examination (VT-G) of ASME Section XI IWE accessible containment surfaces including general exam of torus interior, reactor building, drywell dome interior, drywell head bolting and drywell dome flange surface (CISI-VT-20-001 to CISI-VT-20-008, CISI-VT-20-013, CISI-VT-20-015)

71111.11Q - Licensed Operator Requalification Program and Licensed Operator Performance

Licensed Operator Performance in the Actual Plant/Main Control Room (IP Section 03.01)
(1 Sample)

- (1) The inspectors observed and evaluated licensed operator performance in the control room during a power reduction to begin refueling and maintenance outage J1R24 on September 13, 2020

Licensed Operator Requalification Training/Examinations (IP Section 03.02) (1 Sample)

- (1) The inspectors observed a simulator evaluation that involved implementation of digital feedwater level control during reactor startup and during transients on September 3, 2020

71111.12 - Maintenance Effectiveness

Maintenance Effectiveness (IP Section 03.01) (2 Samples)

The inspectors evaluated the effectiveness of maintenance to ensure the following structures, systems, and components (SSCs) remain capable of performing their intended function:

- (1) Torus internal structural monitoring on September 18, 2020
- (2) Drywell internal structural monitoring on September 22, 2020

Quality Control (IP Section 03.02) (1 Sample)

The inspectors evaluated the effectiveness of maintenance and quality control activities to ensure the following SSC remains capable of performing its intended function:

- (1) Commercial dedication of emergency diesel fuel oil and breaker component on September 9, 2020

71111.13 - Maintenance Risk Assessments and Emergent Work Control

Risk Assessment and Management Sample (IP Section 03.01) (7 Samples)

The inspectors evaluated the accuracy and completeness of risk assessments for the following planned and emergent work activities to ensure configuration changes and appropriate work controls were addressed;

- (1) Elevated risk during residual heat removal service water heat exchanger outlet isolation valve, 10MOV-89B, repair on July 29, 2020
- (2) 'B' reactor feedwater pump motor gear unit failure on July 29, 2020
- (3) Emergent repair of hydraulic control unit associated with control rod 14-27 on August 12, 2020
- (4) Elevated risk while placing shutdown cooling in service on September 14, 2020
- (5) Elevated risk during reactor cavity flood-up on September 15, 2020
- (6) Elevated risk during planned 'B' 125 volts direct current battery maintenance on September 18, 2020
- (7) Elevated risk during reactor cavity draindown on September 28, 2020

71111.15 - Operability Determinations and Functionality Assessments

Operability Determination or Functionality Assessment (IP Section 03.01) (5 Samples)

The inspectors evaluated the licensee's justifications and actions associated with the following operability determinations and functionality assessments:

- (1) 'A' emergency service water supply piping line 46-10"-WES-151-37 through-wall leak on July 21, 2020
- (2) 'A' emergency service water supply piping line 46-10"-WES-151-37 through-wall leak flaw extent of condition evaluation on July 30, 2020
- (3) 'A' residual heat removal system snubber support failure on August 11, 2020
- (4) P4 test port leak on hydraulic control unit 14-27 on August 11, 2020
- (5) Reactor core isolation cooling condensate storage tank 'B' logic level switch on September 2, 2020

71111.18 - Plant Modifications

Temporary Modifications and/or Permanent Modifications (IP Section 03.01 and/or 03.02) (1 Sample)

The inspectors evaluated the following temporary or permanent modifications:

- (1) Permanent Modification: Engineering Change 629806, 10700 Bus Trip on Loss of Coolant Accident Project to Support Replacement of Normal Station Service Transformer 71T-4, on September 15, 2020

71111.19 - Post-Maintenance Testing

Post-Maintenance Test Sample (IP Section 03.01) (7 Samples)

The inspectors evaluated the following post maintenance test activities to verify system operability and functionality:

