



UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION III
801 WARRENVILLE ROAD
LISLE, ILLINOIS 60532-4351

May 27, 1998

Mr. John K. Wood
Vice President - Nuclear
Davis-Besse Nuclear Power Station
Centerior Service Company
5501 North State Route 2
Oak Harbor, OH 43449

SUBJECT: PLANT PERFORMANCE REVIEW (PPR) - DAVIS-BESSE

Dear Mr. Wood:

The NRC staff recently completed the semiannual Plant Performance Review (PPR) of the Davis-Besse Nuclear Power Plant. The staff conducts these reviews for all operating nuclear power plants to develop an integrated understanding of safety performance. The results are used by NRC management to facilitate planning and allocation of inspection resources. The PPR for Davis-Besse involved the participation of all technical divisions in evaluating inspection results and safety performance information for the period of October 1997 through March 1998. PPRs provide NRC management with a current summary of licensee performance and serve as inputs to the NRC Systematic Assessment of Licensee Performance (SALP) and senior management meeting (SMM) reviews.

Enclosure 1 contains a historical listing of plant issues, referred to as the Plant Issues Matrix (PIM), that were considered during this PPR process to arrive at an integrated view of licensee performance trends. The PIM includes only items from inspection reports or other docketed correspondence between the NRC and the Centerior Service Company. The PPR may also have considered some predecisional and draft material that does not appear in the attached PIM, including observations from events and inspections that had occurred since the last NRC inspection report was issued, but had not yet received full review and consideration. This material will be placed in the PDR as part of the normal issuance of NRC inspection reports and other correspondence.

This letter advises you of our planned inspection effort resulting from the Davis-Besse PPR process. It is provided to minimize the resource impact on your staff and to allow for scheduling conflicts and personnel availability to be resolved in advance of inspector arrival onsite. Enclosure 2 details our inspection plan for the next 6 months. The rationale or basis for each inspection outside the core inspection program is provided so that you are aware of the reason for emphasis in these program areas. Resident inspections are not listed due to their ongoing and continuous nature.

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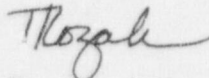
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J. Wood

-2-

We will inform you of any changes to the inspection plan. If you have any questions, please contact me at 630/829-9866.

Sincerely,



Thomas J. Kozak, Chief
Reactor Projects Branch 4

Docket No.: 50-346
License No.: NPF-3

Enclosures: 1. Plant Issues Matrix
2. Inspection Plan

cc w/encls: John P. Stetz, Senior
Vice President - Nuclear
J. H. Lash, Plant Manager
J. L. Freels, Manager
Regulatory Affairs
State Liaison Officer, State
of Ohio
Robert E. Owen, Ohio
Department of Health
C. A. Glazer, State of Ohio,
Public Utilities Commission

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/s/ T. Kozak

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Enclosure 1

Davis-Besse Nuclear Power Station

Plant Issues Matrix

October 1997 through March 1998

PLANT ISSUES MATRIX

Davis-Besse

5/29/98

Search Sorted by Date (Descending) and SMM Codes (Ascending): Search Column = "Date" ; Beginning Date = "10/1/97" ; Ending Date = "3/31/98"

#	DATE	TYPE	SOURCE	ID BY	SALP	SMM CODES	DESCRIPTION
1	3/31/1998	Positive	IR 98004	Licensee	Plant Support	1C	The licensee continued maintaining proper control of radiological areas and proper control of personnel entering and exiting these areas.
2	3/31/1998	Positive	IR 98004	NRC	Plant Support	1C 5A	The central and secondary alarm stations were staffed with attentive and knowledgeable personnel. The licensee appropriately identified and replaced degraded equipment with temporary equipment before the end of the inspection period.
3	3/31/1998	Negative	IR 98004	NRC	Engineering	4B	Weak engineering support to maintenance activities led to the inadvertent breach of the emergency ventilation system negative pressure boundary when workers drilled through a drain pipe imbedded in the decay heat removal heat exchanger room. Although no spread of contamination occurred, breaching the boundary exposed the workers to a potentially contaminated system. Clear guidance on the necessary drawings for the engineers to review before approving the drilling location was not readily available to the evaluating engineer due to this being an infrequently performed evolution. The plant response to the event was timely and adequate. The proposed corrective actions were acceptable.
4	3/31/1998	Positive	IR 98004	NRC	Engineering	5A	The inspectors observed activities of the licensee's offsite review committee. The inspectors concluded that the offsite review committee members effectively communicated concerns and suggestions with plant management.
5	3/31/1998	Positive	IR 98004	Licensee	Plant Support	5A 5C	Security management issued a memorandum to heighten security officer awareness following recent lapses in security guard attentiveness to their duties. These expectations were discussed during shift security turnovers. Security guard attentiveness improved subsequent to this action.
6	3/31/1998	Positive	IR 98004	NRC	Engineering	5B	The inspectors concluded that engineering personnel performed a thorough and detailed review of the concerns raised by Information Notice 97-90, "Use of Non-conservative Acceptance Criteria In Safety-Related Pump Surveillance Tests"
7	3/31/1998	Positive	IR 98004	Licensee	Engineering	5C	The licensee was adequately addressing NRC and self-identified Updated Safety Analysis Report discrepancies.

