

CLINTON POWER STATION

CLINTON, ILLINOIS

ANNUAL OPERATING REPORT

JANUARY 1, 1988 THROUGH DECEMBER 31, 1988

USNRC DOCKET 50-461

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ATTACHMENTS

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ATTACHMENT 1

The work job function tabulation reflect the number of personnel who received greater than 100 mRem in any of the six identified categories. The totals section at the bottom of the report reflects all work and job functions where personnel received exposures which added up to greater than 100 mRem.

Due to job rotations during the year, there is not a direct correlation between the individual dose tabulation and the totals summary.

ILLINOIS POWER COMPANY
CLINTON POWER STATION
RR 3, P.O. BOX 228
CLINTON IL 61727

LICENSE: NPF-62

REGULATORY GUIDE 1.16 INFORMATION

END OF YEAR REPORT 1988

WORK & JOB FUNCTION	# PERSONNEL (>100 MREM)			TOTAL MAN-REM		
	UTILITY STATION CONTRACT			UTILITY STATION CONTRACT		
REACTOR OPERATIONS & SURVEILLANCE						
MAINTENANCE & CONSTRUCTION	0	0	0	0.000	.079	.233
OPERATIONS	0	33	1	0.000	10.344	1.692
HEALTH PHYSICS & LAB	0	36	24	.046	23.228	12.404
SUPERVISORY & OFFICE STAFF	0	7	0	.007	1.934	.268
ENGINEERING STAFF	0	3	1	0.000	1.046	.374
ROUTINE PLANT MAINTENANCE						
MAINTENANCE & CONSTRUCTION	0	58	122	0.000	24.057	40.544
OPERATIONS	0	0	0	0.000	0.000	.017
HEALTH PHYSICS & LAB	0	0	1	0.000	.004	.490
SUPERVISORY & OFFICE STAFF	0	0	0	0.000	.012	0.000
ENGINEERING STAFF	0	3	1	0.000	.938	.253
INSERVICE INSPECTION						
MAINTENANCE & CONSTRUCTION	0	0	0	0.000	.003	.038
OPERATIONS	0	0	0	0.000	0.000	.029
HEALTH PHYSICS & LAB	0	0	0	0.000	.004	.007
SUPERVISORY & OFFICE STAFF	0	0	0	.007	.002	.021
ENGINEERING STAFF	0	0	0	0.000	.012	0.000
SPECIAL PLANT MAINTENANCE						
MAINTENANCE & CONSTRUCTION	0	0	0	0.000	0.000	.097
OPERATIONS	0	0	0	0.000	0.000	.010
HEALTH PHYSICS & LAB	0	0	0	0.000	.004	.007
SUPERVISORY & OFFICE STAFF	0	0	0	0.000	.002	0.000
ENGINEERING STAFF	0	0	0	0.000	0.000	0.000
WASTE PROCESSING						
MAINTENANCE & CONSTRUCTION	0	24	11	0.000	9.294	12.643
OPERATIONS	0	0	0	0.000	0.000	.010
HEALTH PHYSICS & LAB	0	0	0	0.000	.097	.007
SUPERVISORY & OFFICE STAFF	0	0	0	0.000	.002	0.000
ENGINEERING STAFF	0	0	0	0.000	0.000	0.000
REFUELING						
MAINTENANCE & CONSTRUCTION	0	0	0	0.000	0.000	.011
OPERATIONS	0	0	0	0.000	0.000	.010
HEALTH PHYSICS & LAB	0	0	0	0.000	.004	.007
SUPERVISORY & OFFICE STAFF	0	0	0	0.000	.002	.049
ENGINEERING STAFF	0	0	0	0.000	0.000	0.000
TOTALS						
MAINTENANCE & CONSTRUCTION	0	97	149	0.000	33.434	53.566
OPERATIONS	0	33	5	0.000	10.344	1.769
HEALTH PHYSICS & LAB	1	68	26	.046	23.340	12.923
SUPERVISORY & OFFICE STAFF	0	7	0	.013	1.953	.268
ENGINEERING STAFF	0	9	2	0.000	1.997	.627
GRAND TOTALS	1	189	182	.059	71.068	69.223

ATTACHMENT II

PERFORMANCE CHARACTERISTICS AND TESTS

SAFETY/RELIEF VALVE CHALLENGES

There were seven valves challenged on March 18, 1988 during performance of surveillance procedure 9056.02 SAFETY/RELIEF VALVE ACTUATION. Valve actuations were satisfactory.

Twelve valves were challenged on July 12, 1988, due to an unplanned Reactor Scram. These valves were cycled for approximately two hours to relieve reactor pressure. Valve actuations were satisfactory.

Primary Coolant Specific Activity Analyses

All analyses for Specific activity of primary coolant were within the limits of Technical Specification 3.4.5.

ATTACHMENT III

FACILITY CHANGES REPORTABLE UNDER 10CFR50.59

CLINTON POWER STATION
10CFR50.59 REPORT
FOR
MODIFICATIONS
FROM APRIL THROUGH DECEMBER
1988

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0163 FA APF005

REPLACE WATT TRANSDUCER WITH EQUIVALENT MODEL

This field alteration replaces watt transducer 1JY-AP773 with a currently available model. This watt transducer provides input to the performance monitoring system to indicate the power flowing between the 4.16 KV bus 1B1 and the 4.16 KV bus 1ET4. Unlike the original model, the new model is not 1E qualified. This watt transducer does not perform a safety function. It is seismically mounted in control room panel 1H13-P851. Also, electrical failure of the watt transducer will not affect the 1E bus to which it is connected.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

87-2671 FA CF006

REPLACE AND ADD GAUGES FOR ISI TESTING REQUIREMENTS

This field alteration replaces existing gauges with gauges that have a different range, and adds new gauges to satisfy inservice inspection (ISI) testing requirements. These gauges will be installed on the residual heat removal system, the high pressure core spray system, and the diesel oil systems. The gauges will not affect the operation or seismic qualification of the systems involved, and will be installed per the original installation specifications. Also, the gauges will be isolated during normal system operation.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0270 FA CF017

REPLACE INOPERABLE DISSOLVED OXYGEN MONITORS

This field alteration replaces inoperable dissolved oxygen monitors with dissolved oxygen monitors from a different manufacturer. These instruments monitor the dissolved oxygen concentration in the reactor recirculation system, the reactor water cleanup system, and the control rod drive system. The range, power requirements, and sensitivity of the new instruments are comparable to that of the instruments being replaced. These instruments are not safety related, but are seismically mounted due to their location in the containment building.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0301 FA CPF002

REMOVE CP SYSTEM CONDUCTIVITY MONITORS FROM SERVICE

This field alteration removes conductivity elements in the condensate polishing (CP) system from service. These elements provided input to a local recorder, a local panel annunciator, and a main control room status annunciator. Subsequent conductivity monitoring of the CP system will be performed by the process sampling (PS) system. The PS system's conductivity annunciating functions are similar to those being removed from the CP system. However, so that the PS system will provide the same conductivity recording function as the CP system did before this change, this field alteration connects the PS system conductivity elements to a local PS system recorder. This change will not effect the operation of the PS system and will provide a more reliable method of monitoring the conductivity in the CP system.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0281 PM FC-12

REPLACE FC SYSTEM PUMP MOTORS

This plant modification replaces the existing fuel pool cooling and cleanup (FC) system pump motors which are not environmentally qualified, with motors that are environmentally qualified. The new motors have greater power requirements, and therefore, their power cables, control cables, and diesel fuel oil reserve requirements have been revised. The new motors will also require cooling water. This cooling water will be provided by the component cooling water (CC) system. Instrumentation has been provided to trip the FC pump motors if cooling water to them is lost because of a failure in the non-safety related CC system.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0238 PM FC-17

