



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

THE PENNSYLVANIA STATE UNIVERSITY

DOCKET NO. 50-5

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 26
License No. R-2

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to Facility Operating License No. R-2 filed by the Pennsylvania State University (the licensee), dated April 11, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied; and
 - F. Publication of notice of this amendment is not required since it does not involve a significant hazards consideration nor amendment of a license of the type described in 10 CFR Section 2.106(a)(2).

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C.(2) of License No. R-2 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 26, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

William C. Long

for

Charles L. Miller, Director
Standardization and Non-Power
Reactor Project Directorate
Division of Reactor Projects - III, IV,
V and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
Appendix A Technical
Specifications Changes

Date of Issuance: May 31, 1989

ENCLOSURE TO LICENSE AMENDMENT NO. 26

FACILITY OPERATING LICENSE NO. R-2

DOCKET NO. 50-5

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Remove Pages

41
42
48

Insert Pages

41
42
48

6.0 ADMINISTRATIVE CONTROLS

6.1 ORGANIZATION

6.1.1 STRUCTURE

The University Senior Vice President for Research and Dean of the Graduate School (level 1) has the responsibility for the reactor facility license. The management of the facility is the responsibility of the Director and the Deputy Director (level 2), who report to the Senior Vice President for Research and Dean of the Graduate School through the Head of the Nuclear Engineering Department and the Dean of the College of Engineering. Administrative and fiscal responsibility is within the offices of the Department Head and the Dean.

The minimum qualifications for the position of Director of the PSBR are an advanced degree in science or engineering, and 2 years experience in reactor operation. Five years of experience directing reactor operations may be substituted for an advanced degree.

The Director can at any time temporarily delegate his authority to the Deputy Director who can in-turn further delegate his authority to a qualified Senior Reactor Operator (level 3).

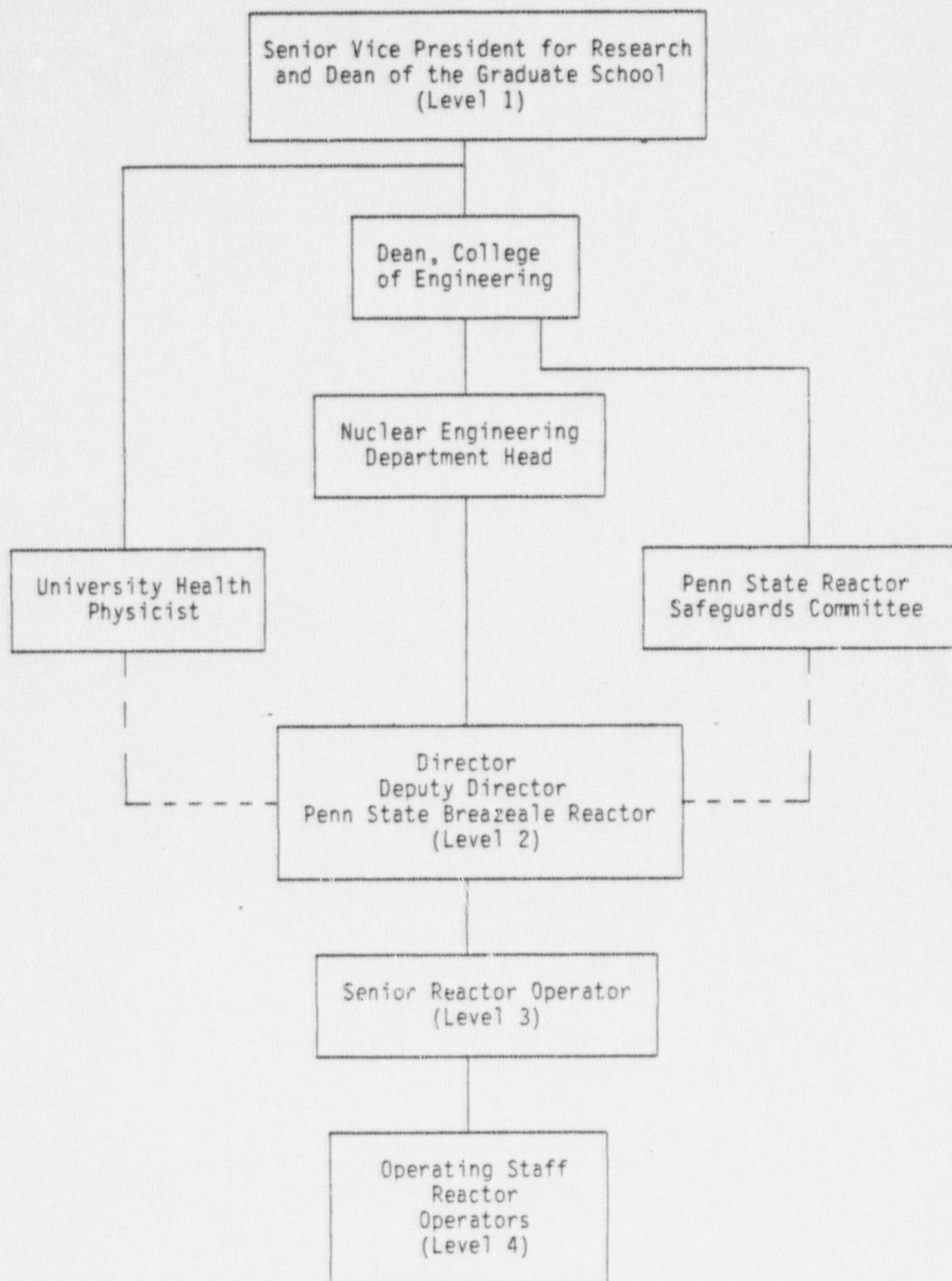
The reactor operators (level 4) report to the Senior Reactor Operator (level 3) for operational matters.

