

ENCLOSURE 1

NOTICE OF VIOLATION

Florida Power Corporation
Crystal River Nuclear Station

Docket No. 50-302
License No. DPR-72

During the Nuclear Regulatory Commission (NRC) inspection conducted on May 31 - June 2, 1989, a violation of NRC requirements was identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10CFR Part 2, Appendix C (1989), the violation is listed below:

Technical Specification 6.8.1 requires written procedures to be established, implemented, and maintained as recommended in Appendix A of Regulatory Guide 1.33, November, 1972. Appendix A of Regulatory Guide 1.33 recommends general plant operating procedures for plant heatup and procedures for operation of the pressurizer spray control system. It also recommends administrative procedures on adherence to procedures. Administrative Instruction AI-400E, Performance and Transmittal of Procedures, Revision 1, requires that operators have procedures present when performing them, except for routine actions that are frequently repeated and where the operators are familiar with the procedure content.

1. Contrary to the above, operators failed to have procedures present for performing actions that were not routine to them. At about 0015 on May 28, 1989, operators initiated pressurizer spray from Decay Heat Removal to degas the Reactor Coolant System. The operators had not performed these actions before and were not familiar with the procedure content. This contributed to the May 29 overheating of Emergency Feedwater piping and its containment penetration and to subjecting the A steam generator to unanalyzed thermal stresses.
2. Contrary to the above, general plant operating procedure OP-02, Plant Heatup, Revision 80, was inadequate in that it did not adequately address using pressurizer spray control to degas the reactor coolant system while in Mode 5, with no reactor coolant pumps operating. There were no steps, conditions, cautions, or limits to prevent heating the Emergency Feedwater piping and its containment penetration beyond design limits or to prevent subjecting the A steam generator to unanalyzed thermal stresses. As a result, at about 0600 on May 29, 1989, these conditions occurred.

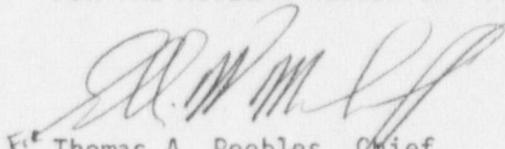
This is a Severity Level IV violation (Supplement 1).

Pursuant to the provisions of 10 CFR 2.201, Florida Power Corporation is hereby required to submit a written statement or explanation to the Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555, with a copy to the Regional Administrator Region II, and a copy to the NRC Resident Inspector, Crystal River, within 30 days of the date of the letter transmitting this Notice. This reply should be clearly marked as a "Reply to a Notice of Violation" and should include: (1) admission or denial of the

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violation, (2) the reason for the violation if admitted, (3) the corrective steps which have been taken and the results achieved, (4) the corrective steps which will be taken to avoid further violations, and (5) the date when full compliance will be achieved. Where good cause is shown, consideration will be given to extending the response time. If an adequate reply is not received within the time specified in this Notice, an order may be issued to show cause why the license should not be modified, suspended, or revoked or why such other action as may be proper should not be taken.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Peebles, Chief
Operations Branch
Division of Reactor Safety

Dated at Atlanta, Georgia
this 7TH day of July 1989