PLANT ISSUES MATRIX

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#	DATE	TYPE	SOURCE	ID BY	SALP	SMM CODES	DESCRIPTION
8	3/27/1998	Positive	IR 98004	NRC	Operations	1A	Implementation of the new work support center has significantly reduced distractions in the control room. Further, the administrative workload of the control room senior reactor operator has been reduced, allowing him to devote more of his attention towards supervising control room and shift personnel and towards observing the performance of plant equipment.
9	3/27/1998	Positive	IR 98004	NRC	Operations	1C	Overall, observed simulator training was pertinent and effective towards providing assurance that operators were proficient to shut down the plant in an orderly manner.
10	3/27/1998	Positive	IR 98004	NRC	Operations	5A	The inspectors reviewed a Quality Assessment Audit Report that evaluated the effectiveness of control room activities. The inspectors concluded that the findings were generally consistent with observations made by the inspectors and that it had good scope and depth.
11	3/19/1998	Licensing	98NRRPMP	NRC	Engineering	4C	Licensing submittals by the licensees continue to be excellent. The licensing bases are thoroughly reviewed, the technical issues are addressed in-depth, and safety issues are appropriately dispositioned.
12	3/19/1998	Licensing	98NRRPMP	NRC	Engineering	4C 3A	Communications between Davis-Besse licensing staff and NRR continue to be very effective. Some schedules slip on non-safety-significant issues due to the licensees' workload, though all safety issues are handled promptly.
13	2/26/1998	Positive	IR 98003	NRC	Engineering	4A	Engineering was proactive in reviewing inspection findings at other nuclear sites for applicability to Davis-Besse and in reviewing original licensing review questions and answers to verify appropriate incorporation in the Updated Safety Analysis Report.
14	2/26/1998	Misc	IR 98003	NRC	Engineering	5B	Engineering judgement proved to be correct concerning the structural adequacy of mounting brackets for the borated water storage tank level instrument.
15	2/26/1998	Negative	IR 98003	NRC	Engineering	5B	There was a lack of significant progress on environmental qualification of support equipment in the emergency core cooling system rooms
16	2/26/1998	Positive	IR 98003	NRC	Engineering	5B	Timely and appropriate engineering evaluation of the seismic concern, for non-safety pressure gages tied to safety systems, facilitated prompt closure of this item.

PLANT ISSUES MATRIX

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#	DATE	TYPE	SOURCE	ID BY	SALP	SMM CODES	DESCRIPTION
17	2/18/1998	Positive	IR 98002	NRC	Operations	1A 5A	The inspectors observed that the plant was operated in a controlled, conservative manner. Plant issues that were identified were appropriately handled in accordance with the licensee's corrective action program or material deficiency program. Operators were knowledgeable of Technical Specification (TS) operability requirements and tagouts were implemented in accordance with the tagout procedure. Generally, the operators exhibited good knowledge of plant equipment status and properly used plant operating procedures. Control room operators and equipment operators were observed to be adequately fulfilling their duties (Sections O1.1 and O4.1).
18	2/18/1998	Positive	IR 98002	NRC	Plant Support	1C 3A	Overall, radiation protection activities relating to a diving activity in the spent fuel pool transfer canal and a high integrity container lift were performed in a professional, well planned manner (Section R1.1).
19	2/18/1998	Positive	IR 98002	NRC	Plant Support	1C 3B	The fire brigade team effectively responded to a challenging drill scenario (Section F4.1).
20	2/18/1998	Positive	IR 98002	NRC	Maintenance	2A 3A	During surveillance activities, equipment was observed to perform as described by the Updated Safety Analysis Report. Maintenance personnel communicated adequately and adhered to procedure requirements while performing maintenance and surveillance activities. The inspectors observed that oversight of maintenance activities was effective. Maintenance and surveillance testing activities were professionally conducted (Section M1.1).
21	2/18/1998	Misc	IR 98002	NRC	Operations	3A	The operations manager appropriately communicated physical fitness expectations to the operating crews after being notified by the inspectors that a shift supervisor stood watch with the medical condition of laryngitis (Section O6.1).
22	2/18/1998	VIO/SL-IV	IR 98002	Self-Revealed	Operations	3A	Plant operations were impacted on two occasions when operators displayed a lack of attention to detail. The first example involved the inadvertent isolation of seal injection to all four reactor coolant pump seal packages due to an operator error. The second example involved the lifting of a letdown system relief valve due to an operator not performing a procedure in the correct sequence. These events are two examples of a violation of TS 6.8.1 (Section O1.2).

