

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

W. L. STEWART
SENIOR VICE PRESIDENT
POWER

July 14, 1989

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D. C. 20555

Serial No 89-497
NAPS/PAK: bgp
Docket No. 50-338
License No. NPF-4

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION UNIT 1
REACTOR COOLANT STANDPIPE INSTALLATION

Generic Letter 88-17, Loss of Decay Heat Removal, required programmed enhancements consisting of hardware installations and/or modification implementation by the end of the first refueling outage that is initiated 18 months or later following receipt of the letter, or by the end of the second refueling outage following receipt of the letter, whichever occurred first. The Virginia Electric and Power Company response to NRC Inspection Report 88-01 dated April 29, 1988, and responses to Generic Letter 88-17 dated January 6, 1989 and February 3, 1989 committed to the installation of a permanent reactor vessel standpipe arrangement during the 1989 refueling outage.

Design Change Package (DCP) 88-11 was developed and implemented to direct the installation activities for the new standpipe as previously committed. However, the modification will not be completed during the 1989 refueling outage. Mechanical and electrical work for DCP 88-11 was completed during the 1989 refueling outage except for 1) installation of the spool pieces to connect the standpipe to the Reactor Coolant System, 2) installation of a metal gasket at the transmitter remote seal, 3) hydrostatic testing and 4) calibration of the transmitter and remote indicator.

To ensure the safety of other containment systems remains unaffected during a seismic event, the standpipe was left in the following condition:

1. All portions of the standpipe modification are seismically secure and verified by a stress calculation based on the as-built configuration.
2. Flushing of the new piping is complete.

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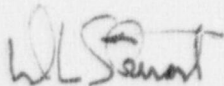
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3. All wiring is complete. This includes wiring from the Rosemount transmitter to the 7300 racks and wiring from the racks to the control room indicator and annunciator. All portions of the modification will be left de-energized.
4. The new system is tied in to existing piping only at the interface with the instrumentation blowdown header.
5. The RCS boundary will be restored with blind flanges per the existing unmodified design.

Appropriate measures have been taken to ensure the new level indication system will be operable prior to the next entry into a reduced inventory condition (i.e., vessel level lower than three feet below the reactor vessel flange) and therefore, will not impact overall plant safety.

If you have further questions or require additional information, please contact us.

Very truly yours,



W. L. STEWART

cc: U. S. Nuclear Regulatory Commission
101 Marietta Street, N.W.
Suite 2900
Atlanta, GA 30323

Mr. J. L. Caldwell
NRC Senior Resident Inspector
North Anna Power Station