



Carolina Power & Light Company

JUL 25 1989

SERIAL: NLS-89-214

United States Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-325 & 50-324/LICENSE NOS. DPR-71 & DPR-62
TRANSMITTAL OF LOCA REPORTS

Gentlemen:

On November 17, 1988 the Nuclear Regulatory Commission approved Amendment 19 to General Electric Company (GE) document NEDE-24011-P-A (GESTAR-II). Amendment 19 to GESTAR-II supports the elimination of linear heat generation rate (LHGR) from the Technical Specifications for boiling water reactors. Elimination of LHGR from the Brunswick Plant Technical Specifications was approved by Amendments 131 and 161 dated May 25, 1989. To coincide with implementation of Amendments 131 and 161 to the Brunswick Technical Specifications, Carolina Power & Light Company is providing information, as described below, in accordance with the Safety Evaluation for Amendment 19 to GESTAR-II.

In conjunction with Amendment 19 to GESTAR-II, GE committed licensees to submit to the NRC, on a proprietary basis, a copy of the information that will be available to the operator in the plant control room. This information for each fuel bundle type consists of:

1. The axial location of each lattice in the bundle.
2. The composite MAPLHGR, considering both thermal-mechanical and emergency core cooling system requirements, as a function of average exposure for each lattice in the bundle.

General Electric also committed licensees to provide a bundle description including the same type of information found in GESTAR-II for any fuel bundle type included in a reload which is not included in GESTAR-II or one of its approved amendments.

The above information is contained in the Loss-of-Coolant Accident Analysis Reports for Unit 1, Cycle 7 and Unit 2, Cycle 8, the current cycles for each unit. These reports are classified as proprietary by GE. One copy each of the "Loss-of-Coolant Accident Analysis Report for Brunswick Steam Electric Plant Unit 1," NEDE-24165-P, Rev. 1 and the "Loss-of-Coolant Accident Analysis Report for Brunswick Steam Electric Plant Unit 2," NEDE-24053-P, Rev. 2 are enclosed.

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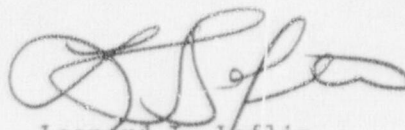
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In addition, GE indicated that each non-proprietary reload submittal would include a table of the most limiting and least limiting MAPLHGR for each multiple lattice fuel bundle type. This information is located in the Supplemental Reload Licensing Report for each Brunswick Unit. A copy of the "Supplemental Reload Licensing Report for Brunswick Steam Electric Plant Unit 1 Reload 6 Cycle 7," 23A5896, Rev. 1 and the "Supplemental Reload Licensing Report for Brunswick Steam Electric Plant Unit 2 Reload 7 Cycle 8," 23A5855, Rev. 1 are enclosed.

Enclosed are affidavits for withholding the above referenced Loss-of-Coolant Accident Analysis Reports for Unit 1 and Unit 2 as information proprietary to GE.

Please refer any questions regarding this submittal to Mr. William R. Murray at (919) 546-6661.

Yours very truly,



Leonard I. Loflin

Manager

Nuclear Licensing Section

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Enclosures

cc: Mr. S. D. Ebnetter
Mr. W. H. Ruland
Mr. E. G. Tourigny

ENCLOSURE 4

SUPPLEMENTAL RELOAD LICENSING REPORT
BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1
RELOAD 6, CYCLE 7
23A5896, REV. 1