- (1) 88CR-2, post maintenance test review for the reactor building crane following removal of the trolley festoon cable tray on July 15, 2020
- (2) 10MOV-89B, residual heat removal service water heat exchanger outlet isolation valve following packing replacement on August 11, 2020
- (3) 23MOV-15, high pressure coolant injection inboard steam isolation valve following diagnostic testing on September 23, 2020
- (4) ST-22J, safety relief valve electric lift logic system functional and simulated actuation test following main valve replacement on September 25, 2020
- (5) 12MOV-69, reactor water cleanup outboard isolation valve following stem nut cleaning and repacking on September 25, 2020
- (6) High pressure coolant injection system turbine overspeed trip test following preventative maintenance overhaul on September 27, 2020 and September 29, 2020
- (7) ST-39H, reactor pressure vessel leakage test following refueling on September 30, 2020

71111.20 - Refueling and Other Outage Activities

Refueling/Other Outage Sample (IP Section 03.01) (1 Partial)

- (1) The inspectors evaluated refueling outage J1R24 activities from September 13, 2020 through September 30, 2020

71111.22 - Surveillance Testing

The inspectors evaluated the following surveillance tests:

Surveillance Tests (other) (IP Section 03.01) (5 Samples)

- (1) ST-24J, RCIC Flow Rate and Inservice Test (IST) on September 2, 2020
- (2) ST-39B-X7A, B, C, D Type C Leak Test Main Steam Line MSIV on September 15, 2020
- (3) ST-22A, ADS Simulated Automatic Actuation Test on September 25, 2020

- (4) ST-9CB, Emergency Diesel Generator (EDG) B and D Load Sequencing Test and 4KV Emergency Power System Voltage Relays Instrument Functional Test on September 27, 2020
- (5) ST-6M, Standby Liquid Control Recirculation, Injection Test (IST, ISI) on September 29, 2020

Containment Isolation Valve Testing (IP Section 03.01) (1 Sample)

- (1) ST-1B, Main Steam Isolation Valve Fast Closure Test on September 14, 2020

71114.02 - Alert and Notification System Testing

Inspection Review (IP Section 02.01-02.04) (1 Sample)

- (1) The inspectors evaluated Exelon's maintenance and testing of the FitzPatrick alert and notification system on July 13–16, 2020, for the period of July 2018 through June 2020

71114.03 - Emergency Response Organization Staffing and Augmentation System

Inspection Review (IP Section 02.01-02.02) (1 Sample)

- (1) The inspectors evaluated the readiness of FitzPatrick's emergency preparedness organization on July 13–16, 2020

71114.04 - Emergency Action Level and Emergency Plan Changes

Inspection Review (IP Section 02.01-02.03) (1 Sample)

- (1) The inspectors evaluated the following submitted Emergency Action Level and Emergency Plan changes onsite on July 13–16, 2020:
 - Evaluation No. 19-54, EP-AA-121-F-11, James A. FitzPatrick Equipment Matrix, Revision 3
 - Evaluation No. 19-62, EAP-44, Core Damage Estimation, Revision 9
 - Evaluation No. 20-03, EP-AA-111-F-15, FitzPatrick PAR Flowchart, Revision B

This evaluation does not constitute NRC approval

71114.05 - Maintenance of Emergency Preparedness

Inspection Review (IP Section 02.01 - 02.11) (1 Sample)

- (1) The inspectors evaluated the maintenance of the emergency preparedness program on July 13–16, 2020, for the period of July 2018 through June 2020

71114.06 - Drill Evaluation

Select Emergency Preparedness Drills and/or Training for Observation (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated the conduct of a routine emergency planning drill on August 13, 2020

RADIATION SAFETY

71124.01 - Radiological Hazard Assessment and Exposure Controls

Radiological Hazard Assessment (IP Section 03.01) (1 Partial)

- (1) The inspectors reviewed logs and discussed the conduct of radiologically significant work via telephone. Comprehensive documents for two jobs were requested. This was done remotely due to the COVID-19 public health emergency.