REVISE SUPPLY PIPING IN THE FC SYSTEM

This plant modification revises the supply piping between the fuel pool cooling and cleanup (FC) system heat exchangers and the containment and fuel building fuel pools. Specifically, this plant modification revises the point where the containment fuel pool supply line branches off from the fuel building pool supply line, replaces two temporary globe valves with a single globe valve, revises the size of two orifices, adds a flow element and local flow indication to the containment fuel pool supply line, and adds local pressure gauges to the suction side of the FC pumps. These modifications will allow more accurate control of the FC system and will maintain the original flow requirements. The revised piping and components are seismically designed and meet the requirements of the ASME Code.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0080 FA FCF003

REVISE FC SYSTEM VALVE LINEUP

This field alteration changes the valve lineup on the fuel pool cooling and cleanup (FC) system piping and instrumentation diagram to show valve 1FC012A as normally open and valve 1FC012B as normally closed. This lineup will direct flow from the upper containment pools to the fuel pool cooling surge tank instead of to the spent fuel storage pool. This change does not affect the FC system's ability to provide cooling flow to the spent fuel storage pool. USAR section 9.1.3.2 allows flow from the upper containment pools to be directed to either the surge tank or to the spent fuel storage pool. Directing the flow to the spent fuel storage pool provides the pool with an additional source of cooling, and valve 1FC012B can be opened if it is determined that additional cooling of the spent fuel storage pool is necessary.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0194 PM FP-32

PROVIDE FIRE PROTECTION TO OUTSIDE OIL FILLED TRANSFORMERS

This plant modification will improve the fire protection provided to the outside oil-filled transformers by installing a new deluge sprinkler system, by expanding the coverage of existing deluge sprinkler systems, by installing oil containment berms, and by installing heat shield walls. In addition, several security lighting fixtures will be installed to illuminate the new berms and heat shield walls. The sprinkler system modifications are in accordance with the original design requirements of the fire protection (FP) system and therefore, the FP system's functional requirements and the design basis have been maintained. Also, the installation of the berms and heat shield walls will not interfere with any plant systems as they are passive fire protection features.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0204 PM FP-78

PROVIDE SPRINKLER PROTECTION IN THE TURBINE BUILDING

This plant modification adds automatic sprinkler protection to the turbine-generator bearings, the turbine underskirt, the turbine and motor driven reactor feedwater pump rooms, and the mezzanine areas beneath the main generator. Concrete curbs will also be installed at three locations in the turbine building to control the spread of potential turbine lube oil spills and thus allow for effective fire suppression. Finally, this plant modification deletes two smoke detectors from the control room ventilation system air intake area in the control building. The new sprinkler systems have been designed in accordance with the original design requirements of the fire protection (FP) system and therefore, the FP system's functional requirements and the design basis have been maintained. Malfunctions of these systems will not increase the probability or consequence of any accident. The smoke detectors being deleted from the air intake area of the control room ventilation system are not required because of the lack of combustible material in the area.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0234 FA FPF011

PROVIDE FIRE PROTECTION WATER TO PLANT SUPPORT BUILDING

The plant support building was used as a temporary office space during plant construction, and is now being renovated to provide a permanent office space. This field alteration provides an underground water supply connection from the fire protection system to the new wet pipe sprinkler system in the plant support building. The water supply connection was designed in accordance with the requirements for underground fire protection piping. Also, the additional water demand from the wet pipe sprinkler system will not impact the capability of the existing sprinkler systems to suppress fires, in accordance with 10CFR50, Appendix R requirements.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0342 FA FWF011

REVISE FEEDWATER SYSTEM PUMP TRIP LOGIC

This field alteration changes the low feedwater suction header pressure annunciator setpoint from a temperature/pressure dependent value to 400 psig and changes the low feedwater suction pressure feedwater pump trip logic from one-out-of-one to two-out-of-three with a time delay. These changes provide a trip logic which is not susceptible to single failure and which reduces feedwater pump trips due to short fluctuations in feedwater suction header pressure. This field alteration also eliminates the automatic starting of the spare condensate and condensate booster pumps on low feedwater suction header pressure. This change may increase the number of feedwater pump trips, but this increase is adequately compensated by the logic changes which reduce the number of feedwater pump trips.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0311 FA HPF004

MODIFY AND ADD ORIFICES TO HP SYSTEM TEST RETURN LINE

This field alteration modifies an existing orifice and adds three new orifices in the high pressure core spray (HP) system test return line. This modification will eliminate the cavitation which is present at the orifice to be modified. The new orifice plates are being added at existing flanges and, in combination with the modified existing orifice, will provide the same overall pressure drop and system flow rate as the existing orifice.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0129 FA IAF003

REVISE INSTRUMENT AIR SOURCE FOR WE SYSTEM VALVES

This field alteration revises a piping and instrumentation diagram (P&ID) to show the as-built locations of the instrument air source for radwaste reprocessing and disposal equipment drain (WE) system valves OWE069, OWE070, OWE071, OWE056C, and OWE162. The revised P&ID will show that the instrument air supply for these valves comes from panel OWE02JB instead of panels OWE02JA and OWE02JC. The functions of the instrument air system and the WE system are not affected by this change. This change does not alter the source of the instrument air, only its point of control.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0307 FA IAF009

REVISE INSTRUMENT AIR SYSTEM DESIGN DRAWINGS

This field alteration revises instrument air (IA) system design drawings to show the as-built configuration of the instrument air supply to plant chilled water (WO) system valves 1WO204 and 1WO236. Valves 1WO204 and 1WO236 control the flow of chilled water to area coolers in the essential switchgear heat removal (VX) system. The instrument air supply to these valves is provided by temperature transmitter units in the VX system rather than by IA system valves 1IS852 and 1IS851. The temperature transmitter units provide the necessary control of valves 1WO204 and 1WO236.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

87-2661 PM LD-23

INCREASE MAIN STEAM LINE ISOLATION SETPOINTS

This modification increases the setpoints for the turbine building high temperature main steam line isolation instruments. These instruments isolate the main steam lines upon the detection of a steam line leak in the turbine building. The higher setpoints result in higher localized temperatures; however, these higher temperatures will cause no adverse environmental impact on systems, structures, components, or cables. Also, these instruments are not required to monitor the reactor coolant pressure boundary or limit radioactive releases below 10CFR100 limits.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0156 FA LDF010

ELIMINATE NUISANCE ALARM IN THE MAIN CONTROL ROOM

Bubbles and turbulence in the flow through the containment and drywell sump weir boxes cause nuisance alarms of the "High Change Flow Containment/Drywell Equipment/Floor Drain" annunciator in the main control room. The annunciator is intended to alarm when the rate of change of flow to the containment and drywell sump weir boxes is greater than 1 gpm/hr. This field alteration disables the annunciator by disconnecting a cable. The flow rate to the containment and drywell sump weir boxes is still monitored by existing instrumentation and high flow rates are annunciated in the main control room. This design meets the requirements of Regulatory Guide 1.45.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0149 FA LMF001

REMOVE LM SYSTEM PARTS LOCATOR PRINTER FROM SERVICE

This field alteration removes the loose parts monitoring (LM) system printer from service. The printer does not operate properly due to excessive channel noise. This printer was intended to list the order in which channel accelerometers detect loose parts, the impact energy of the loose part, and the loose part's location. Removing the printer from service will not adversely affect the LM system or equipment operation. The LM system will still have indication of all channels that detect a loose part and indication of which channel detects the loose parts first.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0202 PM K-53

