

July 11, 1989
LIC-89-645

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102-2247
402/536-4000

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, DC 20555

- References:
1. Docket No. 50-285
 2. Generic Letter No. 89-06, "Task Action Plan Item I.D.2 - Safety Parameter Display System - 10CFR50.54(f)", dated April 12, 1989
 3. Letter from NRC (A. Bournia) to OPPD, "Summary of Audit with Omaha Public Power District on Combined Detailed Control Room Design Review and Safety Parameter Display System", dated September 30, 1987
 4. Letter from OPPD (K. J. Morris) to NRC (Document Control Desk), dated June 19, 1989, LIC-89-618
 5. Letter from OPPD (K. J. Morris) to NRC (Document Control Desk), dated June 22, 1989, LIC-89-625

Gentlemen:

SUBJECT: Certification of the Fort Calhoun Unit No. 1 Safety Parameter Display System to Requirements of NUREG-0737, Supplement 1, as Required by Generic Letter 89-06

As required by reference 2, Omaha Public Power District has conducted a certification review of the Fort Calhoun Station Safety Parameter Display System (SPDS) and concluded that the NUREG-0737, Supplement 1 requirements have been met with the exception of one item which was identified during the 1987 NRC audit of the SPDS. This item is further discussed below.

Reference 3 describes the results of the SPDS audit performed by the NRC from September 14 through 17, 1987 and lists 4 areas that were not in compliance with the requirements of NUREG-0737, Supplement 1. These deficient items (in bold) and their current status are summarized below:

1. **There is not a continuous display of information because some displays do not have critical safety function boxes, and because the audible alarm can be disabled.**

The necessary changes will be made to ensure that there is a continuous display of information. Full compliance will be achieved at the end of the 1990 outage.

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2. Implementation of procedures and operator training is inadequate because Shift Technical Advisors need more training on what computer points drive the information blocks and status bars, and what information blocks and status bars drive critical safety function boxes.

This item has been addressed and corrected by the training department. Shift Technical Advisor Training on SPDS display system logic has been completed as committed to in Reference 4. This item is considered closed.

3. There is a need to revise the design to incorporate human factor principles in resolving discrepancies on some displays (i.e. Reactor Coolant System startup values displayed to the left of the vertical axis which is inconsistent with other displays).

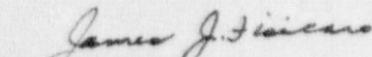
As described in reference 5, OPPD has incorporated accepted human factors principals into those display formats containing discrepancies. These changes were implemented by the end of the 1988 refueling outage. This item is considered closed.

4. Parameter selection does not include main steam line radiation.

This item has been addressed by adding the RM-064 (Main Steam Line Radiation Monitor) to the Containment Integrity screen of the SPDS. This item is considered closed.

In summary, the Fort Calhoun Station SPDS meets the requirements of NUREG-0737, Supplement 1 (taking into account the information of NUREG-1342) with the exception of the above noted deficiency on the continuous display of information which is scheduled for resolution during the 1990 refueling outage. If you have further questions on this matter or require additional information, please contact me or members of my staff.

Sincerely,


K. J. Morris
Division Manager
Nuclear Operations

KJM/pjc

c: LeBoeuf, Lamb, Leiby & MacRae
R. D. Martin, NRC Regional Administrator
A. Bournia, NRC Project Manager
P. H. Harrell, NRC Senior Resident Inspector