UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
801 WARRENVILLE ROAD
LISLE, ILLINOIS 60532-4351

May 27, 1998

Mr. M. Wadley, Vice President
Nuclear Generation
Northern States Power Company
414 Nicollet Mall
Minneapolis, MN 55401

SUBJECT: PLANT PERFORMANCE REVIEW (PPR) - MONTICELLO

Dear Mr. Wadley:

The NRC staff recently completed the semiannual Plant Performance Review (PPR) of the Monticello Nuclear Plant. The staff conducts these reviews for all operating nuclear power plants to develop an integrated understanding of safety performance. The results are used by NRC management to facilitate planning and allocation of inspection resources. The PPR for Monticello involved the participation of all technical divisions in evaluating inspection results and safety performance information for the period October 1997 through March 1998. The PPRs provide NRC management with a current summary of licensee performance and serve as inputs to the NRC Systematic Assessment of Licensee Performance (SALP) and senior management meeting (SMM) reviews.

Enclosure 1 contains a historical listing of plant issues, referred to as the Plant Issues Matrix (PIM), that was considered during this PPR process to arrive at an integrated view of licensee performance trends. The PIM includes only items from inspection reports or other docketed correspondence between the NRC and Northern States Power Company. The PPR may also have considered some predecisional and draft material that does not appear in the attached PIM, including observations from events and inspections that had occurred since the last NRC inspection report was issued, but had not yet received full review and consideration. This material will be placed in the PDR as part of the normal issuance of NRC inspection reports and other correspondence.

This letter advises you of our planned inspection effort resulting from the Monticello PPR review. It is provided to minimize the resource impact on your staff and to allow for scheduling conflicts and personnel availability to be resolved in advance of inspector arrival onsite. Enclosure 2 details our inspection plan for the next 6 months. The rationale or basis for each inspection outside the core inspection program is provided so that you are aware of the reason for emphasis in these program areas. Resident inspections are not listed due to their ongoing and continuous nature.

We will inform you of any changes to the inspection plan. If you have any questions, please contact me at (630) 829-9872.

Sincerely,

Michael Kunowski for

J. W. McCormick-Barger, Chief
Reactor Projects Branch 7

Docket No.: 50-263
License No.: DPR-22

Enclosures: 1. Plant Issues Matrix
2. Inspection Plan

cc w/encls: Plant Manager, Monticello
State Liaison Officer, State of Minnesota

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PLANT ISSUES MATRIX
Monticello

5/19/98

Search Sorted by Date (Descending) and SMM Codes (Ascending): Search Column = "SALP" ; SALP Area = "Operations" ; Beginning Date = "10/1/97" ; Ending Date = "3/31/98"

#	DATE	TYPE	SOURCE	ID BY	SALP	SMM CODES	DESCRIPTION
1	2/23/1998	VIO/SL-IV	IR 98002	NRC	Operations	1A	Three concerns were identified with respect to the restoration of equipment to operational readiness after maintenance activities were performed. These included not providing adequate instructions for restoring a reactor core isolation cooling valve to service; declaring a residual heat removal service water pump operable with an outstanding hold-and-secure card located on the control room hand-switch; and not providing adequate instructions to fill a portion of the fire protection system. Two violations (of Criterion V of 10 CFR Part 50, Appendix B) for inadequate procedures and failure to follow administrative procedures were cited. The tracking numbers for these violations are No. 50-263/98002-01(DRP) and No. 50. 263/98002-02(DRP).
2	2/23/1998	Positive	IR 98002	NRC	Operations	3A	The training provided to operations personnel for the handling and inspection of new fuel bundles and channels was adequate. Operators conducted the inspection and handling activities in a controlled manner.
3	2/23/1998	Positive	IR 98002	NRC	Operations	3A 1A	Operator performance during routine operations and surveillance test activities was good. With one exception, operators were knowledgeable of plant conditions and equipment status. An operator's questioning attitude prevented a potential Group I isolation during a routine main steam line high flow surveillance test and standby gas treatment system operability test.
4	2/23/1998	Positive	IR 98002	NRC	Operations	3B	Operations personnel displayed good teamwork during three simulator training scenarios. The senior reactor operator verified that actions were taken in accordance with the emergency operating procedures and displayed good command and control over the crew.
5	1/12/1998	Positive	IR 97018	NRC	Operations	1A 3A	Two reactor power reductions (one on December 13, 1997, and one on January 10, 1998) were conducted in a controlled manner. Operators responded appropriately to a low circulating water and intake structure basin level condition. Power ascensions to 100 percent were performed without problems. Teamwork/Skill Level
6	1/12/1998	Positive	IR 97018	NRC	Operations	3A	In general, the conduct of operations was acceptable. Operator performance during routine operations and surveillance test activities was good. However, the inspectors noted inconsistent communication techniques used by operators during responses to annunciators. Other/NA