Instructions to Workers (IP Section 03.02) (1 Partial)

- (1) Comprehensive documents for two radiologically significant jobs were requested. This was done remotely due to the COVID-19 public health emergency.

Contamination and Radioactive Material Control (IP Section 03.03) (1 Partial)

The inspectors evaluated licensee processes for monitoring and controlling contamination and radioactive material.

- (1) The inspectors reviewed logs and discussed controls via telephone. This was done remotely due to the COVID-19 public health emergency.

Radiological Hazards Control and Work Coverage (IP Section 03.04) (1 Partial)

The inspectors evaluated in-plant radiological conditions during facility walkdowns and observation of radiological work activities.

- (1) The inspectors reviewed initial radiological surveys of the drywell and the turbine stop valves. The inspector requested comprehensive documents for two radiologically significant jobs. This was done remotely due to the COVID-19 public health emergency.

High Radiation Area and Very High Radiation Area Controls (IP Section 03.05) (1 Partial)

The inspectors evaluated licensee controls of the following High Radiation Areas and Very High Radiation Areas:

- (1) The inspectors reviewed licensee procedures for high and very high radiation area controls and reviewed the results of any events that were documented in Exelon's issue reporting system

Radiation Worker Performance and Radiation Protection Technician Proficiency (IP Section 03.06) (1 Partial)

- (1) The inspectors reviewed logs and discussed the conduct of radiologically significant work via telephone

71124.05 - Radiation Monitoring Instrumentation

Walkdowns and Observations (IP Section 03.01) (8 Samples)

The inspectors evaluated the following radiation detection instrumentation during plant walkdowns:

- (1) Area radiation monitors in the reactor building, turbine building, and radioactive waste building
- (2) Ion chambers in use in the reactor building, turbine building, and radioactive waste building
- (3) Portable ion chambers and telepoles stored 'ready for use'
- (4) Portable friskers used at the exit to the radiologically controlled area
- (5) Post-accident liquid sampling station
- (6) Process radiation monitors in the control room
- (7) Remote gaseous and particulate air monitoring for the drywell
- (8) Whole body counter

Calibration and Testing Program (IP Section 03.02) (13 Samples)

The inspectors evaluated the calibration and testing of the following radiation detection instruments:

- (1) Eberline RO20AA, SN 12532
- (2) Eberline MS-2, SN 1623
- (3) Eberline MS-3, SN 1036
- (4) Ludlum-3, SN 313206
- (5) Ludlum-177, SN 323559
- (6) Ludlum L-2000, SN 302439
- (7) Mirion BAK-4280, SN 427017-131
- (8) Mirion RDS-31ITX, SN 712470
- (9) Mirion RDS-31ITXSD, SN 720241
- (10) Thermo Fisher AMS-4, SN 13067
- (11) Thermo Fisher AMS-4, SN 13070
- (12) Radeco AVS, SN 14613
- (13) Radeco AVS Gooseneck, SN 1011330082

Effluent Monitoring Calibration and Testing Program Sample (IP Sample 03.03) (2 Samples)

The inspectors evaluated the calibration and maintenance of the following radioactive effluent monitoring and measurement instrumentation:

- (1) Radwaste building exhaust radiation monitors 17RM-458A and 17RM-458B
- (2) Reactor building closed loop cooling process radiation monitor 17RM-352

OTHER ACTIVITIES – BASELINE

71151 - Performance Indicator Verification

The inspectors verified licensee performance indicators submittals listed below:

EP01: Drill/Exercise Performance (IP Section 02.12) (1 Sample)

(1) April 1, 2019 - March 31, 2020

EP02: ERO Drill Participation (IP Section 02.13) (1 Sample)

(1) April 1, 2019 - March 31, 2020

EP03: Alert & Notification System Reliability (IP Section 02.14) (1 Sample)

(1) April 1, 2019 - March 31, 2020

OR01: Occupational Exposure Control Effectiveness Sample (IP Section 02.15) (1 Sample)

(1) April 1, 2019 - March 31, 2020

PR01: Radiological Effluent Technical Specifications/Offsite Dose Calculation Manual Radiological Effluent Occurrences (RETS/ODCM) Radiological Effluent Occurrences Sample (IP Section 02.16) (1 Sample)

(1) April 1, 2019 - March 31, 2020

INSPECTION RESULTS

No findings were identified.