INSTALL SODIUM HYPOCHLORITE BASED WATER CHLORINATION SYSTEM

This plant modification installs a sodium hypochlorite based water chlorination system to replace the chlorine gas based system presently in use. This new system will provide chlorination to the circulating water system, the plant service water system, the fire protection system, the potable water system, and the demineralized water makeup system. This modification involves the installation of bulk storage and day tanks, piping, valves, instruments, pumps, and structures at the circulating water screen house and at the makeup water pump house. The new chlorination system will perform all of the functions of the present system. Accidental spills of sodium hypochlorite will not create a toxic environment. All piping and supports are designed to seismic standards when appropriate, and core holes drilled in the circulating water screen house will not affect the screen house's seismic qualification.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0172 FA MF005

REPLACE DEFECTIVE ROSEMOUNT TRANSMITTERS

As a result of an investigation of potentially defective Rosemount transmitters, several defective Rosemount model 1152 transmitters are being replaced with Rosemount model 1153 transmitters. Also, several of the replaced transmitters are being revised from an electrical classification of 1E to non-1E because an electrical failure of these transmitters would not add a significant load to their respective power supplies. The new model 1153 transmitters have the same form, fit, and function and are qualified for the same environment as the model 1152 transmitters. Also, there is no safety impact from downgrading the transmitters from an electrical classification of 1E to non-1E because they only function to provide alarm inputs while other transmitters exist to provide the necessary indications.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0152 FA MF013

CHANGE COOLING WATER TRANSFER FOR RR SYSTEM PUMP

This field alteration revises the component cooling water (CC) system and the shutdown service water (SX) system such that the transfer of the source of cooling water for the reactor recirculation (RR) system pumps from the CC system to the SX system is initiated by a remote manual operation rather than by automatic controls. The automatic protection deleted by this modification is not necessary to meet the requirements of the Standard Review Plan. The plant operator will be able to initiate manual cooling prior to RR pump seal damage.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0087 FA MF015

REVISE GATE VALVES TO BE MANUAL FLOW CONTROL VALVES

This field alteration revises piping and instrumentation diagrams, physical piping layout drawings, and valve lists to show that valves 1FC040 and OWE095 are manual flow control valves rather than gate valves. These changes will reconcile the affected documents with the actual field configuration and with the vendor piping and instrumentation drawings. The installed flow control valves are properly sized and provide the appropriate shutoff capability.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0293 FA MF025

REVISE DESIGN DOCUMENTS TO SHOW AS-BUILT CONFIGURATION

This field alteration revises design documents for the breathing air (RA) system, the gland seal steam (GS) system, the instrument air (IA) system, and the service air (SA) system to assign equipment numbers to components in the systems and to reflect piping and valves which are currently installed in the plant. These changes provide a more accurate representation of the field configurations of the RA, GS, IA, and SA systems. Also, the additional valves and piping being shown are not safety related and are installed in non-safety portions of the systems.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0361 FA MF030

REVISE DUPLICATE EQUIPMENT NUMBERS OF VALVES IN SA SYSTEM

This field alteration revises duplicate equipment numbers of valves in the service air (SA) system. The physical configuration of the SA system is not being changed. This change will eliminate the potential confusion resulting from duplicate valve numbers.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0282 FA MSF008

MODIFICATION OF MSIV LEAKOFF LINES

This field alteration replaces the existing socket welded couplings on the leak off lines of the inboard main steam isolation valves (MSIV) with threaded union joints. This modification will make it easier to remove and replace the MSIV valve bonnets when maintenance work is required on the MSIVs. The replacement unions meet the originally specified code and design requirements.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0209 PM NB-18

REPLACE REACTOR WATER LEVEL INSTRUMENTATION

This plant modification upgrades the design and qualification of the reactor water level fuel zone range instrumentation from non-safety related to safety related by replacing the existing reactor water level measuring instruments with instruments that are seismically and environmentally qualified and which meet electrical class 1E requirements. Also, level indicators that are seismically qualified and which meet electrical class 1E requirements will be provided in the main control room. The existing power supply is adequate to support the additional electrical loads in the main control room. The seismic qualification of the main control room panels will not be affected by the new indicating instruments.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0371 PM NB-27

INCORPORATE MEOD, FWHOS, AND PROCESS COMPUTER CHANGES

This plant modification revises the core power/flow map and various instrument setpoints to incorporate the maximum extended operating domain (MEOD) analyses, and to allow the plant to operate at all power levels with one feedwater heater out of service (FWHOS). This modification also revises the process computer software to be compatible with the cycle 2 reload. The MEOD and FWHOS changes have been analyzed to ensure that the core thermal limits, the containment design parameters, and the accident analyses were not adversely affected. Also, calculations performed by the process computer with the revised software will be independently verified prior to using the process computer for performing surveillances required by the Technical Specifications.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0212 FS OGF010

REVISE ANNUNCIATOR WINDOW AND DESIGN DOCUMENTS

During startup testing, it was discovered that the flow instrument originally specified to provide input to the off gas system high flow annunciator did not have sufficient range to indicate a high flow condition. Wiring changes were made at that time to obtain a high flow input signal from a flow instrument that had sufficient range. This field alteration revises an inscription on an annunciator window and revises design documents to indicate the correct instrument that provides the high flow signal.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0147 PM PR-28

ADD MASS FLOW MEASUREMENT DEVICES TO PR SYSTEM MONITORS

This plant modification adds mass flow measurement devices to process radiation (PR) system monitors CRIX-PR001, ORIX-PR002, and ORIX-PR004. This modification will increase the accuracy of these monitors and will not have an affect on the availability of the PR system or its alarm functions.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0128 FA PSF007

RERCUTE SAMPLE LINE WITHIN FEEDWATER PROCESS SAMPLE PANEL

This field alteration revises the routing of a grab sample line within feedwater process sample panel 1PL88JA. The grab sample line presently obtains a feedwater sample from a point between two coolers within the sample panel. The routing of the grab sample line will be revised so that the feedwater sample is taken after the second cooler. This modification will allow cooler samples to be taken from the panel. The panel is not safety related and does not support a safety related system function. Also, the additional flow which will be periodically directed through the second cooler will not have significant impact on the cooling capacity of the cooler.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0189 FA PSF017

REVISE VALVE CONTROLS ON PROCESS SAMPLING SYSTEM P&ID

This field alteration revises a piping and instrumentation diagram (P&ID) to show the correct controls for component cooling water outlet valve OPS697 of process sampling system refrigeration subsystem OPS01C. This refrigeration subsystem provides chilled water to cool radwaste building process samples. The as-built configuration of the valve controls are consistent with the vendor drawing of the subsystem, and thus ensures proper operation of the subsystem and proper temperature control of chilled water.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0187 FA PSF018

REVISE PROCESS SAMPLING SYSTEM VALVE NUMBERS

This field alteration revises valve numbers on the process sampling (PS) system piping and instrumentation diagram (P&ID) and adds valves to the PS system valve list which are already shown on the P&ID. These changes reconcile discrepancies between the as-built configuration of the plant and the design documents and do not affect the PS system design basis.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0123 PM RH-32

REPLACE OPERATORS ON RH SYSTEM VALVES

The Limitorque model SMB-0-25 operators currently installed on the residual heat removal (RH) system suppression pool return valves have a thrust rating which is less than the actual thrust required to close the valves under system flow conditions. This plant modification replaces the model SMB-0-25 operators with model SMB-1-25 operators. The new operators are larger and will provide the additional thrust required to ensure that the RH system suppression pool return valves fully close. The motors are the same size as the previous motors, and therefore there is no change to the electrical requirements of the operators. The new operators are seismically and environmentally qualified and will improve the valves' stroke times. Also, the RH system piping is qualified for the larger operator. This modification also replaces the valves' stem, yoke, and other associated parts to accommodate the new operators. These parts meet the material requirements of the original parts.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.53 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0215 PM RS-04

