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U.S. Nuclear Regulatory Commission
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Southern Nuclear Operating Company
Vogtle Electric Generating Plant Unit 3
ITAAC Closure Notification on Completion of ITAAC 3.3.00.02f [Index Number 774]

Ladies and Gentlemen:

In accordance with 10 CFR 52.99(c)(1), the purpose of this letter is to notify the Nuclear Regulatory Commission (NRC) of the completion of Vogtle Electric Generating Plant (VEGP) Unit 3 Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC) Item 3.3.00.02f [Index Number 774] for verifying that the key dimensions of the as-built nuclear island structures are consistent with the dimensions defined on VEGP Unit 3 Combined License (COL) Appendix C Table 3.3-5. The closure process for this ITAAC is based on the guidance described in NEI 08-01, "Industry Guideline for the ITAAC Closure Process under 10 CFR Part 52," which was endorsed by the NRC in Regulatory Guide 1.215.

This letter contains no new NRC regulatory commitments. Southern Nuclear Operating Company (SNC) requests NRC staff confirmation of this determination and publication of the required notice in the Federal Register per 10 CFR 52.99.

If there are any questions, please contact Tom Petrak at 706-848-1575.

Respectfully submitted,

Thomas A. Petrak for

Michael J. Yox
Regulatory Affairs Director Vogtle 3 & 4

Enclosure: Vogtle Electric Generating Plant (VEGP) Unit 3
Completion of ITAAC 3.3.00.02f [Index Number 774]

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Southern Nuclear Operating Company
ND-20-0655
Enclosure

Vogtle Electric Generating Plant (VEGP) Unit 3
Completion of ITAAC 3.3.00.02f [Index Number 774]

ITAAC Statement

Design Commitment

2.f) The key dimensions of nuclear island structures are defined on Table 3.3-5.

Inspections/Tests/Analyses

An inspection will be performed of the as-built configuration of the nuclear island structures.

Acceptance Criteria

A report exists and concludes that the key dimensions of the as-built nuclear island structures are consistent with the dimensions defined on Table 3.3-5.

ITAAC Determination Basis

Inspections of the as-built configuration of the nuclear island structures were conducted to confirm that the key dimensions of the as-built nuclear island structures are consistent with the dimensions defined on VEGP Unit 3 Combined License (COL) Appendix C Table 3.3-5 (Attachment A).

Inspection of the distance from the bottom of containment sump to top surface of embedded containment shell key dimension was performed per ITAAC 3.3.00.09 and is documented in the ITAAC Completion Package for ITAAC 3.3.00.09 (Reference 1). Inspection of the site grade level relative to the design plant grade floor elevation key dimension was performed per ITAAC 3.3.00.02b and is documented in the ITAAC Completion Package for ITAAC 3.3.00.02b (Reference 2). For the remainder of the key dimensions identified in Attachment A inspections were performed by surveying the as-built configuration of the nuclear island structures using survey equipment in accordance with site survey and measurement procedures. The results of the as-built measured dimensions are summarized in Attachment A. The as-built measured dimension inspection results were compared to the nominal dimensions defined in Attachment A to validate that the dimensions of the as-built nuclear island structures are consistent with the nominal dimensions defined on Attachment A.

The as-built measured dimension inspection results are documented in the Principal Closure Documents SV3-1000-ITR-800774 and SV3-1000-801774 (References 3 & 4) supporting the ITAAC 3.3.00.02f Completion Package (Reference 5) and verify that the key dimensions of the as-built nuclear island structures are consistent with the dimensions defined on VEGP Unit 3 COL Appendix C Table 3.3-5.

Principal Closure Documents SV3-1000-ITR-800774 and SV3-1000-801774 (References 3 & 4) are available for NRC inspection as part of the ITAAC 3.3.00.02f Completion Package.

ITAAC Finding Review

In accordance with plant procedures for ITAAC completion, Southern Nuclear Operating Company (SNC) performed a review of all findings pertaining to the subject ITAAC and associated corrective actions. This review found there are no relevant ITAAC findings associated with this ITAAC. The ITAAC completion review is documented in the ITAAC Completion Package for ITAAC 3.3.00.02f (Reference 5) and is available for NRC review.

ITAAC Completion Statement

Based on the above information, SNC hereby notifies the NRC that ITAAC 3.3.00.02f was performed for VEGP Unit 3 and that the prescribed acceptance criteria were met.

Systems, structures, and components verified as part of this ITAAC are being maintained in their as-designed, ITAAC compliant condition in accordance with approved plant programs and procedures.

References (available for NRC inspection)

1. SVP_SV0_002198, Attachment 2, ITAAC 3.3.00.09 Unit 3 Completion Package.
2. 3.3.00.02b-U3-CP-Rev0, ITAAC Completion Package.
3. SV3-1000-ITR-800774, Rev. 0; Unit 3 Key Dimensions of Nuclear Island Structures: (X1-X7) ITAAC 3.3.00.02f (774).
4. SV3-1000-ITR-801774, Rev. 1; Unit 3 Key Dimensions of Nuclear Island Structures: (Vertical Dimensions) ITAAC 3.3.00.02f (774).
5. 3.3.00.02f-U3-CP-Rev0 ITAAC Completion Package
6. NEI 08-01, "Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52"

Attachment A: Excerpt from VEGP Unit 3 COL Appendix C - Table 3.3-5

Key Dimension *	Nominal Dimension *	Tolerance *	As-built Measured Dimension
Distance between Outside Surface of walls at Column Line I & N when Measured at Column Line 1	91 ft-0 in	+3 ft -1 ft	90 ft-9 5/8 in X1 (Ref. 3)
Distance from Outside Surface of wall at Column Line 1 to Column Line 7 when Measured at Column Line I	138 ft-0 in	+3 ft -1 ft	138 ft-0 in X2 (Ref. 3)
Distance from Outside Surface of wall at Column Line 11 to Column Line 7 when Measured at Column Line I	118 ft-0 in	+3 ft -1 ft	118 ft-0 in X3 (Ref. 3)
Distance between Outside Surface of walls at Column Line I & Q when Measured at Column Line 11	117 ft-6 in	+3 ft -1 ft	116 ft-7 3/16 in X4 (Ref. 3)
Distance from Outside Surface of wall at Column Line Q to Column Line N when Measured at Column Line 11	29 ft-0 in	+3 ft -1 ft	29 ft-0 in X5 (Ref. 3)
Distance between Outside Surface of shield building wall to shield building centerline when Measured on West Edge of Shield Building	72 ft-6 in	+3 ft -1 ft	72 ft-6 in X6 (Ref. 3)
Distance between shield building centerline to Reactor Vessel centerline when Measured along Column Line N in North-South Direction	7 ft-6 in	± 3 in	7 ft-6 in X7 (Ref. 3)
Distance from Bottom of Containment Sump to Top Surface of Embedded Containment Shell	2 ft-8 in	± 3 in	2 ft-10 in See Reference 1
Distance from top of Basemat to Design Plant Grade	33 ft-6 in	± 1 ft	Max 33 ft-6 1/4 in Min 33 ft-5 13/16 in (Ref. 4)
Distance of Design Plant Grade (Floor elevation 100'-0") relative to Site Grade	0 ft	± 3 ft-6 in	See Reference 2
Distance from Design Plant Grade to Top Surface of Shield Building Roof	229 ft-0 in	± 1 ft	Max 229 ft-0 1/16 in Min 228 ft-8 13/16 in (Ref. 4)

* Excerpt from VEGP Unit 3 COL Appendix C - Table 3.3-5