EXIT MEETINGS AND DEBRIEFS

The inspectors confirmed that proprietary information was controlled to protect from public disclosure.

- On October 22, 2020, the inspectors presented the integrated inspection results to Mr. Pat Navin, Site Vice President, and other members of the licensee staff.
- On July 16, 2020, the inspectors presented the emergency preparedness program inspection results to Mr. Pat Navin, Site Vice President, and other members of the licensee staff.
- On July 23, 2020, the inspectors presented the radiation monitoring instrumentation inspection results to Mr. Timothy Peter, Plant Manager, and other members of the licensee staff.
- On October 8, 2020, the inspectors presented the inservice inspection activities inspection results to Mr. Pat Navin, Site Vice President, and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.04	Drawings	FM-15A	Flow Diagram Reactor Building Closed Loop Cooling	67
		FM-20A	Flow Diagram Residual Heat Removal System 10	72
		FM-20B	Flow Diagram Residual Heat Removal System 10	73
		FM-48A	Flow Diagram Standby Gas Treatment System 01-125	32
		FM-93A	Flow Diagram Fuel Oil Line Emergency Diesel Generators System 93	22
		FM-941	Flow Diagram Air Start-Up Lines Emergency Diesel Generators System 93	13
		FM-94A	Flow Diagram Air Start-up Lines Emergency Diesel Generators	13
	Procedures	OP-13	Residual Heat Removal System	98
		OP-13C	Residual Heat Removal System	16
		OP-20	Standby Gas Treatment System	41
		OP-22	Diesel Generator Emergency Power	69
		OP-30B	Decay Heat Removal System	20
		OP-40	Reactor Building Closed Loop Cooling	53
OSP-30B.001	Operation of the Decay Heat Removal Standby Diesel Generator	7		
71111.05	Corrective Action Documents	04371127		
	Fire Plans	PFP-PWR 33	Pump Rooms (Screenwell)/Elevation 255' Fire Area/Zone XII/SP-1, XIII/SP-2, IB/FP-1, FP-3	2
		PFP-PWR14	Crescent Area-East/Elev.227', 242' Fire Area/Zone XVII/RB-1E	3
		PFP-PWR18	Drywell Elevation 268' Fire Area/Zone XIV/PC-1	0
		PFP-PWR27	Reactor Building / Elevation 344' Fire Area/Zone IX/RB-1A	0
		PFP-PWR28	Reactor Building / Elevation 369', Fire Area/Zone IX/RB-1A	0
		PFP-PWR42	Turbine Building North/Elev. 252' Fire Area/Zone IE/TB-1	3
		PFP-PWR43	Turbine Building-South Elev. 252' Fire Area/Zone IE/TB-1	3
		PFP-PWR45	Turbine Building-North/Elev. 272' Fire Area/Zone IE/TB-1	07
PFP-PWR46	Turbine Building-South/Elev. 272' Fire Area/Zone IE/TB-1,	5		