ADD SWITCHES TO REMOTE SHUTDOWN SYSTEM

This plant modification adds a switch and indicating light to the division 1 remote shutdown panel to provide control of 4.16 KV breaker 252-AT1AA1. This plant modification also installs switches and indicating lights in various motor control centers to eliminate the need for electrical jumpers, rewiring, and fuse removal when utilizing the division 2 remote shutdown method. The new switches are seismically qualified and will not affect the seismic qualification of the remote shutdown panel or the motor control centers. The additional cable runs and wiring installations maintain the electrical separation criteria and do not exceed the allowable cable tray loadings. The existing power supplies are adequate for the new electrical loads.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0184 FA SXF004

REMOVE ELECTRICAL CONNECTIONS FROM SX SYSTEM VALVES

This field alteration removes all electrical connections from the Limitorque operators on shutdown service water (SX) system valves 2SX073A and 2SX073B. These valves were intended to provide water to the Unit 1 standby gas treatment (VG) system charcoal filter beds from the Unit 2 SX pumps. The Unit 2 SX pumps, however, were never installed and the piping that was to carry water to these valves has been terminated with end caps. Since these valves serve no function, the removal of power to the valve operators will not impact the SX or VG systems.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0199 FA VDF001

ADD RELAYS AND REVISE SETPOINTS IN THE VD SYSTEM

This field alteration adds relays to the trip circuitry of diesel generator room ventilation (VD) system fans 1VD01CA, 1VD01CB, and 1VD01CC so that the circuitry will trip the fans upon any initiation of the diesel generator bay carbon dioxide fire suppression system. This field alteration also raises the diesel generator bay low ambient temperature alarm setpoint to 65 degrees fahrenheit. These changes will allow the VD system to operate as described in the USAR. Also, the relays added to the trip circuitry have been siesmically and electrically qualified and therefore, maintain the design basis of the VD system.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0004 PM VS-07

INSTALL BACKUP COOLING SYSTEM IN SERVICE BUILDING

This plant modification provides a backup cooling system to serve the security areas of the service building. This system will ensure that adequate cooling is provided to the security areas if the normal cooling provided by the service building ventilation system is lost. The backup cooling system is located in the service building and does not interact with any plant systems.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0186 FA VWF004

REVISE VW SYSTEM DESIGN DRAWINGS

This field alteration revises design drawings for the radwaste building ventilation (VW) system to add instruments and an instrument number not previously shown, and to correct an item number. These revisions reflect the as-built and as-tested configuration of the VW system. The functions of the system remain as described in the USAR.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0233 FA VWF005

REVISE EQUIPMENT NUMBERS OF BALANCING DAMPERS IN VW SYSTEM

This field alteration revises the equipment numbers of balancing dampers in the radwaste building ventilation (VW) system to eliminate duplicate equipment numbers. The renumbering of these dampers will have no effect on the operation of the VW system.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0188 FA IMF008

REMOVE WM SYSTEM SILICA ANALYZERS

This field alteration removes the silica analyzers, the silica recorder, and the silica analyzer failure annunciator in the demineralized water makeup (WM) system from service. The silica analyzers are not required for WM system operation. The silica content of the demineralized makeup water will be monitored by taking grab samples of the water before it is used in the plant.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0329 FA WSF007

ADD ORIFICE TO PLANT SERVICE WATER SYSTEM

This field alteration adds an orifice to the plant service water (WS) system generator stator cooler and hydrogen cooler return header. This orifice will increase the backpressure in the header and reduce the cavitation across the outlet valves of the generator stator coolers. The cooling capability of the generator stator and hydrogen coolers will not be adversely impacted by the installation of the orifice. The design of the orifice meets the original design requirements of the WS system piping.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0318 FA WXF007

REPLACE OUTDATED CLOSED CIRCUIT TELEVISION CAMERAS

This field alteration modifies the radwaste system drum handling crane closed circuit television system by replacing outdated cameras 0WX37SA, B and C. Also, the instrument air lines which provided air to the cameras are being capped because air is not required to keep dirt off of the lenses of the new cameras. The cameras are used for crane operation and for monitoring activities in the truck bay and storage area. This modification will allow the camera system to operate as originally intended.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0299 FA WZF001

CORRECT DISCREPANCIES IN WZ SYSTEM P&ID

This field alteration corrects discrepancies in the piping and instrumentation diagrams (P&ID) for the chemical radwaste reprocessing and disposal (WZ) system to show two local instruments as being panel mounted and to correct the drawing zone reference on a line continuation flag. These changes do not affect the design bases or function of the WZ system or any interfacing systems.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

CLINTON POWER STATION
10CFR50.59 REPORT
FOR
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LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0111 TEMP MOD 88-028 GAG OPEN RE AND RF SYSTEM CONTAINMENT ISOLATION VALVES

This temporary modification gags open the containment and drywell isolation valves for the equipment drain (RE) system and the floor drain (RF) system during Planned Outage #2 while the plant is in operating mode 4. Since these isolation valves close upon loss of instrument air, gagging them open will allow the RE and RF systems to remain operable while the instrument air system is taken out of service. Per the Technical Specifications, these isolation valves are not required to be operable while the plant is in operating mode 4.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0122 TEMP MOD 88-034 INSTALL LARGER OPERATOR ON RH SYSTEM VALVE

The present operator on residual heat removal (RH) system valve 1E12-F024A is not capable of closing the valve under normal system flow. This temporary modification replaces the current Limitorque model SMB-0-25 operator with a Limitorque model SMB-1-25 operator, as well as the valve's stem and yoke. The new operator is capable of exerting a larger amount of thrust on the valve stem and thus, permits the valve to close under normal system flow. The new operator is seismically and environmentally qualified and will improve the valve's stroke time. The motor is the same size as the previous motor, and therefore there is no change to the electrical requirements of the operator. Also, the RH system piping is qualified for the larger operator. This temporary modification will be in place only while the plant is in operating mode 4.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LGG DOCUMENT
NUMBER EVALUATED

TITLE

38-0208 TEMP MOD 88-036 MODIFY CIRCULATING WATER SYSTEM VALVES AND INTERLOCKS

This temporary modification installs electrical jumpers to bypass a circulating water (CW) system pump start interlock, adds blank flanges on condenser water box priming line valves 1CW018 and 1CW019 and condenser inlet water box priming valves 1CW01MC and 1CW01MD, and removes the internals from condenser outlet water box priming valves 1CW01MA and 1CW01MB. The CW system pump start interlock prevents the starting of the CW pumps when the condenser water boxes are not full, and thus prevents damage to the condenser from waterhammer. This interlock is being bypassed to allow the water boxes to be filled with the CW pumps instead of with the vacuum priming system. While this interlock is bypassed, the operational methodology of the CW system will be revised to minimize the potential for waterhammer in the water boxes. The modifications to the condenser water box priming valves will prohibit the use of the vacuum priming system for filling the condenser water boxes. The priming valves are being modified to facilitate testing which will quantify air leakage into the water boxes.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0127 TEMP MOD 88-038 GAG OPEN INSTRUMENT AIR SYSTEM VALVES

This temporary modification blocks open the instrument air (IA) system containment inboard isolation valve 1IA006 and drywell outboard isolation valve 1IA007 while the plant is in operating mode 4 to allow the instrument air supply to be maintained in the containment while maintenance is being performed on these isolation valves. Blocking the valves open will render them inoperable. However, per the Technical Specifications, these valves are not required to be operable while the plant is operating in Mode 4. This temporary modification will be removed prior to the plant entering operating modes 1, 2 or 3.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0131 TEMP MOD 88-042 REMOVE LABORATORY VENTILATION SYSTEM COOLING COIL FOR REPAIR

