



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

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PDR
5/12/89

December 2, 1988

The Honorable Lando W. Zech, Jr.
Chairman
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Chairman Zech:

SUBJECT: THREE HUNDRED FORTY-THIRD MEETING OF THE ADVISORY COMMITTEE ON
REACTOR SAFEGUARDS, NOVEMBER 17-18, 1988

The Advisory Committee on Reactor Safeguards held its 343rd meeting, in
Bethesda, Md., on November 17-18, 1988.

ACRS REPORTS/ACTIONS

The Committee prepared its report on the Safety Evaluation Report for
the "Power Reactor Inherently Safe Module" (PRISM) Design. (Report to
Chairman Zech dated November 22, 1988)

The report on this subject has been sent to you.

OTHER ACTIONS, AGREEMENTS, ASSIGNMENTS, AND REQUESTS

The Committee was briefed by representatives of the Staff on the Pro-
posed Regulatory Guide 1.9, Revision 3, and prepared its letter on ACRS
Action on Proposed Regulatory Guide 1.9, Revision 3, "Selection, Design,
Qualification, Testing, and Reliability of Diesel Generator Units as
Onsite Electric Power Systems at Nuclear Power Plants," (letter to Mr.
Stello dated November 22, 1988). The Committee concurred in the NRC
Staff's proposal to issue this Guide for public comment and stated that
it would review the proposed final version of this Guide after the
public comment period, together with the public comments and the Staff's
response to them.

The Committee prepared a letter on ACRS Action on Proposed Regulatory
Guide, Task No. EE-006-5, "Qualification of Safety-Related Lead Storage
Batteries for Nuclear Power Plants," (letter to Mr. Stello dated
November 22, 1988). The Committee concurred in the regulatory position
of this Guide.

Dr. Siess briefed the Committee on the nature of the NRC Staff's pro-
posed revisions to Standard Review Plan Sections 6.5.2, 6.5.4, and
6.5.5. The Committee decided that there is no need for further ACRS
action on this matter. Mr. Fraley has informed Dr. Murley of the
Committee's decision. (See letter from R. Fraley to T. Murley, dated
November 23, 1988.)

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The Committee continued its discussion of the NRC Staff's proposed expanded program for the in-situ testing of safety-related motor-operated valves. This subject will be discussed further at a future meeting of the Mechanical Components Subcommittee at which Industry representatives will present their views on the proposed program.

The Nominating Panel, composed of Dr. Kerr, Mr. Ward, and Mr. Carroll, reported its nomination for Committee officers for 1989. The election will be scheduled during the December 15-17, 1988 ACRS meeting. Nominations for the Member-at-Large appointment to the Planning Committee and election of new officers will be completed during the December election.

The Committee discussed the status of new members and has forwarded an additional nomination for the Commission's consideration.

Since the last report of ACRS activities, the following subcommittee meetings have been held:

- ° Mechanical Components, October 26-27, 1988 - The Subcommittee reviewed recent work related to valve reliability as part of its discussion of the NRC Staff's proposed new requirements for in-situ testing and surveillance of safety-related motor-operated valves.
- ° Advanced Boiling Water Reactors, November 15-16, 1988 - The Subcommittee continued its review of this proposed standard plant.

FUTURE AGENDA

The following items have been scheduled for consideration during the ACRS meeting on December 15-17, 1988:

Meeting with Director, NRC Office of Nuclear Regulatory Research - Discuss topics of mutual interest, including the status of implementation of recommendations of the National Research Council on Revitalizing Nuclear Safety Research and on Human Factors Research.

Quantitative Safety Goals - Review and comment on proposed NRC Staff plan for implementation of NRC's Safety Goal Policy.

Containment Systems - Review and comment on the proposed NRC recommendations for improvements to the BWR Mark I containment.

Thermal-Hydraulic Phenomena - Review NRC Code Scaling Applicability and Uncertainty (CSAU) Evaluation Methodology proposed for use with best-estimate ECCS evaluation models.

Equipment Qualification-Risk Scoping Study - Review and comment on the Equipment Qualification-Risk Scoping Study.

3LaSalle Power Oscillation Event - Briefing regarding status of programs to address implications of the core power oscillation event which occurred recently at the LaSalle Nuclear Power Plant.

Reactor Operator Regualification - Briefing regarding lessons learned from revised operator qualification methodology.

Sodium Advanced Fast Reactor (SAFR) - Initial session to discuss the SAFR design.

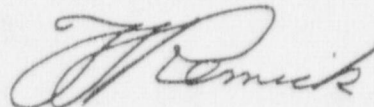
ACRS Subcommittee Reports - Reports regarding status of assigned ACRS subcommittee activities, including thermal-hydraulic phenomena regarding status of joint NRC/B&W OTSG follow-on research program, the international conference on quality and quality assurance, and the visit to the Fermi Nuclear Plant.

Anticipated ACRS Activities - Discuss anticipated ACRS subcommittee activities and topics proposed for consideration by the full Committee and the split of responsibility between the ACRS and ACNW.

Election of ACRS Officers - Discuss qualifications of members proposed as candidates for ACRS Officers for CY 1989 and conduct elections.

Meeting with Director, OGPA - Report regarding exchange of safety-related information.

Sincerely,



Forrest J. Remick
Acting Chairman