## Bechtel•KWU ALLIANCE

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June 16, 1989

Mr. J. Norberg Special Assistant Division of Engineering and Systems Technology Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Subject: References for Topical Report: Repair of Defective Steam Generator Tubes by Sleeving, BKAT-01-P, Revision 1, April 1989

Dear Mr. Norberg:

Per the request of Mr. H. F. Conrad, enclosed are references for the subject topical report. Attachment 1 provides the list of enclosures.

Please contact Mr. C. W. Hultman, 258-4508, if you have any further questions.

Very truly youns,

Hugo F. Pomrehn General Manager Bechtel-KWU Alliance

CAL/cam

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Attachment: 1. List of Enclosures

Enclosures: As listed on Attachment 1

cc: H. F. Conrad w/all C. W. Hultman w/o

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## ATTACHMENT 1

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## LIST OF ENCLOSURES

F.W. Pement, G. Economy, A. McIlree: "In-Situ Stress Relief of Alloy 600 U-Bends to Improve FWSCC Resistance," Proc. of the 1988 JAIF International Conference on Water Chemistry in Nuclear Power Plants, Ed. JAIF, Tokyo, 1988, Volume 2, pp 658-660

G. Slama, M. Sort: "Prevention and Repair of PWR SG Tubing," Proc. of the 1988 JAIF International Conference on Water Chemistry in Nuclear Power Plants, Ed. JAIF, Tokyo, 1988, Volume 1, pp 101-104

F.W. Pement, et al.: "Treatment of Alloy 600 Simulated Tube -Tube Sheet Transitions for Improved Resistance to PWSCC. II. Performance of Treated Specimens in Accelerated Environments," Proc. of the Third International Symposium on Environmental Degradation of Materials in Nuclear Power Systems - Water Reactors, Ed. G.J. Theus and J.R. Weeks, Warrendale, PA, 1988, pp 563-569.

F.P. Vaccaro, et al.: "Remedial Measures for SCC of Alloy 600 SG Tubing," Proc. of the Third International Symposium on Environmental Degradation of Materials in Nuclear Power Systems - Water Reactors, Ed. G.J. Theus and J.R. Weeks, Warrendale, PA, 1988, pp 570-579.

C.F. Lo, W.E. Mayo, S. Weissmann: "Comparison of Stress Relief in Alloys 690 and 600," EPRI Alloy 690 Workshop, New Orleans, LA, 1989

J.R. Crum and R.C. Scarberry, "Corrosion Testing of Inconel Alloy 690 for PWR Steam Generators," Journal of Materials for Energy Systems, Volume 4, No. 3, December 1982, p. 125-130.

A.J. Sedrik, J.W. Schultz, M.A. Cordovi: Bosholm Gijutsu 28, 1979, p. 82-95.

K. Smith, A. Klein, P. Saint-Paul, J. Blanchet: "A Material with Improved Corrosion Resistance for PWR Steam Generator Tubes."

L. von Bernus, "Steam Generator Tube Testing: Automated Defect Detection and Documentation, Analytical Measurement," Atomkernenergie/Kerntechnik, Vol. 47, 1985.

L. von Bernus, A. Curtis, "The DIWAN/SEGANS Steam Generator Inspection System, Development Goals and Field Experiences," EPRI Steam Generator Workshop, 1988

L. von Bernus, G. Engl, H. Jacob, "Recent Development in Steam Generator Degradation Analysis by Special Ultrasonic and Eddy Current Techniques," EPRI Steam Generator Workshop, 1985.