The laboratory ventilation (VL) system is equipped with a cooling coil (OVL04A) which provides cooling to the laboratory and laundry areas. The cooling coil is not functioning because of a ruptured tube. This temporary modification permits operation of the VL system with the cooling coil removed for repair. The VL system is not safety related and removal of the cooling coil will not adversely affect the system's operation since the coil is already inoperable.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0325 TEMP MOD 88-044 GRAVITY FEED SODIUM HYPOCHLORITE TO CW AND WS SYSTEMS

This temporary modification provides chlorination for the circulating water (CW) system and the plant service water (WS) system by gravity feeding sodium hypochlorite near the pump suctions of these systems. The addition of sodium hypochlorite will reduce the biofouling and the intrusion of corbicula into the CW and WS systems. The storage of sodium hypochlorite will not create a chemical hazard and the concentrations used will not have an adverse affect of system piping.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0182 TEMP MOD 88-059 BYPASS RADWASTE FEED TANK LOW LEVEL PUMP TRIP

This temporary modification bypasses the radwaste feed tank low level pump trip for radwaste floor drain evaporator feed tank pump 1WF03P. This will allow feed tank 1WF03T to be completely pumped dry prior to maintenance being performed in the tank. The function of the low feed tank level trip is to protect the feed tank pump from cavitation. To prevent pump cavitation while this temporary modification is in place, an operator will be stationed at the pump so that it can be immediately shut off at the first sign of cavitation. This trip function will be restored when maintenance in the feed tank is complete.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0205 TEMP MOD 88-066 BYPASS REFUELING PLATFORM INTERLOCK

This temporary modification bypasses a refueling bridge interlock which prevents the refueling bridge trolley from entering the fuel transfer tube upender zone. This interlock ensures that the refueling bridge trolley will not interfere with the fuel transfer tube during refueling activities. Bypassing the interlock will allow the refueling bridge and the refueling bridge trolley to be positioned for maintenance activities. This temporary modification will be removed prior to placing the plant in the refueling mode.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0285 TEMP MOD 88-071 INSTALL AIR JUMPER AROUND VALVE

This temporary modification installs an air jumper around solenoid operated valve 1FSV-VR104. This solenoid operated valve controls the instrument air supplied to the air operated continuous containment purge supply air fan A isolation damper. This modification will maintain the isolation damper in the open position and allow continued operation of supply air fan A while the inoperable solenoid valve is repaired. The isolation damper does not have a containment isolation function, but prevents the backflow of air through the train A fan while the train B fan is operating.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0296 TEMP MOD 88-072 INSTALL TEMPORARY LIFTING DEVICE ON FUEL HANDLING PLATFORM

This temporary modification installs a temporary lifting device to the upper trolley handrail of the fuel handling platform. This lifting device will be used to raise and lower a test mandrel while performing a drag test of the spent fuel storage racks. Fuel will not be handled and fuel will not be stored in the storage racks while the temporary lifting device is installed. Also, the lifting device will not be used over any other equipment which may be damaged by its failure.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0346 TEMP MOD 88-073 ALLOW HWC TEST EQUIPMENT TO REMAIN INSTALLED

Temporary test equipment had been installed to allow its preconditioning prior to its use in the hydrogen water chemistry (HWC) test. The HWC test has been postponed. This temporary modification will allow the test equipment to remain installed until the HWC test is performed. All isolation valves between the test equipment and the plant systems will remain closed. Thus, system operability will not be affected.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0368 TEMP MOD 88-080 INSTALL TEMPORARY INSTRUMENTS IN THE FC AND CC SYSTEMS

This temporary modification installs temporary instruments in the fuel pool cooling and cleanup (FC) system and in the component cooling water (CC) system until permanent instruments are obtained. The permanent instruments are required per plant modifications FC-12 and FC-17. The temporary instruments will support post modification testing and operation of the FC system until the permanent instruments are available. Failure of the temporary instruments will not prevent the FC system or other systems from performing their intended functions.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0355 TEMP MOD 88-084 MODIFY FUEL BUILDING WALL PENETRATION SEAL

This temporary modification modifies a fuel building wall penetration seal to allow the routing of cryogenic tubing from liquid nitrogen tanks located outside of the fuel building to a freeze chamber installed on component cooling water system piping. The integrity of the modified fuel building wall penetration will be verified to ensure that secondary containment integrity is maintained. Also, the placement of liquid nitrogen tanks outside the fuel building and the routing of liquid nitrogen inside the fuel building will not create missiles or impact the function of any plant equipment.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0389 TEMP MOD 88-085 PROVIDE TEMPORARY POWER SUPPLY TO FP SYSTEM EQUIPMENT

This temporary modification provides a temporary power supply to fire protection (FP) system panels and printers. The power will be supplied from a regular lighting cabinet. Although the temporary power supply for this modification is not from a class 1E source, it will maintain the operability of the panels and printers during a planned outage of the normal 1E power supply provided by the Division 1 bus. This modification will be installed just prior to the start of the Division 1 bus outage and will be removed as part of the restoration of the Division 1 bus.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

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TITLE

88-0360 TEMP MOD 88-086 BYPASS INTERLOCKS ON FUEL HANDLING PLATFORM

This temporary modification bypasses interlocks on the fuel handling platform to allow the monorail hoist operator to control the platform without a second person present, and to allow movement of the platform when the fuel handling mast is not centered in the gate openings of the fuel transfer pool. The purpose of these interlocks is to prevent the fuel handling mast from colliding with pool obstacles during movements of spent fuel. These interlocks will be bypassed only while transferring new fuel from the new fuel storage vault to the spent fuel pool using the monorail hoist. During these transfers, the fuel handling mast will be positioned such that it will not collide with pool obstacles. Also, the lateral movements of the new fuel will be made with the fuel suspended above the tops of the pools.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0365 TEMP MOD 88-088 BYPASS CYCLED CONDENSATE STORAGE SYSTEM PUMP TRIP

During the first refueling outage, a planned outage of the auxiliary power (AP) system will deenergize a relay and cause a cycled condensate storage (C) system tank low level trip of the CY system pumps. The deenergized relay will also remove a permissive signal for the CY system pumps to start. This temporary modification will lift the leads on the relays' trip contacts and install jumpers across the relays' permissive contacts to permit operation of the CY system pumps during the AP system outage. The CY system tank low level pump trip and pump start permissive, which prevent cavitation damage to the CY system pumps, are disabled by this modification. However, precautions will require frequent monitoring of the water level in the CY system tank to preclude a low level condition. Also, plant safety would not be impacted if the CY system pumps were to become inoperable because of cavitation.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0369 TEMP MOD 88-092 ROUTE FREEZE SEAL VENT TO SERVICE AIR SYSTEM PIPING

This temporary modification routes rigid piping from the vents on a freeze seal apparatus to a flanged connection on a service air (SA) system line so that the nitrogen gas vented from the freeze seal can be released to the atmosphere outside of the fuel building. Failure of the temporary piping will result in a breach of the secondary containment boundary. However, if the operators of the freeze seal apparatus detect a failure of the temporary piping, an isolation valve in the SA system piping will be closed to restore the secondary containment boundary.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0382 TEMP MOD 88-094 BLOCK OPEN CONTAINMENT AND DRYWELL ISOLATION VALVES

This temporary modification blocks open containment and drywell isolation valves in the breathing air system, the containment equipment and floor drain systems, the instrument air system, and the service air system during the first refueling outage when the plant is in operating modes 4 and 5. Preventive maintenance activities during the first refueling outage will deenergize the solenoids of these valves, allowing the valves to close. This modification will maintain the valves in the open position and allow their respective systems to function while the preventive maintenance activities are performed. This modification will also prevent these valves from performing their automatic containment isolation function. However, per the Technical Specifications, this function is not required while the plant is in operating modes 4 and 5.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