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7	12/19/1997	Positive	IR 97018 LER 97014	NRC	Operations	3A	The operations shift supervisor promptly evaluated operability of the automatic depressurization system when a discrepancy between the Updated Final Safety Analysis and as-built plant conditions was identified by the inspectors. The inspectors were following up on the identification by the licensee of improper logic circuit testing per NRC Generic Letter 96-01, "Testing of Safety-Related Logic Circuits."
8	11/25/1997	Positive	IR 97018 LER 97013 50.72 No. 33312	Self-Revealed	Operations	1B 3A	Operations personnel responded promptly and appropriately to a loss of both recombiner trains and subsequent manual reactor scram. The reactor was manually scrammed because a potential for a loss of condenser vacuum developed after the condenser offgas recombiner and offgas auxiliary systems tripped because of an electrical fault. Procedure adherence and informative briefings were observed. The shift manager maintained excellent command and control of activities during the event. The reactor startup was conducted without incident. Conservative Decision
9	11/24/1997	VIO/SL-IV	IR 97015	NRC	Operations	2A 3A	The licensee's program for protecting plant equipment from cold weather conditions appeared adequate. Plant personnel were instructed to consider environmental concerns when planning and performing maintenance in the winter months. However, the inspectors identified that the intent of the winter checklist procedure was changed due to an improper use of the temporary change process. The licensee subsequently identified that the procedure had not been approved in accordance with plant procedures. A subsequent operability determination indicated that the procedure change was not safety significant. A violation of 10 CFR 50, Appendix B, Criterion V, "Instructions, Procedures, and Drawings," was identified. The tracking number for this violation is No. 50-263/97015-01(DRP). Personnel Performance Deficiency
10	11/24/1997	Positive	IR 97015	NRC	Operations	3A 1A	The operators' performance during routine operations and surveillance activities were acceptable. With one exception, the operators were knowledgeable of plant conditions and equipment status. Operators' questioning attitude resulted in the identification of blockage in a motor cooling line and of a faulty bus insulator. Teamwork/Skill Level

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11	11/19/1997	Positive	IR 97016	NRC	Operations	3A	1C	Performance of the Control Room Simulator crew during the 1997 Emergency Preparedness exercise was effective. The Shift Manager and Shift Supervisor exercised proper command and control over the operators. The crew detected reactor events and took proper actions. Operator actions indicated a detailed understanding of plant events. Teamwork/Skill Level
12	10/30/1997	NCV	IR 97015	Self-Revealed	Operations	3A		An operator's failure to follow the electric fire pump startup procedure and inadequate instruction for shutting down the fire pump resulted in an automatic start of the diesel fire pump without operators' knowledge. Operations personnel appropriately declared the diesel fire pump inoperable after it operated for 1.5 hours without a minimum flow path. Subsequently, the pump was declared operable after a successful operability test was performed. The tracking number for this Non-Cited Violation is No. 50-2C3/97015-02(DRP). Inadequate Procedure/Instruction
13	10/13/1997	Positive	IR 97014	NRC	Operations	2A		The Division 1 and 2 batteries and the residual heat removal service water system were in good material condition. Involved Management
14	10/13/1997	Positive	IR 97014	NRC	Operations	3A	1A	Operator performance during routine operations and surveillance activities was good, and operations personnel conducted the planned reactor power decrease in a controlled manner. However, operations personnel were unaware that a condensate storage tank level instrument was inoperable during a maintenance activity. Teamwork/Skill Level