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			OR-2	
	Procedures	OP-AA-201-003	Fire Drill Performance	17
71111.07A	Drawings	4.95-15	Heat Exchanger Plugging Map 15E-1C RBCLC	3
	Procedures	SP-01.25	Reactor Building Closed Loop Cooling Sampling and Analysis	21
		SP-04.03	Service and Circulating water Systems Chemical Treatment	19
	Work Orders	04682358		
71111.08G	Corrective Action Documents		CR-JAF-2008-3311	09/23/2008
		4138318	Through Wall Leak on ESW Line 3-WES-151-136	05/16/18
	NDE Reports	APR-R24-02	Safe End to Nozzle	09/21/2020
		B20MT046	3" Pipe / Valve	10/01/2020
		B20PT026	Valve to Elbow W2 and Elbow to Pipe W4	10/01/2020
		CISI-VT-20-001	IWE General Exam Torus interior RB 227 to 272	09/24/2020
		CISI-VT-20-002	IWE General Exam RF Cavity 340-369	09/21/2020
		CISI-VT-20-003	IWE General Exam Torus supports RB 227	09/21/2020
		CISI-VT-20-004	IWE General Exam RB 326 - 344	09/24/2020
		CISI-VT-20-005	IWE General Exam RB 227-272	09/21/2020
		CISI-VT-20-006	IWE General Exam RB 344 - 369	09/20/2020
		CISI-VT-20-007	IWE General Exam RB 300 - 326	09/24/2020
		CISI-VT-20-008	IWE General Exam RB 272 -300	09/24/2020
		CISI-VT-20-013	IWE General Exam Drywell Dome RF 369	09/24/2020
		CISI-VT-20-015	Drywell Head Bolting	09/24/2020
		ISI-PT-20-001	Instrumentation Nozzle Weld	09/18/2020
		ISI-UT-20-004	Nozzle to Cap Overlay	09/22/2020
		ISI-UT-20-015	Valve to Elbow	09/24/2020
		J1R24-VEN-001	Valve to Tee Weld Overlay	09/22/2020
		J1R24-VEN-005	Nozzle Inner Radius	09/22/2020
	Work Orders	04937146	Replace 13MOV-131 in R24	07/21/2020
71111.11Q	Miscellaneous	N-JF-OPS-LORT-DFWLC-2005	Requalification Guide for DFWLC Mod 20-05	0
71111.12	Miscellaneous	Purchase Order 0714821	EDG A & C Fuel Oil Receipt 34461	
		Purchase Order 0725006	EDG B & D Fuel Oil Receipt 35606	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
	Procedures	Purchase Order 0726089	EDG A & C Fuel Oil Receipt 35610	
		AP-15.12	Receiving Inspection for Dedication of No. 2 Diesel Fuel Oil	9
		ER-AA-450	Structures Monitoring	9
		NO-AA-30	Independent Inspection Process Description	7
		ST-15B	Suppression Chamber and Drywell Deterioration Inspection	11
71111.13	Corrective Action Documents	04359266		
		04361956		
	Drawings	2.04-30	RFP Turbine Control Diagram	C
		FM-27B	Flow Diagram Control Rod Drive System 03	35
	Miscellaneous	G080-0056	Hydraulic Control Unit Part No's 729E950 G1 thru G6	1
	Procedures	AOP-27	Control Rod Malfunction	15
		MP-004.01	Disassembly of Reactor Vessel for Refueling (ISI)	47
		OP-13D	RHR – Shutdown Cooling	37
		OP-25	Control Rod Drive Hydraulic System	093
		OP-30A	Refueling Water Level Control	24
		OP-43A	125 VDC Power System	32
		OP-65	Startup and Shutdown Procedures	127
	Work Orders	05068725		
71111.15	Corrective Action Documents	04109193		
		04164618		
		04344822		
		04361956		
		4351640-09		
	Drawings	1.61.152	Elem Diag RCIC System	9
		FM-46B	Flow Diagram Emergency Service Water System 46 & 15	57
	Engineering Changes	631938	ESW N-513 Thru-Wall Leak Evaluation 46-10-WES-151-37 OP Eval	0
	Miscellaneous	4351640	Operability Evaluation	
	Procedures	OP-AA-108-115	Operability Determination	23
71111.18	Corrective Action Documents	04369111		
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		00004		
	Engineering Changes	629806	10700 Bus Trip on Loss of Coolant Accident Project to Support Replacement of Normal Station Service Transformer 71T-4	0
71111.19	Corrective Action Documents	04371087		
		CR-JAF-2013-01218		
	Drawings	6.37-362	Valve Outline Drawing, 23MOV-15	C
		FM-24A	Flow Diagram Reactor Water Cleanup System 12	73
	Miscellaneous	A391-0057F(A10)	Installation Operation and Maintenance Manual Double Disc Gate Valve	8
		TP16-1-112	Recommendations to Resolve Flowserve 10CFR Part 21 Notification Affecting Anchor Darling Double Disc Gate Valve Wedge Pin Failures	5 (Proprietary)
	Procedures	MA-AA-723-300	Diagnostic Testing of Motor Operated Valves	13
		MA-AA-723-302	Installation and Checkout of Quick Stem Sensor (QSS) on Valve Stems	5
		MP-059.60	Anchor Darling Pressure Seal Gate and Globe Valve (ISI)	18
		MP-059.78	Disconnecting Leads for MOV Maintenance/Testing	07
		MST-023.01	HPCI Turbine Overspeed Trip Test and Calibration	9
		ST-22J	SRV Electric Lift Logic System Functional and Simulated Automatic Actuation Test	9
		ST-22J	SRV Electric Lift Logic System Functional and Simulated Automatic Actuation Test	9A
	Work Orders	ST-22J SRV	Electric Lift Logic System Functional and Simulated Automatic Actuation Test	10
		0339333		
		05061232		
71111.20	Procedures	OP-65	Startup and Shutdown Procedure	128
	Work Orders	04738808		
71111.22	Corrective Action Documents	04369251		
		04369253		
		04369255		
		04369727		

