

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

OREGON STATE UNIVERSITY

DOCKET NO. 50-243

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 10 License No. R-106

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to Facility Operating License No. R-106 filed by Oregon State University (the licensee), dated June 14, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied; and
 - F. Publication of notice of this amendment is not required since it does not involve a significant hazards consideration no. amendment of a license of the type described in 10 CFR Section 2.106(a)(2).

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- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C.(2) of License No. R-106 is hereby amended to read as follows:
 - (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 10, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Seymour H. Weiss

Seymour H. Weiss, Director Non-Power Reactor, Decommissioning and Environmental Project Directorate Division of Reactor Projects III, IV, V and Special Projects Office of Nuclear Reactor Regulation

Enclosure: Appendix A Technical Specifications Changes

Date of Issuance: August 15, 1989

ENCLOSURE TO LICENSE AMENDMENT NO. 10

FACILITY OPERATING LICENSE NO. R-106

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Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Remove Pages	Insert Pages
34	34
35	35

- g. Offsite environmental monitoring surveys.
- h. Fuel inventories and transfers.
- Facility radiation and contamination surveys.
- j. Radiation exposures for all personnel.
- k. Updated, corrected, and as-built drawings of the facility.

6.7 REPORTING REQUIREMENTS

In addition to the requirements of applicable regulations, and in no way substituting therefore, reports shall be made to the NRC as follows:

- a. A report within 24 hours by telephone or telegraph to the NRC, Region V, of:
 - Any accidental release of radioactivity above applicable limits in unrestricted areas, whether or not the release resulted in property damage, personal injury, or exposure;
 - Any violation of a safety limit;
 - Operation with a safety system setting less conservative than specified in Section 2.2, Limiting Safety System Settings;
 - Operation in violation of a Limiting Condition for Operation;
 - Failure of a required reactor or experiment safety system component which could render the system incapable of performing its intended safety function unless the failure is discovered during maintenance tests or periods of reactor shutdown;
 - Any unanticipated or uncontrolled change in reactivity greater than \$1.00;
 - 7. An observed inadequacy in the implementation of either administrative or procedural controls, such that the inadequacy could have caused the existence or development of a condition which could result in operation of the reactor outside the specified safety limits; and
 - 8. A measurable release of fission products from a fuel element.
- b. A report within 14 days in writing to the NRC, Document Control Desk, Washington, D.C., with a copy to the NRC, Region V.
 - Any accidental release of radioactivity above permissible limits in unrestricted areas, whether or not the release resulted in property damage, personal injury, or exposure; the written report (and, to the extent possible, the preliminary telephone or telegraph report) shall describe, analyze, and evaluate safety implications, and outline the corrective measures taken or planned to prevent reoccurrence of the event;

- Those events reported as required by Sections 6.7.a.2 through 6.7.a.8.
- b. A report within 30 days in writing to the NRC, Document Control Desk, Washington, D.C., with a copy to the NRC, Region V.
 - Any significant variation of measured values from a corresponding predicted or previously measured value of safety-connected operating characteristics occurring during operation of the reactor;
 - Any significant change in the transient or accident analyses as described in the Safety Analysis Report;
 - 3. Any changes in facility organization or personnel; and
 - Any observed inadequacies in the implementation of administrative or procedural controls such that the inadequacy causes or could have caused the existence or development of an unsafe condition with regard to reactor operations.
- d. A report within 90 days after completion of starting testing of the reactor (in writing to the NRC, Document Control Desk, Washington, D.C. and a copy to NRC, Region V) upon receipt of a new facility license, or an amendment to the license authorizing an increase in reactor power level, describing the measured values of the operating conditions or characteristics of the reactor under the new conditions including:
 - An evaluation of facility performance to date in comparison with design predictions and specifications.
 - A reassessment of the safety analysis submitted with the license application in light of measured operating characteristics when such measurements indicate that there may be substantial variance from prior analysis.
- e. An annual report within 75 days following the 30th of June of each year (in writing to the NRC, Document Control Desk, Washington, D.C. and a copy to the NRC, Region V).
 - A brief summary of operating experience including experiments performed and changes in facility design, performance characteristics and operating procedures related to reactor safety occurring during the reporting period, and results of surveillance test and inspections.
 - A tabulation showing the energy generated by the reactor (in megawatt-hours), hours reactor was critical, and the cumulative total energy output since initial criticality.
 - The number of emergency shutdowns and inadvertent scrams, including reasons therefore.
 - Discussion of the major maintenance operations performed during the period, including the effect, if any, on the safety of the operation of the reactor and the reasons for any corrective maintenance required.