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Weakness	Overall Weak Licensee Performance

ID BY

Licensee	The licensed utility
NRC	The Nuclear Regulatory Commission
Self-Revealed	Identification by an event (e.g., equipment breakdown)
Other	Identification unknown

NOTES

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** URIs are unresolved items about which more information is required to determine whether the issue in question is an acceptable item, a deviation, a nonconformance, or a violation. However, the NRC has not reached its final conclusions on the issues, and the PIM entries may be modified when the final conclusions are made.

SENIOR MANAGEMENT MEETING CODES

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5	Problem Identification and Resolution: A - Identification B - Analysis C - Resolution

PLANT ISSUES MATRIX
Monticello

5/19/98

Search Sorted by Date (Descending) and SMM Codes (Ascending): Search Column = "SALP" ; SALP Area = "Maintenance" ; Beginning Date = "10/1/97" ; Ending Date = "3/31/98"

#	DATE	TYPE	SOURCE	ID BY	SALP	SMM CODES	DESCRIPTION
1	2/23/1998	Negative	IR 98002	NRC	Maintenance	2B	In general, maintenance staff's performance of observed work activities and surveillance tests was acceptable. However, maintenance personnel did not establish proactive foreign material exclusion control during the reactor building closed cooling water heat exchanger work activities.
2	1/21/1998	VIO/SL-IV	IR 98002	NRC	Maintenance	2B	The work order (WO) associated with moving channel crates to the refueling floor did not contain adequate instructions, which resulted in a heavy load movement without proper evaluation. One violation (of Criterion V of 10 CFR Part 50, Appendix B) for an inadequate procedure was identified. The tracking number for this violation is No. 50-263/98002-03(DRP).
3	1/12/1998	Positive	IR 97018	NRC	Maintenance	2A 3B	The material condition of equipment was acceptable. The operators interviewed were knowledgeable of degraded conditions, including an inoperable discharge canal radiation monitor and a difficult-to-move control rod drive. Teamwork/Skill Level
4	12/19/1997	LER	IR 97018 LER 97014	Licensee	Maintenance	5A 5C 3A	During a review of logic circuit surveillances per NRC Generic Letter 96-01, "Testing of Safety-Related Logic Circuits," licensee engineering personnel identified that the Division II automatic depressurization system inhibit switch was not being tested as required by Technical Specifications. This issue was a Non-Cited Violation and has tracking number No. 50-263/97018-02(DRP). Self-Critical
5	11/26/1997	NCV	IR 97018	NRC	Maintenance	2B 1B 5A	Inadequate planning of surveillance test and maintenance activities by planning, maintenance, and operations personnel resulted in a reactor scram signal for 13 control rods and a reactor water level transient. The licensee initiated a thorough investigation and implemented several corrective actions. This was treated as two examples of a Non-Cited Violation (of Criterion V of 10 CFR 50, Appendix B) for failure to follow a procedure and inadequate procedure. The tracking number for this issue is No. 50-263/97018-01(DRP). Inadequate Oversight
6	11/24/1997	Positive	IR 97015	NRC	Maintenance	2B 2A 3B	In general, work associated with selected maintenance and surveillance activities was conducted in an acceptable manner. Good foreign material exclusion controls were implemented for open systems; however, the return-to-service of the #14 residual heat removal service water pump was delayed due to workers' unfamiliarity with the seal water piping design. Involved Management