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	Drawings	04369728		
		1.83-37	Auto Depressurization System	22
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		1.83-39	Elem Drawing - Auto Depressurization Sys	M
		ESK-11AAu	Elementary Diagram 09-ECCS1-EP and 09-ECCS2-EP	1
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	Procedures	ST-1B	Main Steam Isolation Valve Fast Closure Test	27
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		ST-39B-X7C	Type C Leak Test Main Steam Line C MSIVs (IST)	17
		ST-39B-X7D	Type C Leak Test Main Steam Line D MSIVs (IST)	17
		ST-6M	Standby Liquid Control Recirculation, Injection Test (IST, ISI)	11
	Work Orders	ST-9CB	Emergency Diesel Generator (EDG) B and D Load Sequencing Test and 4KV Emergency Power System Voltage Relays Instrument Functional Test	6
		04877902		
		04878371		
		04878373		
		04878375		
04878383				
04878410				
05024253				
71114.02	Miscellaneous	Design Report	Nine Mile Point Nuclear Generating Station and James A. FitzPatrick Nuclear Power Plant Public Alert and Notification System	1
71114.03	Miscellaneous	EP-AA-1014	Radiological Emergency Plan Annex for James A. FitzPatrick Station	4
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71114.04	Procedures	EP-AA-120-1001	10 CFR 50.54(q) Change Evaluation	10
71114.05	Corrective Action Documents	04170901		
		04227133		
		04271107		
		04302779		
		04303614		
	04323222			
	Miscellaneous		Nine Mile Point Nuclear Station and James A. FitzPatrick Nuclear Power Plant 2019 Population Update Analysis	0
	Procedures	EP-AA-121-F-11	James A. FitzPatrick Equipment Matrix	3
71114.06	Procedures	EP-AA-112-100-F-01	Shift Emergency Director Checklist	AB
		EP-AA-112-F-01	Command and Control Turnover Briefing Form	H
		EP-AA-122-300-F-04	Drill & Exercise Post-Event Critique & Report Development Guidance	C
71124.05	Corrective Action Documents	04268293		
		04269079		
		04277823		
	Procedures	RP-AA-700	Controls for Radiation Protection Instrumentation	8