

July 14, 1989

Docket No. 50-320

Mr. M. B. Roche
Vice President and Director - TMI-2
GPU Nuclear Corporation
P.O. Box 480
Middletown, Pennsylvania 17057

Dear Mr. Roche:

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SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR EVALUATION OF PDMS
SAFETY ANALYSIS REPORT

Reference: Letter with Attached Safety Assessment, 4410-88-L-0068/0378P
from F. R. Standerfer to NRC dated August 16, 1988 re "Post
Defueling Monitored Storage Proposed License Amendment and
Safety Analysis Report"

By letter dated August 16, 1988 (reference above) you submitted your Safety
Analysis Report (SAR) for long term storage of the TMI-2 facility. Based on
our continued review of your SAR we have determined that additional
information or further clarification is required. We request that you respond
to the enclosed questions.

The reporting and/or recordkeeping requirements contained in this letter
affect fewer than ten respondents. Therefore OMB clearance is not required
under P.L. 96-511.

Sincerely,

/s/

Michael T. Masnik, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

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PDR ADDCK 05000320
P PNU

[REQUEST FOR INFORMATION]

LA:PDI-4
SNorris
7/12/89

MTM
PM:PDI-4
MMasnik:cb
7/13/89

PD:PDI-4
JStolz
7/14/89

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cc w/enclosure:
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JStolz
7/14/89

Mr. M. B. Roche
GPU Nuclear Corporation

Three Mile Island Nuclear Station
Unit No. 2

cc:

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Mr. M. B. Roche
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Three Mile Island Nuclear Station
Unit No. 2

R. E. Rogan
GPU Nuclear Corporation

S. Levin
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W. J. Marshall
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ENCLOSURE

ADDITIONAL QUESTIONS CONCERNING TMI-2 PDMS SAFETY ANALYSIS REPORT

1. What system has been established to verify continued maintenance of isolation of the areas and components that assure sub-criticality?
2. Is the reactor vessel to be sealed prior to PDMS? If the indexing fixture and shielded work platform are to remain in place, what is the expected dose rate from radiation (both direct and scattered) on the 305-ft and 347-ft levels?
3. What are the locations of the fire system hose and hose reels that will be operational during PDMS?
4. Will emergency lighting be maintained in the Service and Turbine buildings during PDMS?
5. Has a decision been made as to dismantling the redwood cooling tower?
6. What is the status of procedure development for putting FS-P-1 fire pump in/out of layup or standby?
7. Have you considered the possibility and effects of radiolytic generation of hydrogen and/or nitric acid in the reactor vessel?