PLANT ISSUES MATRIX
Monticello

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#	DATE	TYPE	SOURCE	ID BY	SALP	SMM CODES	DESCRIPTION
7	10/13/1997	Positive	IR 97014	NRC	Maintenance	2B 3A	In general, the work associated with selected maintenance and surveillance activities was conducted in a professional and thorough manner. For work on the residual heat removal service water system, excellent foreign material exclusion control as well as careful work accomplished in accordance with procedures were observed during the replacement of surge check valves. Good oversight by quality control and engineering personnel was also provided. Involved Management
8	10/13/1997	Negative	IR 97014	NRC	Maintenance	2B 4B	The planning and approval of Work Order 9501551, "Repair Interior Coating of #12 CST [condensate storage tank]," was inadequate. The licensee initially failed to identify that operation with only one CST in service with the high pressure coolant injection (HPCI) and reactor core isolation cooling (RCIC) pump suction valves lined up to the CST was in violation of Technical Specifications. In addition, the licensee aborted a routine surveillance test when the suppression pool supply valves re-opened unexpectedly. An engineer did not fully evaluate plant conditions prior to requesting the initiation of the surveillance test. The tracking number for this Unresolved Item is No. 50-263/97014-03(DRP). Inadequate Procedure/Instruction
9	10/13/1997	Negative	IR 97014	NRC	Maintenance	2B 5C	The inspectors identified that the surveillance tests for HPCI and RCIC steam line isolation valves could lead to preconditioning of the valves prior to determining the "as found" valve closure times. The system engineer initiated the appropriate corrective action document to resolve the issue prior to performance of the next tests. Inadequate Procedure/Instruction

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PLANT ISSUES MATRIX
Monticello

5/19/98

Search Sorted by Date (Descending) and SMM Codes (Ascending): Search Column = "SALP" ; SALP Area = "Engineering" ; Beginning Date = "10/1/97" ; Ending Date = "3/31/98"

#	DATE	TYPE	SOURCE	ID BY	SALP	SMM CODES	DESCRIPTION
1	2/23/1998	VIO/SL-IV	IR 98002	NRC	Engineering	2B	The acceptance criterion in a Technical Specification (TS)-required surveillance test did not reflect the TS requirements. The licensee had several opportunities to identify the discrepancy, including during the biennial review of the surveillance test. Also, the inspectors identified a weakness in the licensee's TS change process, in that the process did not assure that changes to TSs are reflected in related procedures in a timely manner. The incorrect acceptance criterion in a required test was a violation of 10 CFR Part 50, Appendix B, Criterion V, "Instructions, Procedures, and Drawings." The tracking number for this violation is No. 50-263/98002-04(DRP).
2	2/23/1998	Positive	IR 98002	NRC	Engineering	3A	Training provided to station personnel for the inspection of new fuel bundles and channels was adequate. Engineering personnel performed a very detailed and meticulous inspection of the new fuel bundles and channels.
3	1/12/1998	URI	IR 97018	NRC	Engineering	4A 2B	The inspectors identified three discrepancies among the USAR (Updated Safety Analysis Report), a General Electric design specification, the plant configuration, and a surveillance test procedure for the HPCI (high pressure coolant injection) and the RCIC (reactor core isolation cooling) systems. The tracking number for this Unresolved Item is No. 50-263/97018-04(DRP).
4	1/12/1998	URI	IR 97018	NRC	Engineering	4C	The inspectors questioned whether the low pressure instruments for HPCI (high pressure coolant injection) and RCIC (reactor core isolation cooling) should be included in Technical Specification Table 3.2.1, "Instrumentation That Initiates Primary Containment Isolation Functions." The tracking number for this Unresolved Item is No. 50-263/97018-03(DRP).
5	12/19/1997	Positive	IR 97018 LER 97014	NRC	Engineering	4B	Engineering personnel responded appropriately by confirming operability and evaluating safety significance of licensee-identified problems with the adequacy of testing the Division II automatic depressurization system inhibit switch (identified during a review of logic circuit surveillances per NRC Generic Letter 96-01, "Testing of Safety-Related Logic Circuits").
6	11/6/1997	Positive	IR 97015	Licensee	Engineering	4B 3A	Engineering personnel demonstrated an excellent questioning attitude during review of setpoint calculations for the turbine stop valve limit switch actuations. Self-Critical