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LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0191 1001.01 R6

CPS ORGANIZATION, RESPONSIBILITIES, AND QUALIFICATIONS

This plant administrative procedure defines the Clinton Power Station (CPS) organization, division of responsibilities, and minimum personnel qualifications. This procedure is being revised to reflect the current management organization of CPS. This revision is consistent with the CPS organization as described in the Technical Specifications.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0212 2800.19 R0

CONDENSER WATER BOX AIR ACCUMULATION TEST

This test procedure determines the amount of air accumulation in the condenser water boxes during plant operation. The test requires the installation of temporary gauges, the removal of a loop seal and the internals of two float valves, and the installation of blind flanges at two other float valves. Failure of the temporary gauges will not affect any instrumentation required for safe operation and shutdown of the plant. Removal of the loop seal and modification of the float valves will not affect other systems because isolation valves in the effected piping will be closed, which is their normal position.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0313 2800.23 R0

PRECONDITIONING FOR THE HYDROGEN WATER CHEMISTRY TEST

This test procedure will install temporary test equipment and tubing to the reactor water cleanup (RT) system piping, to process sample lines of the feedwater (FW) system, and to the inlet and outlet headers of the condensate (CD) system. This test procedure also provides for the installation of a temporary cooler to cool the water in a portion of the tubing connected to the RT system. Reactor coolant flow will then be established through the test equipment to precondition it prior to performing the hydrogen water chemistry test. This preconditioning will not impact the operability of permanent plant equipment or prevent plant equipment from performing its safety function. Also, failure of the test equipment will not prevent plant equipment from performing its safety function.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0302 4979.02 R2 CANC CRITICALITY ALARM RESPONSE DURING FUEL RECEIPT

This plant procedure identifies actions to be taken by personnel in response to criticality monitor alarms which occur during fuel receipt. This procedure is being cancelled because the Clinton Power Station Operating License grants an exemption to the requirements for providing criticality alarms during fuel receipt.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0213 9061.12 R23

PROCESS SAMPLING VALVE POSITION VERIFICATION TEST

This test procedure provides instructions for verifying the position of valves in the post accident sampling system (PASS), which is part of the process sampling system, while the plant is shut down. This revision adds the steps necessary to allow the test to be performed while the plant is operating. Performing this position verification test while the plant is operating will not affect the operation of any equipment required to safely operate or shut down the plant.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0297 9861.07 R20

DRYWELL BYPASS LEAK RATE TEST

This is a new test procedure which will verify that the drywell bypass leak rate is within the design limits. The test will be performed while the plant is shutdown. The test method ensures that the structures and components critical to drywell integrity are conservatively leak tested by pressurizing the drywell with air. Drywell pressure instrumentation will be isolated to prevent the drywell pressurization from generating erroneous loss of coolant accident signals.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG NUMBER	DOCUMENT EVALUATED	TITLE
88-0158	CR 1-88-02-054	AS-BUILT CONFIGURATION OF LIMIT SWITCHES

This condition report documents the as-built configuration of the limit switches installed on reactor core isolation cooling (RI) system testable check valve 1E51-F066. The original design specified that the limit switches be installed to provide indication of the disc position of the valve. However, the limit switches have been installed on the valve actuator and do not indicate disc position because of a lost motion device between the actuator and the disc. The disc position indication was originally specified as part of the valve's testable feature which allowed the valve to be stroke tested while the plant was operating. Since the current testing program only requires that the valves be stroke tested while the plant is shutdown, the testable feature is not utilized and disc position indication is not required.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0157	CR 1-88-03-063	RECLASSIFY RH SYSTEM RADWASTE ISOLATION VALVES
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This condition report revises the classification of residual heat removal (RH) system radwaste isolation valves 1E12-F040 and 1E12-F049 from passive safety related to active safety related. These valves automatically close upon receiving a signal of low reactor water level, high drywell pressure, RH system heat exchanger room high area temperature, or RH system heat exchanger room cooler high differential temperature. These valves and their operators are seismically and environmentally qualified. Also, the valve operators are powered from 1E buses.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0150 CR 1-88-04-036 REVISE EQUIPMENT NUMBERS ON WX SYSTEM

This condition report revises valve and device numbers on piping and instrumentation diagrams, logic diagrams, and electrical schematics of the radwaste sludge processing (WX) system. These revisions correct transcription errors, provide clarification, and incorporate changes to equipment identification numbers. These revisions do not change the physical configuration of the WX system.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0133 CR 1-88-04-071 REVISE VALVE 1C41-F336 FROM TESTABLE TO NON-TESTABLE

This condition report revises a piping and instrumentation diagram to show the as-built configuration of check valve 1C41-F336 as not having a testable feature. The testable feature is not required to verify operability of the check valve because a flow rate test and leak rate test are performed instead.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER, 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0132 CR 1-88-04-139 REVISE PROCESS SAMPLING SYSTEM EQUIPMENT NUMBERS

This condition report revises the equipment identification numbers assigned to the suction valves of the main secondary and alternate secondary cooling pumps (1PS01PA and OPS01PB) in the process sampling (PS) system. This change will eliminate duplicate equipment identification numbers. There are no changes to plant hardware or equipment parameters.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0384 CR 1-88-11-034 CORRECT COMPUTER INPUTS TO THE DISPLAY COMPUTER SYSTEM

This condition report revises a piping and instrumentation diagram (P&ID) to correct the source of inputs into the display computer (CZ) system. The P&ID incorrectly shows an input to the CZ system from the reactor recirculation jet pump non-calibrated flow transmitters. The correct input is from the reactor recirculation jet pump calibrated flow transmitters. The original design intent of this input was to provide calibrated jet pump flow.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG NUMBER	DOCUMENT EVALUATED	TITLE
88-0117	EPIP EC-09 R2	SECURITY DURING EMERGENCIES

This Emergency Plan Implementing procedure (EPIP) provides guidance for the security force in the event that the Clinton Power Station (CPS) Emergency Plan is implemented. This revision deletes unnecessary definitions and modifies other definitions to be consistent with the CPS Physical Security Plan, revises the organization who is responsible for issuing Emergency Response Organization access badges, adds a requirement for the posting of security force personnel at emergency response facility entrances, adds information regarding personnel accountability and the evacuation of non-essential personnel, and updates the personnel titles in accordance with the recent management reorganization. The personnel title changes require a revision to the Emergency Plan, which is part of the USAR. The rest of the changes are consistent with the Emergency Plan.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0366	EPIP FE-05 R5	EMERGENCY EQUIPMENT AND SUPPLIES
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This Emergency Plan Implementing Procedure (EPIP) provides instructions for periodically performing an inventory of emergency equipment and supplies. This revision identifies Decatur Memorial Hospital as a location for a hospital kit, revises personnel titles, adds clarification for the locking or sealing of emergency kits and supplies, revises the contents and quantities of items in various emergency kits, and states that dosimeters and dosimeter chargers will be stored in the emergency equipment inventories of the Technical Support Center and the Emergency Operations Facility rather than in field or decontamination kits. All of the above revisions, with the exception of the storage of dosimeters and dosimeter chargers, comply with the requirements of the Emergency Plan. Although the dosimeters and dosimeter chargers will not be in the field or decontamination kits, they will be available for issuance as appropriate.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0276 EPIP MS-04 R0 PROCESSING NRC AND IDNS PERSONNEL DURING AN EMERGENCY

This is a new Emergency Plan Implementing Procedure (EPIP) created to provide the administrative mechanism to process the Nuclear Regulatory Commission (NRC) Incident Response Team and Illinois Department of Nuclear Safety (IDNS) personnel as they report to Clinton Power Station during an emergency. This procedure conforms to the requirements of the Emergency Plan; however, the list of EPIPs in the Emergency Plan will be revised to reflect this new procedure.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0259 FDDR, FECH, ECN REVISE VALVE STROKE TIMES

