

WILLIAM T. COTTLE Vice President Nuclear Operations

August 21, 1989

U.S. Nuclear Regulatory Commission Mail Station P1-137 Washington, D.C. 20555

Attention: Document Control Desk

Gentlemen:

SUBJECT: Grand Gulf Nuclear Station

Unit 1

Docket No. 50-416 License No. NPF-29 Reactor Scram Caused by

Lightning Strike LER 89-010-00 AECM-89/0155

Attached is Licensee Event Report (LER) 89-010-00 which is an interim report.

Yours truly,

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WTC:cg Attachment

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JEZZ 1.

Attachment to AECM-89/0155 U.S. MUCLEAR REGULATORY COMMISSION APPROVED DIME NO. 3180-0104 EXPIRES 8/31/86 LICENSEE EVENT REPORT (LER) DOCKET NUMBER (2) FACILITY NAME (1) Grand Gulf Nuclear Station OF 014 Unit 1 0 15 10 10 10 14 11 16 Reactor Scram Caused by Lightning Strike EVENT DATE (6) LER NUMBER (6) OTHER SACULTIES INVOLVED (8) REPORT DATE (7) SEQUENTIAL DOCKET NUMBERIS FACILITY NAMES MONTH DAY YEAR YEAR MONTH DAY YEAR 0 | 5 | 0 | 0 | 0 | 0 7 2 2 8 9 8 9 01 0 0 0 d 8 2 1 8 9 0 15 10 10 10 1 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR \$: (Check one or many or the following) (11) OPERATING 73.71(b) 26.403(b) 20.405(a) 60.73(a)(2)(iv) 26.406(a)(1)(f) 90.38(a)(1) 80 73(n)(2)(v) 72 72 (a) 1,0,0 OTHER (Specify in Abstract below and in Text, NRC Form 366A) 20.4064()(11(8) 60.73(a)(2)(vii) 80.38(a)(2) 26.406(a)(1)(NI) 80.73(e)(2)(i) 80.73(e)(2)(viii)(A) 20,408 to)(1)(lv) 80.73(a)(2)(N) \$0.73(e)(2)(viii)(8) 20.406(a)(1)(v) 80.736((2)((()) 80 73(a)(2)(u) LICENSEE CONTACT FOR THIS LER (12)

Ronald Byrd / Licensing Engineer

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

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YES III you, semplote EXPECTED SUBMISSION DATE!

On July 22, 1989, a severe electrical storm passed over Grand Gulf Nuclear Station. During the storm, the reactor automatically scrammed due to a high neutron flux signal on the Average Power Range Monitors (APRMs). Additionally, a spike to the Reactor Core Isolation Cooling (RCIC) system logic caused RCIC to automatically actuate and inject into the reactor vessel. Reactor water level decreased to -15 inches and was raised to the level 8 high level trip (+53.5 inches) in approximately 2 minutes.

A channel check of APRM indications was performed during plant restart on July 23, 1989 to confirm proper response. No abnormalities were observed. System Energy requisitioned a specialist in lightning protection to perform a plant survey and study of the existing plant lightning protection system. The contractor has submitted proposals to provide lightning dissipation arrays on vulnerable structures. System Energy is presently evaluating implementation methods and constructing a schedule for implementation. Current progress indicates that implementation will likely be completed by December 31, 1989. In any case, implementation will be completed no later than the startup from the fourth refueling outage (RFO4).

PCOM LEP 89-010-00 -

MRC Form 36E

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NAC Form 2004 (15-63) LICENSEE EVI	ENT REPORT (LER) TEXT CONTINU	IATION APPROVED O	U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/96						
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A. Reportable Occurrence

TEXT IF more assess is required: use additional NRC Form 386A's/ (17)

On July 22, 1989, the reactor automatically scrammed due to a lightning induced spike on the Average Power Range Monitors (APRMs). This Reactor Protection System (RPS) actuation is reported pursuant to 10CFR50.73(a)(2)(iv).

B. Initial Conditions

The plant was operating at approximately 100 percent power at the time of occurrence,

C. Description of Occurrence

On July 22, 1989, a severe electrical storm passed over Grand Gulf Nuclear Station. At 1723 during the storm, the reactor automatically scrammed due to a high neutron flux signal on the APRMs (EIIS code: IG). All APRM channel upscale alarms annunciated and immediately cleared. Evaluation of the event revealed that the APRM high neutron flux signal lasted less than one-tenth of a second. Additionally, a spike to the Reactor Core Isolation Cooling (RCIC) system (EIIS code: BN) logic, simulated a low water level signal and caused RCIC to automatically actuate.

Following the scram, the reactor water level decreased to approximately -15 inches but was raised to level 8 (+53.5 inches) at 1725 by the RCIC injection. Prior to reaching level 8, operators decreased RCIC injection flow and secured reactor feed pump "B". When level 8 was reached, the RCIC injection valve automatically closed and the "A" Reactor Feed Pump Turbine tripped as designed. The reactor water level decreased below level 8 at 1731. Reactor Feed Pump "A" was restarted at 1734.

D. Apparent Cause

The RPS actuation and subsequent reactor scram were caused by an initiation signal from 7 of 8 APRM channels. The trip function of the APRMs is set to occur when core thermal power reaches 118 percent instantaneously. Review of computer traces indicate that channel B spiked to approximately 118 percent. Considering the accuracy of the traces, it is concluded that the trip setpoint for channel B was not reached.

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The approximate peak indicated flux for each channel is as follows:

APRM	Channel	Peak	Indic
	A		126%
	В		118%
	C		126%
	D		140%
	E		123%
	F		123%
	G		134%
	H		140%

The Post Trip Analysis concluded that the cause of the instrumentation signal spikes was an induced voltage and/or ground potential spike caused by lightning activity at the site.

A previous similar event was reported in LER 88-012. The cause of the APRM spikes for that event was determined to be a fence grounding strap that was routed in close proximity to conduit containing Division 3 and 4 APRM signal cables. The grounding straps for the fence were relocated and evaluations for final resolution continued. Final resolution had not been completed at the time of the July 22, 1989 occurrence.

E. Supplemental Corrective Actions

A channel check of APRM indications was performed during plant restart on July 23, 1989 to confirm proper response. No abnormalities were observed.

The fence located on the roof of the Control Building and Turbine Building was dismantled as a precautionary measure. An inspection of the fence revealed no evidence of it having been struck by lightning.

System Energy requisitioned a specialist in lightning protection to perform a plant survey and study of the existing plant lightning protection system. The contractor has submitted proposals to provide lightning dissipation arrays on vulnerable structures. System Energy is presently evaluating implementation methods and constructing a schedule for implementation. Current progress indicates that implementation will likely be completed by December 31, 1989. In any case, implementation will be completed no later than the startup from the fourth refueling outage (RFO4).

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F. Safety Assessment

The Post Trip Analysis confirmed that all safety systems functioned properly and that plant response to these automatic actions was as expected. RPS response times were satisfactory when compared to expected or required times. Reactor water level remained at least 151 inches above the top of active fuel during the event. All Emergency Core Cooling Systems were operable but were not required to be automatically or manually initiated.