PLANT ISSUES MATRIX

5/19/98

Monticello

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7	10/17/1997	Positive	IR 97015	NRC	Engineering	4B 5C	Engineering personnel performed an acceptable operability determination for the residual heat removal service water pump motor cooler low flow condition. Comprehensive corrective actions were planned to address flow restrictions in the piping and to trend performance. Involved Management
8	10/13/1997	Positive	IR 97014	Licensee	Engineering	4B 3A 5A	An engineer demonstrated a good questioning attitude and identified that continued operation with one CST and the HPCI and RCIC suction valves aligned to the #11 CST was contrary to Technical Specifications. Self-Critical
9	10/13/1997	Positive	IR 97014 LER 97012	Licensee	Engineering	4C 3A 5C	Engineering personnel demonstrated a good questioning attitude in identifying a vortex concern and a non-conservative Technical Specification low level setpoint for the condensate storage tanks. The licensee initiated prompt corrective actions including establishing an administrative limit and implementing procedure changes. The tracking number for the Unresolved Item involving the non-conservative low level setpoint is No. 50-263/97014-03(DRP). Self-Critical
10	10/13/1997	Positive	IR 97014	NRC	Engineering	5A 4B	Engineering personnel responded appropriately to an industry event concerning potential effects of ventilation on tank levels. Self-Critical

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#	DATE	TYPE	SOURCE	ID BY	SALP	SMM CODES		DESCRIPTION
1	3/13/1998	Positive	IR 98005	NRC	Plant Support	1C		Security training records reviewed were complete and accurate.
2	3/13/1998	Negative	IR 98005	NRC	Plant Support	1C		Records reviewed were complete and accurate. Procedural guidance was for recovery actions after loss of some specific dual security systems was weak and upgraded during the inspection. A category of material exempted from search upon entry into the protected area was questionable.
3	3/13/1998	Negative	IR 98005	NRC	Plant Support	1C		The reliability of an important security system component has not improved since the previous inspection. Other security equipment observed functioned as designed and compensatory measures for equipment failure were seldom required. Maintenance support for security equipment appeared timely and effective.
4	3/13/1998	VIO/SL-IV	IR 98005	NRC	Plant Support	1C		A violation was noted pertaining to failure to adequately protect safeguards information on three occasions between October 9, 1997 and February 17, 1998. The procedures for compensatory measures for extended security computer outages will be upgraded. Protected area access control measures were effective.
5	3/13/1998	Positive	IR 98005	NRC	Plant Support	1C	3A	Security force performance has been consistent within the past year. Security force members were knowledgeable of post requirements. Security supervisors were active in providing good oversight of ongoing security activities. Plant personnel demonstrated a high level of awareness and compliance with security requirements during entry into the protected area.
6	3/13/1998	Positive	IR 98005	NRC	Plant Support	5C	1C	The self-assessment program within the security section was extensive, well monitored, and effective.
7	2/23/1998	Positive	IR 98002	NRC	Plant Support	1C		The inspectors noted that effective radiological controls were established for the new fuel inspections and that radiation protection technicians provided support during maintenance and surveillance activities.

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5/19/98

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#	DATE	TYPE	SOURCE	ID BY	SALP	SMM CODES	DESCRIPTION
8	2/13/1998	Positive	IR 98003	NRC	Plant Support	1C	The RP staff properly implemented the 10 CFR Part 61 waste characterization program. The staff sampled waste streams and evaluated the results of the analyses in accordance with procedures. The inspectors identified some minor problems concerning the lack of independence of the review of 1996 data.
9	2/13/1998	Positive	IR 98003	NRC	Plant Support	1C	Plant housekeeping was effective in maintaining areas free of unnecessary equipment and debris. Radiological posting and labeling in the plant was appropriate, and station efforts to reduce instances of unsecured items crossing contaminated area boundaries were effective. One concern was noted, however, with the plant's survey maps at access control not being updated in a timely manner.
10	2/13/1998	Positive	IR 98003	NRC	Plant Support	1C	The radioactive waste processing systems, storage areas, control room, and radioactive materials storage warehouses were well-organized, and waste containers were properly sealed and labeled.
11	2/13/1998	Misc	IR 98003	NRC	Plant Support	1C	The licensee's radioactive waste program procedures were acceptable.
12	2/13/1998	Positive	IR 98003	NRC	Plant Support	2A 5B 5C	The station has had intermittent operability problems over the past several years with the discharge canal radiation monitor sample pump. Licensee staff had progressed logically and expeditiously to determine the cause of the pump failures during this time period and had attempted numerous types of corrective actions without success. At the time of this inspection, plant engineering staff developed a reasonable course of corrective actions to eliminate the problem.
13	2/13/1998	Misc	IR 98003	NRC	Plant Support	2B	The licensee performed periodic inspections of filtration systems associated with the Radwaste Building exhaust and the Reactor Water Cleanup (RWCU) system room exhaust. As the licensee was not committed to Regulatory Guide 1.140, the testing methodology was not fully consistent with the guidance contained in the regulatory guide; however, the testing was adequate.
14	2/13/1998	Positive	IR 98003	NRC	Plant Support	3B 1C	The licensee's training program for plant workers handling radioactive wastes or materials was effective and current. Staff had appropriate educational credentials, experience, and qualifications to perform their assigned tasks.

