



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**

REGION III
2443 WARRENVILLE ROAD, SUITE 210
LISLE, ILLINOIS 60532-4352

September 2, 2020

Mr. Joel P. Gebbie
Senior VP and Chief Nuclear Officer
Indiana Michigan Power Company
Nuclear Generation Group
One Cook Place
Bridgman, MI 49106

SUBJECT: DONALD C. COOK NUCLEAR PLANT – NRC INSPECTION REPORT
05000315/2020010 AND 05000316/2020010

Dear Mr. Gebbie:

On August 12, 2020, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Donald C. Cook Nuclear Plant and discussed the results of this inspection with Mr. S. Lies, Site Vice President and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

/RA/

Richard A. Skokowski, Chief
Engineering Branch 3
Division of Reactor Safety

Docket Nos. 05000315 and 05000316
License Nos. DPR-58 and DPR-74

Enclosure:
As stated

cc w/ encl: Distribution via LISTSERV®

Letter to Joel P. Gebbie from Richard A. Skokowski dated September 2, 2020.

SUBJECT: DONALD C. COOK NUCLEAR PLANT – NRC INSPECTION REPORT
05000315/2020010 AND 05000316/2020010

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**U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report**

Docket Numbers: 05000315 and 05000316

License Numbers: DPR-58 and DPR-74

Report Numbers: 05000315/2020010 and 05000316/2020010

Enterprise Identifier: I-2020-010-0054

Licensee: Indiana Michigan Power Company

Facility: Donald C. Cook Nuclear Plant

Location: Bridgman, MI

Inspection Dates: July 13, 2020 to August 12, 2020

Inspectors: E. Fernandez, Reactor Inspector
G. O'Dwyer, Reactor Engineer
D. Szwarc, Senior Reactor Inspector

Approved By: Richard A. Skokowski, Chief
Engineering Branch 3
Division of Reactor Safety

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting an NRC inspection at Donald C. Cook Nuclear Plant, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

None.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

REACTOR SAFETY

71111.17T - Evaluations of Changes, Tests, and Experiments

Sample Selection (IP Section 02.01) (19 Samples)

The inspectors reviewed the following evaluations, screenings, and/or applicability determinations for 10 CFR 50.59 from 2016 through 2020.

- (1) 2017-0164-01, Fusing of Control Room Instrument Distribution Branch Circuits, Revision 0
- (2) 2018-0146-01, Compensatory Actions for ODE AR 2018-4115 Manual Operator Actions for Flushing RHR Pump Seal Heat Exchanger, Revision 0
- (3) 2018-0328-00, Unit 1 Upflow Conversion Evaluation, Revision 0
- (4) 2019-0234-00, Operation of Control Rod Drive System, Revision 0
- (5) 2019-0202-00, Unit 2 Reactor Controls and Instrumentation Upgrade. Revision 0
- (6) 2018-0261-00, Shop and Field Fabrication of Fire Protection Piping, Revision 0
- (7) 2019-340-00, 2-0HP-4022-016-003, CCW In-Leakage Procedure Alterations, Revision 0
- (8) 2017-0236-00, Updated Final Safety Analysis Report Large Break Loss of Coolant Accident Peak Cladding Temperature Changes for 2016, Revision 0
- (9) 2017-0012-00, NSAL 14-5 WRB-1 and WRB-2 Departure from Nucleate Boiling Correlation Non-Conservatism, Revision 0
- (10) 2017-0411-02, Unit 2 Up-flow Conversion Evaluation, Revision 2
- (11) 2020-0028-00, Annunciator Response: ESW And Component Cooling, 02/12/2020
- (12) 2019-0310-00, ODE 2019-11127-1 for Comp Action for CCW Flow Decreases to RHR Pump Seal Heat Exchangers, Revision 0
- (13) 2019-0176-01, Compensatory Action for ODE on Particulate in CCW Affecting Flowrate to RHR Pump Seal Heat Exchanger, Revision 0
- (14) 2019-0207-00, Replacement of Unit 1 West Motor Driven Feedwater Pump Room Cooler with an ESW Cooled Fan Coil Unit, Revision 0
- (15) 2017-0302-01, Replacement of Unit 1 East Motor Driven Feedwater Pump Room Cooler with an ESW Cooled Fan Coil, Revision 0
- (16) 2018-0285-00, Isolate One Coil of 2-HV-CLV-3 Bank 2 for More than 90 Days with Unit 2 at Power, Revision 0
- (17) 2019-0275-00, DC Cook Unit 2 Mode 6 Boron Dilution Time to Criticality, Revision 0
- (18) 2017-0120-01, Feedwater Malfunction Evaluation UFSAR Section 14.1.10.2, Revision 1

- (19) 2016-0278-01, Unit 2 Turbine Post Retrofit Blade Vibration Monitoring Modification and Associated Temporary Modification and Procedure, Revision 0

INSPECTION RESULTS

No findings were identified.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On August 12, 2020, the inspectors presented the NRC inspection results to Mr. S. Lies, Site Vice President and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.17T	Corrective Action Documents	2017-11777	50.59 Applicability Determination Incorrect	11/17/2017
		2017-11986	Perform DCE on 1-SI-158-L2/L3 Leakage Exceeding IST Limit	11/27/2017
		2017-8813	1-SI-58-L2 and 1-SI-158-L3 Excessive Leakage	09/14/2017
		2018-4115-2	Perform ODE on Unit 2 CCW High Suspended Solids	05/01/2018
		2018-5829	50.59 Screen Rev Needed for ODE-2018-4115 Comp Measure	05/29/2018
		2019-11127	U2 RHR Pump Seal Cooler Low Flow ODE with Screen SS-SE-2019-0310	11/13/2019
		2019-1679	U2 Upflow Mod 50.59 Screened as not Requiring Evaluation but U1 did	02/21/2019
		2019-4886-01	U1 RHR Pump Seal Cooler Low Flow ODE with Screen SS-SE-2019-0176 Completed	05/09/2019
		2019-6467	50.59 Fails to ID Adverse Impact Needed for ODE-2018-4115 Comp Measure	06/27/2019
	2020-0413	NESW Pump Motor Trip Alarm Response Procedures Needed Changes Required 50.59 Screen	01/14/2020	
	Corrective Action Documents Resulting from Inspection	2020-6358	NRC 50.59 Inspector Observation	08/11/2020
	Drawings	OP-1-5143-80	Flow Diagram Emergency Core Cooling (RHR) Unit No. 1	80
		OP-1-5143A-7	Flow Diagram Emergency Core Cooling (RHR) Accumulator Piping Unit No. 1	09/21/2017
		OP-2-12050-33	120V AC Control Room Instrument Distribution Cabinets "CRID-I" thru "CRID-IV" Engineered Safety System	33
	Miscellaneous	2015-0399-02	Replacement of the Radiation Monitoring System (RMS)	0
		2017-0164-01	Fusing of Control Room Instrument Distribution Branch Circuits	1
	Procedures	1-OHP-4030-102-017	RCS Pressure Isolation Valves Leak Rate Surveillance Test	22
12-THP-6020-		Component Cooling Water	19	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		CHM-305		
		PMI-6010	Radiation Protection Plan	23