The University Health Physicist reports directly to the Office of the Senior Vice President for Research and Dean of the Graduate School. The qualifications for the University Health Physicist position are the equivalent of a graduate degree in radiation protection, 3 to 5 years experience with a broad byproduct material license, and certification by The American Board of Health Physics or eligibility for certification.

6.1.2 RESPONSIBILITY

Responsibility for the safe operation of the reactor facility shall be within the chain of command shown in the organization chart. Individuals at the various management levels, in addition to having responsibility for the policies and operation of the reactor facility, shall be responsible for safeguarding the public and facility personnel from undue radiation exposures and for adhering to all requirements of the operating license and technical specifications.

In all instances, responsibilities of one level may be assumed by designated alternates or by higher levels, conditional upon appropriate qualifications.



ORGANIZATION CHART

- f. A report shall be made to the USNRC, Document Control Desk, with a copy to the Regional Administrator.

6.6 REPORTS

6.6.1 OPERATING REPORTS

An annual report shall be submitted within 6 months of the end of The Pennsylvania State University fiscal year to the USNRC, Document Control Desk, with a copy to the Regional Administrator, including at least the following items:

- a. A narrative summary of reactor operating experience including the energy produced by the reactor, and the number of pulses \geq -\$2.00 but less than or equal to \$2.50 and the number greater than \$2.50.
- b. The unscheduled shutdowns and reasons for them including, where applicable, corrective action taken to preclude recurrence.
- c. Tabulation of major preventive and corrective maintenance operations having safety significance.
- d. Tabulation of major changes in the reactor facility and procedures, and tabulation of new tests and experiments, that are significantly different from those performed previously and are not described in the Safety Analysis Report, including a summary of the analyses leading to the conclusions that no unreviewed safety questions were involved and that 10 CFR 50.59 was applicable.
- e. A summary of the nature and amount of radioactive effluents released or discharged to environs beyond the effective control of the owner-operator as determined at or before the point of such release or discharge. The summary shall include to the extent practicable an estimate of individual radionuclides present in the effluent. If the estimated average release after dilution or diffusion is less than 25 percent of the concentration allowed or recommended, only a statement to this effect need be presented.
- f. A summarized result of environmental surveys performed outside the facility.
- g. A summary of exposures received by facility personnel and visitors, in the form indicated in 10 CFR 20.407(b), where such exposures are greater than 25 percent of that allowed or recommended in 10 CFR 20.