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#	DATE	TYPE	SOURCE	ID BY	SALP	SMM CODES	DESCRIPTION
15	1/12/1998	Positive	IR 97018	NRC	Plant Support	3A 1C	The fire brigade responded promptly and appropriately during two routine fire drills. Good interaction between the fire brigade leader and the security force was evident. Teamwork/Skill Level
16	11/21/1997	Negative	IR 97017	NRC	Plant Support	1C	Several instances of unsecured items crossing contaminated area boundaries indicated that additional attention was needed in the area of contamination control practices. These instances appeared to reflect an ongoing problem at the plant with items crossing contaminated area boundaries. Inadequate Oversight
17	11/21/1997	Positive	IR 97017	NRC	Plant Support	1C	Respiratory protection equipment was well-maintained and properly stored. Involved Management
18	11/21/1997	Positive	IR 97017	NRC	Plant Support	1C	Plant personnel requiring respirator use in the performance of their regular or emergency duties were properly qualified. However, one issue regarding the wearing of beards by control room personnel will be reviewed during a future inspection. Other/NA
19	11/21/1997	Positive	IR 97017	NRC	Plant Support	1C	In general, radiological posting and labeling in the plant was appropriate, though one instance of possible overposting was identified in the upper torus area. Involved Management
20	11/21/1997	Positive	IR 97017	NRC	Plant Support	1C 3B	The accessible radioactive waste storage areas, processing areas, and tank rooms were organized, and waste containers were, for the most part, properly sealed and labeled. Radwaste personnel were very knowledgeable about radwaste systems, operations, and procedures. Involved Management
21	11/21/1997	Positive	IR 97017	NRC	Plant Support	1C 3C	The plant's new survey maps were very effective at informing radiation workers of plant radiological hazards and conditions. Involved Management
22	11/21/1997	Negative	IR 97017	NRC	Plant Support	1C 3C 3B	The respiratory protection program was generally well-implemented; however, administrative oversight of the program was minimal. Specifically, the radiation protection supervisor responsible for oversight of the program was tasked with many other responsibilities and, as a result, was unfamiliar with various aspects of the program. Inadequate Oversight

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#	DATE	TYPE	SOURCE	ID BY	SALP	SMM CODES	DESCRIPTION
23	11/21/1997	Positive	IR 97017	NRC	Plant Support	2A 1C	Plant housekeeping was effective in maintaining areas free of debris, and the station's effort to reduce the total contaminated area within the plant was successful. Involved Management
24	11/19/1997	Positive	IR 97016	NRC	Plant Support	1C	The scenario for the 1997 Emergency Preparedness exercise was adequately challenging, and specifically designed to provide greater realism. Overall exercise control was very good. Involved Management
25	11/19/1997	Positive	IR 97016	NRC	Plant Support	3A 1C	Overall performance during the 1997 Emergency Preparedness exercise demonstrated that onsite emergency plan implementation was adequate. Licensee personnel demonstrated their ability to implement the plan by correctly classifying scenario emergencies, notifying offsite agencies of the classified events, activating emergency facilities, providing protective action recommendations when warranted, and taking accident mitigation actions. Interfacility transfers of command and control of event response were orderly and timely. Teamwork/Skill Level
26	11/19/1997	Positive	IR 97016	NRC	Plant Support	3A 1C	Overall, the Technical Support Center staff's performance during the 1997 Emergency Preparedness exercise was very effective. The Emergency Director made effective use of his staff to provide information, projections, and suggestions. He evaluated their inputs and then made rapid decisions which were then plainly communicated. Mitigation activities were well-prioritized. Involved Management
27	11/19/1997	Positive	IR 97016	NRC	Plant Support	3A 1C	Performances of Operational Support Center management and staff during the 1997 Emergency Preparedness exercise were very competent. Involved Management
28	11/19/1997	Positive	IR 97016	NRC	Plant Support	3A 1C	The overall performance in the Emergency Operations Facility during the 1997 Emergency Preparedness exercise was good. However, the lack of sufficient criteria for identifying the loss of the fuel clad fission product barrier in the Emergency Classification Guidelines led to a brief delay in declaring a General Emergency. Involved Management