PLANT ISSUES MATRIX

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#	DATE	TYPE	SOURCE	ID BY	SALP	SMM CODES	DESCRIPTION
23	2/18/1998	VIO/SL-IV	IR 98002	NRC	Operations	4C 3B	The inspectors identified one violation of 10 CFR 50.72(b)(1)(ii)(B) where the licensee did not report to the NRC, within one hour of the discovery, that speed sensing circuitry for Emergency Diesel Generator #1 was not designed per the 10 CFR Part 50, Appendix R, design criteria for hot short protection, a condition outside the design basis of the plant (Section F8.1).
24	2/18/1998	Positive	IR 98002	Licensee	Engineering	5A 5C	The licensee demonstrated an excellent questioning attitude regarding the configuration of a refueling drain canal valve, a valve whose open position ensured sufficient water supply to emergency core cooling system pump suction during an accident. After determining that the valve was not in its locked valve program, the licensee immediately entered containment, verified the valve was in the open position, locked the valve, and plans to enter it in the locked valve program (Section E8.2).
25	1/28/1998	Positive	IR 97015	NRC	Operations	2A	Valve line-ups and major flow paths for both engineered safety features and important-to-safety systems were verified to be consistent with plant procedures/drawings and the Updated Safety Analysis Report (USAR). Other/NA
26	1/28/1998	Positive	IR 97015	NRC	Operations	2A 1B	The inspectors observed that the operators maintained good control of the plant during a failure of the servo control valve for Turbine Control Valve No. 4. Teamwork/Skill Level
27	1/28/1998	Positive	IR 97015	NRC	Maintenance	2B	The licensee removed from service the level controller for both the Train 1 Auxiliary Feed Pump and the Motor Driven Feed Pump for a maintenance outage. The inspectors determined that operations work control personnel were knowledgeable of the maintenance rule requirements associated with this work, and that risk was appropriately considered in the planning stages. Teamwork/Skill Level
28	1/28/1998	Negative	IR 97015	NRC	Operations	3A	A reactor operator missed noticing that the computer display for Group 38, which indicates reactor core nuclear parameters and calculates secondary heat balance power, did not update for a 50-minute period because he became distracted with a problem on the Nuclear Operations Management System. Personnel Performance Deficiency

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29	1/28/1998	Positive	IR 97015	NRC	Operations	3A 1A	Operations activities were conducted in a controlled, conservative manner. Shift briefs were thorough, operators had good knowledge of plant status and activities, and procedures were consistently complied with. Plant management was aware of and responded to operational issues in an appropriate manner. Involved Management
30	1/28/1998	Positive	IR 97015	NRC	Plant Support	3A 1C	A thorough ALARA briefing was conducted for a containment entry to perform reactor coolant pump upper thrust bearing resistance temperature detector circuit modifications. General area radiation dose rate estimates closely matched those actually found in containment. Radiation protection personnel provided excellent assistance and support for the entry team. Total dose received for the entry was low at about 10 millirem. Teamwork/Skill Level
31	1/28/1998	Positive	IR 970*5	NRC	Engineering	3A 3B	Once the turbine control valve servo control valve failure occurred, the plant engineering organization responded well.
32	1/28/1998	Positive	IR 97015	NRC	Plant Support	3A 5A	The licensee's response to smoke from a faulted electrical generator for the elevator machinery room in the radiological restricted area (RRA) was done in accordance with plant procedures. The operators appropriately staffed the brigade, donned proper fire protection equipment and established good communication with control room personnel. Following the event, the licensee appropriately initiated potential condition adverse to quality reports (PCQRs) to document concerns regarding difficulty of personnel exiting the RRA due to smoke. Teamwork/Skill Level
33	1/28/1998	Positive	IR 97015	NRC	Engineering	3B	Station Review Board members displayed good technical knowledge of the subject matter presented to them. Members solicited feedback from the sponsors of the documentation that was reviewed. The members requested additional information be provided when clarification or explanation was needed. Teamwork/Skill Level
34	1/28/1998	Negative	IR 97015	NRC	Maintenance	5A 2B	The inspectors found that surveillance procedures for smoke detectors installed in ventilation ducts were not written in accordance with the vendor's technical manual recommendations. Although the detectors were not required by 10 CFR Part 50, Appendix R, the licensee failed to meet management's expectations because a PCQR was not initially written to document this inconsistency until prompted by the inspectors. Inadequate Procedure/Instruction