These field deviation disposition requests, field engineering change notice, and engineering change notices revise valve stroke times in order to prevent excessive inservice inspection surveillance requirements. The revised valve stroke times will not adversely impact system performance, core thermal limits, or radiological consequences.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0211 FECH 23954

RECLASSIFY RH SYSTEM VALVES 1E12-F037A AND 1E12-F037B

This field engineering change notice revises the classification of residual heat removal (RH) system containment fuel pool shutoff valves 1E12-F037A and 1E12-F037B from passive safety related to active safety related. These valves are containment isolation valves. The valves and their operators are seismically and environmentally qualified. Also, the valve operators are powered from 1E buses.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0210 FECH 23955

RECLASSIFY RESIDUAL HEAT REMOVAL SYSTEM CHECK VALVE

This field engineering change notice revises the classification of residual heat removal (RH) system shutdown cooling inboard bypass check valve 1E12-F475 from passive safety related to active safety related. This valve must close to perform a containment isolation function and also must open to provide thermal relief for the piping between the inboard and outboard isolation valves of the RH system shutdown cooling line. This valve is seismically and environmentally qualified.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0230 FEEN 24008

RECLASSIFY RESIDUAL HEAT REMOVAL SYSTEM RELIEF VALVES

This field engineering change notice revises the classification of residual heat removal (RH) system pump A/B suction relief valves 1E12-F005, 1E12-F017A, and 1E12-F017B from passive safety related to active safety related. These valves perform a containment isolation function and protect the RH suction piping from overpressurization. These valves are seismically and environmentally qualified.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0376 GE 23A5921 R0

SUPPLEMENTAL RELOAD LICENSING SUBMITTAL

This document summarizes the analyses which provide the licensing basis for the new configuration of the reactor core after the replacement and rearrangement of fuel bundles during the first refueling outage. Operation of the plant under the provisions of the reload analyses does not change any mode of plant operation or increase the consequences of accidents as analyzed in USAR.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG	DOCUMENT	
NUMBER	EVALUATED	TITLE

88-0144	PDR 88-0214	3113.01 CIRCULATING WATER
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This procedure deviation for revision adds a section to the circulating water (CW) system operating procedure to allow the use of the CW pumps for filling the condenser water boxes without the use of the vacuum priming system. This revision provides the operational methodology necessary to minimize the potential of waterhammer that may occur in the condenser water boxes.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0310	PEMRDM 249-395	TEMPORARY PLACEMENT OF NITROGEN GAS TUBE TRAILER
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During the first refueling outage, a nitrogen gas tube trailer will be placed between the inner and outer airlock doors of the fuel building railroad bay. This nitrogen gas supply is required for the precharging of the hydraulic control units (HCU) during the outage. Locating the trailer in the fuel building railroad bay and routing a hose from the trailer to the HCU precharging station in the containment will not impact the secondary containment boundary, create additional fire loading, or generate missiles with greater energy than those already evaluated.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

38-0258 RF-1 FWP

FACILITY WORK PLAN FOR THE FIRST REFUELING OUTAGE

The facility work plan for the first refueling outage requires that desks, chairs, work benches, and a staging area for refueling equipment be set up in the control building. Also, a snubber test trailer will be parked in the fuel building. The above equipment will not create any seismic or fire hazards.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0145 TPD 88-0123

3113.01 CIRCULATING WATER

This temporary procedure deviation adds a section to the circulating water (CW) system operating procedure to allow the use of the CW pumps for filling the condenser water boxes without the use of the vacuum priming system. This revision provides the operational methodology necessary to minimize the potential of waterhammer that may occur in the condenser water boxes.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0103 USAR 1.8

COMPLIANCE TO REGULATORY GUIDE 1.64, REVISION 2

USAR Section 1.8 provides the project position on compliance to Regulatory Guides which are applicable to Clinton Power Station. Regulatory Guide 1.64, Revision 2, Quality Assurance Requirements for the Design of Nuclear Power Plants, states that "regardless of their title, individuals performing design verification should not ... have immediate supervisory responsibility for the individual performing the design". This USAR change will take exception to the above statement by allowing an immediate supervisor to perform the design verification provided that the supervisor is the only technically qualified individual, the need is individually documented and approved in advance by the supervisor's management, and quality assurance audits review the frequency and effectiveness of the use of supervisors as design verifiers to guard against abuse. This revision is in accordance with Revision 2 of Standard Review Plan section 17.1.II.3.E4. This section of the Standard Review Plan supercedes Regulatory Guide 1.64, Revision 2.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0185 USAR 1.8 & 5.2

COMPLIANCE TO REGULATORY GUIDE 1.45, REVISION 0

USAR Section 1.8 provides the project position on compliance to regulatory guides which are applicable to Clinton Power Station (CPS). Regulatory Guide 1.45, Revision 0, Reactor Coolant Pressure Boundary Leakage Detection Systems, specifies that the leakage detection system in the containment building should be capable of detecting unidentified leakage at a rate of one gallon per minute in less than one hour by the use of an airborne particulate radioactivity monitoring system. This USAR change will take exception to the above requirement and allow the monitoring of sump and condensate flow as the primary means of quantifying unidentified leakage, while airborne particulate radioactivity monitoring will be used as a secondary means. Since the amount of activity that would become airborne following a one gallon per minute leak in the reactor coolant pressure boundary varies depending on the leak location and coolant temperature, activity, and pressure, accurate correlations between airborne radioactivity and leak rates cannot be made.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN
UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0272 USAR 13.2

OPERATOR AND SHIFT TECHNICAL ADVISOR TRAINING

This USAR change revises the training program descriptions for non-licensed operators, reactor operators, senior reactor operators, and shift technical advisors to reflect the implementation of a systems approach to training. This training approach provides performance based training in an environment that continuously evaluates the training program's effectiveness. Also, the revised training program maintains regulatory commitments and compliance to the applicable federal regulations.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0130 USAR 13.5.1.2

SAFETY REVIEW GROUPS

USAR section 13.5.1.2 states that the Facility Review Group (FRG) will approve and that the Nuclear Review and Audit Group (NRAG) will authorize changes to procedures which conflict with the intent of the Operating License or Technical Specifications, or which involve an unreviewed safety question. This USAR section is being revised to correctly describe the FRG and the NRAG as reviewers of the above changes. This revision will make USAR section 13.5.1.2 consistent with sections 6.5.1 and 6.5.2 of the Technical Specifications.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG NUMBER	DOCUMENT EVALUATED	TITLE
88-0136	USAR 6.2.7.2	ALLOW REFUELING GATE OPEN DURING POWER OPERATION

USAR section 6.2.7.2 is being revised to allow the refueling gate in the upper containment pool to be open, instead of closed, during power operation. This will improve the cleaning of water in the dryer storage/fuel transfer pool by the fuel pool cooling and cleanup system. Operating with the refueling gate open will allow more water to be transferred from the upper containment pool to the suppression pool during an upper pool dump. However, the resultant suppression pool water level is still less than the water level used for the analysis of containment structural integrity.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0167	USAR 6.3.2.6B	REVISE ECCS PUMP TESTING REQUIREMENTS
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This USAR change revises the testing frequency of the emergency core cooling system (ECCS) pumps from once a month to quarterly and the test duration from one hour to a minimum of five minutes. This change will make the testing requirements specified in the USAR consistent with the testing requirements specified in the edition and addenda of the ASME Boiler and Pressure Vessel Code, Section XI currently referenced in 10CFR50.55a.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG NUMBER	DOCUMENT EVALUATED	TITLE
37-2561	USAR 7.3.1.1.2	REVISE POWER SUPPLY DESCRIPTION FOR RELAYS IN THE LD SYSTEM