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#	DATE	TYPE	SOURCE	ID BY	SALP	SMM CODES	DESCRIPTION
29	11/19/1997	Positive	IR 97016	NRC	Plant Support	3A 1C	Field monitoring teams performed well, and exhibited efficient and timely checking and loading of equipment, as well as proficient operation and knowledge of their survey instruments. The teams generally accomplished their assigned tasks; however, some were slowed down considerably by problems with communications, equipment handling, and maps. Teamwork/Skill Level
30	11/19/1997	Positive	IR 97016	NRC	Plant Support	3A 1C	Performance demonstrated during the Recovery phase of incident response (during the 1997 Emergency Preparedness exercise), demonstrated after a scenario "time jump," was acceptable. Recovery discussions observed in the Technical Support Center and Emergency Operations Facility were adequate. Involved Management
31	11/19/1997	Positive	IR 97016	NRC	Plant Support	3A 1C	Event classifications for the 1997 Emergency Preparedness exercise were correct. However, there was a delayed declaration of the General Emergency. Offsite notifications and offsite protective action recommendations were correct and timely. Inplant activities were well thought out and well coordinated. Transfers of command and control were appropriately coordinated. Involved Management
32	11/19/1997	Positive	IR 97016	NRC	Plant Support	5A 1C	The licensee's self-assessment of the 1997 Emergency Preparedness exercise was comprehensive and closely mirrored the NRC evaluation team's conclusions. Self-Critical
33	10/28/1997	VIO/SL-IV	IR 98003	NRC	Plant Support	5C	A violation of 10 CFR 20.1502 was identified for a failure to perform an adequate radiological evaluation of a job prior to the start of work. This violation resulted from an incident on October 28, 1997, in which an instruments and controls specialist became contaminated with radioactive resin during calibration of a pressure indicator on a radioactive waste (radwaste) system. The plant staff's immediate response to the event and the personnel dose assessment were executed properly. However, the licensee's subsequent corrective actions were not comprehensive. The tracking number for this Severity Level IV violation is No. 50-263/98003-01(DRS).
34	10/23/1997	Negative	IR 97201	NRC	Plant Support	1C	During an OSRE (Operational Safeguards Response Evaluation), the NRC identified that the issue and control of security keys required improvement to enhance ingress and egress during emergency conditions.

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35	10/23/1997	Negative	IR 97201	NRC	Plant Support	1C	During an OSRE (Operational Safeguards Response Evaluation), the NRC identified that the licensee should verify the effectiveness of a designated security defensive position.
36	10/13/1997	Positive	IR 97014	NRC	Plant Support	3A 1C 5A	During two security training drills, good communication was established between field and alarm station personnel, and the guard force responded appropriately to the "events." The licensee's critiques of the drills were informative and self-critical. Teamwork/Skill Level

GENERAL DESCRIPTION OF PIM TABLE LABELS

#	A counter number used for NRC internal editing.
DATE	The date of the event or significant issue. For those items that have a clear date of occurrence use the actual date. If the actual date is not known, use the date the issue was identified. For issues that do not have an actual date or a date of identification, use the LER or inspection report date.
TYPE	The categorization of the issue - see the TYPE ITEM CODE table.
SOURCE	The document that contains the issue information: IR for NRC Inspection Report or LER for Licensee Event Report.
ID BY	Identification of who discovered the issue - see table.
SALP	SALP Functional Area Codes - Engineering, Maintenance, Operations, Plant Support and All/Multiple (i.e., more than one SALP area affected).
SMM CODES	Senior Manager Meeting Codes - see table.
DESCRIPTION	Details of the issue from the LER text or from the IR Executive Summaries.