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35	1/28/1998	Negative	IR 97015	NRC	Engineering	5B	Plant engineering personnel missed an opportunity to predict a turbine control valve servo control valve failure. They were not aggressive in responding to a 30 megawatt load swing that had occurred the week before. Further, they did not gather and assess available information regarding the slow response of the turbine control valve found during previous testing. Consequently, the licensee reacted to the issue rather than managed it. Other/NA
36	12/17/1997	LER	LER 97016	Licensee	Operations	2B	The refueling canal drain valve to the reactor cavity sump (DH92) was not locked open per the requirements of the procedure, Operation and Control of Locked Valves, to exclude it from the surveillance requirement for Emergency Core Cooling System and Containment Spray System flow path valves. The bypass valves (DH10 and DH26) for the reactor coolant system to decay heat pump suction isolation valves were not included in the surveillance procedure to verify the ECCS flow path. Personnel Performance Deficiency
37	12/10/1997	LER	LER 97015	Licensee	Engineering	4A	A postulated hot short condition could cause a failure of the Emergency Diesel Generator (EDG) speed switch before the operator could manually disconnect the EDG tachometer control room circuitry from the EDG speed switch. Engineering/Design Deficiency
38	11/10/1997	Positive	IR 97013	NRC	Plant Support	1C	Radiological conditions were properly communicated to plant personnel through postings, barriers and signs. Actual radiation conditions were verified to be consistent with radiation area postings. Teamwork/Skill Level
39	11/10/1997	Positive	IR 97013	NRC	Plant Support	1C	Security personnel were observed to be performing their duties and access control equipment was observed to be operating in accordance with regulatory requirements. Teamwork/Skill Level
40	11/10/1997	Positive	IR 97013	NRC	Operations	2A **	Important-to-safety system lineups and major flowpaths were verified to be in conformance with plant procedures/drawings and the Updated Safety Analysis Report. Equipment material condition was excellent in all cases. Teamwork/Skill Level

PLANT ISSUES MATRIX

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41	11/10/1997	Positive	IR 97013	NRC	Operations	2A 3A	The excellent material condition of the plant resulted in minimal operator challenges. Operators provided immediate response to plant annunciators and exhibited good adherence to procedures. On-shift communications were good. Tagouts provided adequate protection of equipment and personnel during maintenance activities. Involved Management
42	11/10/1997	Negative	IR 97013	NRC	Maintenance	2B	The inspector noted that the implementing procedure for a technical specification surveillance test did not require that test data be recorded. This lack of documented test data prevented supervisory personnel from having the opportunity to verify that the acceptance criteria had been met. Inadequate Procedure/Instruction
43	11/10/1997	Positive	IR 97013	NRC	Maintenance	3A 2B	Good communications between several departments contributed in minimizing the #2 Emergency Diesel Generator unavailability time during a routine maintenance outage. Diesel load swings observed during post maintenance surveillance testing were corrected in a timely manner. Appropriate housekeeping, foreign material exclusion, and fire protection measures were observed. Surveillance activities were performed in conformance with written instructions, and surveillance results satisfied regulatory requirements. Teamwork/Skill Level
44	11/10/1997	Positive	IR 97013	NRC	Engineering	4B	Operability recommendations were technically sound and consistent with regulatory requirements. Plant engineering personnel pursued corrective actions relating to degraded material conditions that affected the operability of plant components in a timely manner. Teamwork/Skill Level
45	11/10/1997	Positive	IR 97013	NRC	Operations	5B	Management Review Committee members effectively administered the initial categorization and assignment of Potential Condition Adverse to Quality Reports. Involved Management
46	10/10/1997	Positive	IR 97012	NRC	Plant Support	1C	The chemistry laboratory quality control program was effective, and ensured that the licensee could obtain accurate chemical and radiochemical analyses. (Section R7.1)