This USAR change revises the description of the power supplies for relays 1UAY-LD506A, 1UAY-LD506B, and 1UAY-LD506C in the leak detection system to state that the relays are powered from safety buses 1A1, 1B1, and 1C1 respectively rather than from the nuclear system protection system (NSPS). These relays are part of the circuitry which isolates the main steam isolation valves (MSIVs) when a leak is detected in the turbine building steam tunnel. The effect of having the relays powered from the safety buses instead of from the NSPS will result in the MSIVs closing immediately after a loss of offsite power (LOOP) accident because of a loss of power to the relays rather than 28 seconds after a LOOP because of insufficient vacuum in the main condenser. The change in the MSIV closure sequence will not affect the minimum critical power ratio limit of the reactor core. Also, a single failure of the safety buses will not result in an MSIV closure.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0332 USAR 7.3.1.1.2 MAIN STEAM LINE RADIATION MONITORING

This USAR change revises the number of trip channels for each main steam line radiation monitor from three channels to two channels and deletes reference to a downscale trip. The main steam line radiation monitors currently initiate a main steam line isolation upon detecting high radiation in the main steam lines or when the monitors are inoperable. The two trip channels for each monitor are sufficient to meet the design intent of the main steam line radiation monitoring system. The downscale trip would not provide any plant protective function since a downscale reading by a radiation monitor would indicate a decrease in main steam line radiation and would not necessitate isolation of the main steam lines.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG NUMBER	DOCUMENT EVALUATED	TITLE
88-0220	USAR 9.2.5.3	REVISE AUXILIARY HEAT LOAD TO THE ULTIMATE HEAT SINK

This USAR change revises the auxiliary heat load to which the ultimate heat sink is subjected during a postulated loss of coolant accident. This heat load is being revised to be consistent with the heat load specified in USAR Table 9.2-3. The revised auxiliary heat load is still less than the heat load used in the design basis of the ultimate heat sink.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0099	USAR 9.4.6.1.1	DELETE OPTION FOR REMOVAL OF TRAVEL STOPS
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Several isolation valves in the containment ventilation (VR) system and the drywell purge (VQ) system have travel stops installed in them which limit their travel to 50 degrees in the open direction. These travel stops were originally intended to be removed during plant operating modes 4 and 5 to provide increased containment building ventilation. This USAR change deletes an option of removing the travel stops during plant operating modes 4 and 5. The design flow rates in the VR and VQ system can still be achieved while operating the systems with the travel stops in place.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG NUMBER	DOCUMENT EVALUATED	TITLE
88-0137	USAR 9.5.2.2.1	REVISE COVERAGE PROVIDED BY PUBLIC ADDRESS SYSTEM

This USAR change revises the description of the public address (PA) system tone generator to state that the sound level produced by the tone generator is 75 to 130 decibels at locations where personnel are assigned work tasks over a period of time exceeding a normal work week. Also, for the remainder of the plant, alarms and emergency messages will be provided by a combination of the existing PA system, alternate communication devices, and general training practices. This change clarifies the extent of coverage intended to be provided by the PA system and maintains the applicable commitments in the Emergency Plan.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0275 USAR TAB 3.11-5 REVISION TO ENVIRONMENTAL ZONE SUMMARY

This USAR change revises the upper and lower temperature ranges for several of the environmental zones listed in USAR Table 3.11-5. These changes reflect the temperatures which are achievable with the installed plant ventilation equipment. The revised upper temperature range required the revision of environmental qualification packages. The revised lower temperature ranges do not effect equipment qualification.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0141 USAR TAB 3.8-5 REVISIONS TO CONTAINMENT PENETRATION DESCRIPTIONS

This USAR change revises tables and figures to reflect the as-built configuration of mechanical and electrical containment penetrations. These changes correct items such as penetration descriptions, penetration elevations, sleeve sizes, and pipe sizes. The design bases for the containment penetrations are not changed by these USAR changes.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0292 USAR TAB 3.9-5 REVISE LIST OF ACTIVE VALVES AND PUMPS

This USAR change adds fuel pool cooling and cleanup (FC) system relief valve 1FC091 to the list of valves and pumps that are classified as active safety related. This relief valve provides overpressurization protection for piping in the FC system. This valve has been seismically and environmentally qualified.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0141 USAR TAB 3.8-5 REVISIONS TO CONTAINMENT PENETRATION DESCRIPTIONS

This USAR change revises tables and figures to reflect the as-built configuration of mechanical and electrical containment penetrations. These changes correct items such as penetration descriptions, penetration elevations, sleeve sizes, and pipe sizes. The design bases for the containment penetrations are not changed by these USAR changes.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

88-0292 USAR TAB 3.9-5 REVISE LIST OF ACTIVE VALVES AND PUMPS

This USAR change adds fuel pool cooling and cleanup (FC) system relief valve 1FC091 to the list of valves and pumps that are classified as active safety related. This relief valve provides overpressurization protection for piping in the FC system. This valve has been seismically and environmentally qualified.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

10CFR50.59 REPORT FROM APRIL THROUGH DECEMBER 1988

LOG DOCUMENT
NUMBER EVALUATED

TITLE

88-0114 USAR TAB 6.2-47 REVISE DESCRIPTION OF RI SYSTEM ISOLATION SIGNAL

USAR Table 6.2-47 indicates that the reactor core isolation cooling (RI) system suppression pool suction valve (1E51-F031) and the RI system outboard containment steam supply valve (1E51-F064) receive an isolation signal when the water level in the reactor drops below level 2. Per the system design, this isolation signal does not isolate these valves, but provides a sealed-in signal which is used as a permissive to isolate valves 1E51-F031 and 1E51-F064 in conjunction with an RI system manual isolation. This USAR change adds a note to Table 6.2-47 to explain this feature. The current USAR description of this isolation signal conflicts with the design intent of the RI system, which is to automatically start when the water level in the reactor drops below level 2.

AS A RESULT OF THE EVALUATION, IT WAS DETERMINED THAT AN UNREVIEWED SAFETY QUESTION DID NOT EXIST.

U-601386
L02-89(03-02)-LP
1A.120

ILLINOIS POWER COMPANY



CLINTON POWER STATION, P.O. BOX 678, CLINTON, ILLINOIS 61727

March 2, 1989

10CFR50.36
10CFR50.59

PRIORITY ROUTING	
First	Second
RA	PC
ORA	EC
✓ RP	Q&S SGA
✓ RS	QA ML
✓ DRSS	OL
✓ ORMA	OL
	RAO

FILE 705

OR0+1

Docket No. 50-461

Mr. A. Bert Davis
Regional Administrator
Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Subject: Clinton Power Station
Annual Operating Report
NRC Docket No. 50-461, NPF-62

Dear Mr. Davis:

In accordance with 10CFR50.36 and paragraph 6.9.1.4 of the Clinton Power Station Technical Specifications, Illinois Power Company submits the Clinton Power Station Annual Operating Report for the period of January 1, 1988, through December 31, 1988. In accordance with 10CFR50.59(b)(2), the summary of Safety Evaluations for 1988 (April to December) is included. The Safety Evaluation summary submittal (U-601279) for the period January 1 to March 31 was previously submitted with the Updated Safety Analysis Report.

Should you have any questions or comments regarding this report, please contact me.

Sincerely yours,

D. L. Holtzscher
Acting Manager -
Licensing and Safety

SFB/pgc

Attachments

cc: NRC Clinton Licensing Project Manager
NRC Resident Inspector
NRC Document Control Desk
Illinois Department of Nuclear Safety

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