TYPE ITEM CODE

DEV	Deviation from NRC Requirements
ED	Escalated Discretion - No Civil Penalty
EEI*	Escalated Enforcement Issue - Waiting Final NRC Action
LER	License Event Report to the NRC
Licensing	Licensing Issue from NRR
Misc	Miscellaneous (Emergency Preparedness Finding, etc.)
NCV	Non-Cited Violation
Negative	Individual Poor Licensee Performance
Positive	Individual Good Licensee Performance
Strength	Overall Strong Licensee Performance
URI**	Unresolved Inspection Item
VIO/SL-I	Notice of Violation - Severity Level I
VIO/SL-II	Notice of Violation - Severity Level II
VIO/SL-III	Notice of Violation - Severity Level III
VIO/SL-IV	Notice of Violation - Severity Level IV
Weakness	Overall Weak Licensee Performance

ID BY

Licensee	The licensed utility
NRC	The Nuclear Regulatory Commission
Self-Revealed	Identification by an event (e.g., equipment breakdown)
Other	Identification unknown

NOTES

* EEIs are apparent violations of NRC requirements that are being considered for escalated enforcement action in accordance with the "General Statement of Policy and Procedure for NRC Enforcement Action" (Enforcement Policy), NUREG-1600. However, the NRC has not reached its final enforcement decision on the issues identified by the EEIs and the PIM entries may be modified when the final decisions are made. Before the NRC makes its enforcement decision, the licensee will be provided with an opportunity to either (1) respond to the apparent violation or (2) request a predecisional enforcement conference.

** URIs are unresolved items about which more information is required to determine whether the issue in question is an acceptable item, a deviation, a nonconformance, or a violation. However, the NRC has not reached its final conclusions on the issues, and the PIM entries may be modified when the final conclusions are made.

SENIOR MANAGEMENT MEETING CODES

1	Operational Performance: A - Normal B - During Transients C - Programs and Processes
2	Material Condition: A - Equipment Condition B - Programs and Processes
3	Human Performance: A - Work Performance B - Knowledge, Skills, and Abilities C - Work Environment
4	Engineering/Design: A - Design B - Engineering Support C - Programs and Processes
5	Problem Identification and Resolution: A - Identification B - Analysis C - Resolution

ENCLOSURE 2

MONTICELLO
INSPECTION PLAN

INSPECTION PROCEDURE	TITLE/ PROGRAM AREA	NUMBER OF INSPECTORS	PLANNED INSPECTION DATES	TYPE OF INSPECTION - COMMENTS
IP 73753	ISI/Maintenance	1	3/23-4/23/98 COMPLETE	Inspection of inservice inspection program (CI)
IP 83750	Radiation Protection/Plant Support	1	4/6-10/98 COMPLETE	Radiation Protection (CI)
IP 82701	Emergency Preparedness/ Plant Support	1	4/27-5/1/98 COMPLETE	Emergency Preparedness Program (CI)
IPs 37550, 40500	Engineering	4	5/11-29/98 ONGOING	Engineering and Technical Support; Corrective Actions (CI)
IP 84750	Radiation Protection/ Plant Support	1	8/10-14/98	Liquid and Gaseous Effluents Inspection (CI)
IP 62706	Maintenance Rule/ Maintenance	1-2	8/11-13/98	Maintenance Rule Followup Inspection (RI)
IP 83750	Radiation Protection	1	11/16-23/98	Radiation Protection (CI)

IP - Inspection Procedure

CI - Core Inspection - Minimum NRC Program (mandatory all plants)

RI - Regional Initiative