PLANT ISSUES MATRIX

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47	10/10/1997	Positive	IR 97012	NRC	Plant Support	1C	The post accident sampling system (PASS) was effectively maintained as required by technical specifications. Material condition and maintenance history was tracked, individuals were trained, and the PASS samples showed good agreement with reactor coolant system samples. One inspection follow-up item was opened to evaluate the long term corrective actions for a PASS configuration which resulted in an alarm of the safety features actuation system radiation monitors. (Section R2.1)
48	10/10/1997	Positive	IR 97012	NRC	Plant Support	1C	There was no discernable impact on the environment from plant operations. Specific aspects of the radiological environmental monitoring program, including material condition of air sampling equipment, and sample collection were appropriately implemented. (Section R1.3)
49	10/10/1997	Positive	IR 97012	NRC	Plant Support	1C	The water chemistry of primary and secondary systems was well maintained and monitored. Levels of corrosive impurities were maintained below industry guidelines. (Section R1.2)
50	10/10/1997	Positive	IR 97012	NRC	Plant Support	5C **	The corrective actions implemented to address a violation regarding the failure to appropriately post a radiation area were effective in preventing recurrence. (Section R8.3)
51	10/2/1997	LER	LER 97013	Licensee	Maintenance	2B	The calibration test for the Safety Features Actuation System Channel 2 containment radiation instrument string was partially performed on June 6, 1996. Upon completion of the test, the Test Cover Sheet was erroneously marked and the schedule was updated to indicate the test was performed to meet the surveillance test schedule, even though the test was not performed in its entirety. Personnel Performance Deficiency
52	10/1/1997	Negative	IR 97009	NRC	Plant Support	1A	The inspectors were concerned that, given the normal duties of the assistant shift supervisor, the management decision to designate the control room assistant shift supervisor as the fire watch brigade captain, could result in delays in manning the fire brigade (Section F8.1). Inadequate Oversight

PLANT ISSUES MATRIX

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53	10/1/1997	VIO/SL-IV	IR 97009	NRC	Operations	1C	The "Conduct of Operations" procedure granted the shift supervisor unilateral authority to allow procedural steps to be performed out-of-order. The NRC identified that this authority did not conform to the requirements of Technical Specification 6.5.3.1.b in that for a procedure change, a second review and approval by another member of plant management staff was not specified in the procedure (Section O8.1). Inadequate Procedure/Instruction
54	10/1/1997	Positive	IR 97009	NRC	Plant Support	1C	Plant personnel exhibited satisfactory adherence to radiation protection program requirements (Section R1). Teamwork/Skill Level
55	10/1/1997	Positive	IR 97009	NRC	Operations	2A	Material condition of engineered safety features and important-to-safety plant systems was excellent (Section O2.1). *****
56	10/1/1997	Negative	IR 97009	NRC	Maintenance	2B	Leakage integrity of the Component Cooling Water system was not verified nor were applicable acceptance criteria established for leakage between the essential and non-essential portions of the system. At the conclusion of the inspection period, engineering personnel were evaluating the need for additional integrated system leakage rate testing (Section E3.1). Inadequate Oversight
57	10/1/1997	Positive	IR 97009	NRC	Maintenance	2B	Maintenance and surveillance activities observed or otherwise reviewed during the inspection period were conducted in accordance with plant procedures and applicable regulatory requirements (Section M1.1). Teamwork/Skill Level
58	10/1/1997	Negative	IR 97009	NRC	Operations	3A **	Communications and documentation associated with a Limiting Conditions for Operation action that was required in response to an inoperable Reactor Protection System channel was inadequate (Section O1.2). Personnel Performance Deficiency
59	10/1/1997	VIO/SL-IV	IR 97009	NRC	Engineering	5C	Inadequate interim corrective actions were established to address a postulated worst case Circulating Water system linebreak. Although an analysis indicated that operators would have insufficient time to respond to the break and subsequent flooding conditions, no actions were taken to provide additional guidance for operator response or to extend the required operator response time. This situation was determined to be a violation of 10 CFR Part 50, Appendix B, Criteria XVI, "Corrective Actions" (Section E8.4). Inadequate Oversight

GENERAL DESCRIPTION OF PIM TABLE LABELS

#	A counter number used for NRC internal editing.
DATE	The date of the event or significant issue. For those items that have a clear date of occurrence use the actual date. If the actual date is not known, use the date the issue was identified. For issues that do not have an actual date or a date of identification, use the LER or inspection report date.
TYPE	The categorization of the issue - see the TYPE ITEM CODE table.
SOURCE	The document that contains the issue information: IR for NRC Inspection Report or LER for Licensee Event Report.
ID BY	Identification of who discovered the issue - see table.
SALP	SALP Functional Area Codes - Engineering, Maintenance, Operations, Plant Support and All/Multiple (i.e., more than one SALP area affected).
SMM CODES	Senior Manager Meeting Codes - see table.
DESCRIPTION	Details of the issue from the LER text or from the IR Executive Summaries.

TYPE ITEM CODE

DEV	Deviation from NRC Requirements
ED	Escalated Discretion - No Civil Penalty
EEI*	Escalated Enforcement Issue - Waiting Final NRC Action
LER	License Event Report to the NRC
Licensing	Licensing Issue from NRC
Misc	Miscellaneous (Emergency Preparedness Finding, etc.)
NCV	Non-Cited Violation
Negative	Individual Poor Licensee Performance
Positive	Individual Good Licensee Performance
Strength	Overall Strong Licensee Performance
URI**	Unresolved Inspection Item
VIO/SL-I	Notice of Violation - Severity Level I
VIO/SL-II	Notice of Violation - Severity Level II
VIO/SL-III	Notice of Violation - Severity Level III
VIO/SL-IV	Notice of Violation - Severity Level IV
Weakness	Overall Weak Licensee Performance

ID BY

Licensee	The licensed utility
NRC	The Nuclear Regulatory Commission
Self-Revealed	Identification by an event (e.g., equipment breakdown)
Other	Identification unknown

NOTES

* EEIs are apparent violations of NRC requirements that are being considered for escalated enforcement action in accordance with the "General Statement of Policy and Procedure for NRC Enforcement Action" (Enforcement Policy), NUREG-1600. However, the NRC has not reached its final enforcement decision on the issues identified by the EEIs and the PIM entries may be modified when the final decisions are made. Before the NRC makes its enforcement decision, the licensee will be provided with an opportunity to either (1) respond to the apparent violation or (2) request a predecisional enforcement conference.

** URIs are unresolved items about which more information is required to determine whether the issue in question is an acceptable item, a deviation, a nonconformance, or a violation. However, the NRC has not reached its final conclusions on the issues, and the PIM entries may be modified when the final conclusions are made.

SENIOR MANAGEMENT MEETING CODES

1	Operational Performance: A - Normal B - During Transients C - Programs and Processes
2	Material Condition: A - Equipment Condition B - Programs and Processes
3	Human Performance: A - Work Performance B - Knowledge, Skills, and Abilities C - Work Environment
4	Engineering/Design: A - Design B - Engineering Support C - Programs and Processes
5	Problem Identification and Resolution: A - Identification B - Analysis C - Resolution

DAVIS-BESSE
INSPECTION/ACTIVITY PLAN

IP - Inspection Procedure
 TI - Temporary Instruction
 Core Inspection - Minimum NRC Inspection Program (mandatory all plants)

INSPECTION/ ACTIVITY	TITLE/ PROGRAM AREA	NUMBER OF NRC INSPECTORS/ INDIVIDUALS	PLANNED DATES	TYPE OF INSPECTION/ACTIVITY- COMMENTS
IP 84750	Chemistry Inspection	2	6/8-12/98	Core
IP 81110	Operational Security Response Evaluation	4	6/29-7/3/98	Core
IP 40500	Effectiveness of Licensee Controls in Identifying, Resolving, and Preventing Problems	3	7/20-8/14/98 onsite weeks 7/20, 8/3	Core
	Operator Licensing Initial Exam	3	7/20-24/98 Initial Exam Prep Validation 8/3-7/98 Administer Initial Exam	Core
IP 83750	Radiation Protection	1	8/10-14/98	Core
IP 82701	Emergency Preparedness	1	9/14-18/98	Core
IP 83750	Radiation Protection	1	11/30-12/3/98	Core
IP 71001	Operator Licensing Requalification Exam	2	11/16-20/98	Core