

ENCLOSURE 1

NOTICE OF VIOLATION

System Energy Resources, Inc.  
Grand Gulf

Docket No. 50-416  
License No. NPF-29

During the Nuclear Regulatory Commission (NRC) inspection conducted on May 20 through June 16, 1989, violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C (1989), the violations are listed below:

- A. 10 CFR 50, Appendix B, Criteria XVI, Corrective Action, measures shall be established to assure that conditions adverse to quality, such as failures, malfunctions, deficiencies, deviations, defective material and equipment, and nonconformances are promptly identified and corrected. The measures shall assure that the cause of the condition is determined and corrective action taken to preclude repetition.

Contrary to the above, on April 29 and May 8, 1989, Reactor Core Isolation Cooling (RCIC) System isolated on a high steam line flow. The RCIC system isolated twice on high steam line flow in December 1984. The corrective action was to adjust the transmitter damping circuits, however the licensee failed to procedurally control these settings which resulted in system isolation when the damping setting were not made to new circuit boards installed during the last refueling outage.

This is a Severity Level IV violation (Supplement I).

- B. 10 CFR 50, Appendix B, Criteria III, Design Control, requires measures shall provide for verifying or checking the adequacy of design, such as by the performance of design reviews, or by the performance of a suitable testing program. Design changes including field changes shall be subject to design control measures commensurate with those applied to the original design.

Technical Specification 6.8.3.c, states, a post-accident sampling program shall be established, implemented and maintained which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluent, and containment atmosphere samples under accident conditions. The program shall include training of personnel, procedures for sampling and analysis, and provisions for maintenance of sampling and analysis equipment.

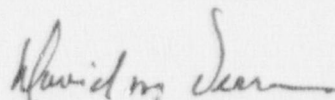
Contrary to the above, the licensee failed to adequately review, implement and test Design Change Package 87-4018, Post Accident Sample (PAS) system atmospheric and liquid panel modification. This resulted in the PAS system being incapable of being used to take a liquid radionuclide accident sample from April 27 to June 7, 1989. Additionally, the system drawing

was revised and did not reflect as build conditions. The system operators and engineer were inadequately trained in the proper operation of the system in that they believed the system could only operate under full reactor system pressure. The sampling program was inadequate in that required semiannual samples for RHR and suppression pool were not taken from January 1986 to June 1989 and RHR B and drywell sample paths were never demonstrated.

This is a Severity Level IV violation (Supplement I).

Pursuant to the provisions of 10 CFR 2.201, System Energy Resources, Inc. is hereby required to submit a written statement or explanation to the Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555, with a copy to the Regional Administrator, Region II: and a copy to the NRC Resident Inspector, System Energy Resources, Inc within 30 days of the date of the letter transmitting this Notice. This reply should be clearly marked as a "Reply to a Notice of Violation" and should include <sup>2</sup>for each violation<sup>3</sup>: (1) admission or denial of the violation, (2) the reason for the violation if admitted, (3) the corrective steps which have been taken and the results achieved, (4) the corrective steps which will be taken to avoid further violations, and (5) the date when full compliance will be achieved. Where good cause is shown, consideration will be given to extending the response time. If an adequate reply is not received within the time specified in this Notice, an order may be issued to show cause why the license should not be modified, suspended, or revoked or why such other action as may be proper should not be taken.

FOR THE NUCLEAR REGULATORY COMMISSION



David M. Verrelli, Chief  
Reactor Projects Branch 1  
Division of Reactor Projects

Dated at Atlanta, Georgia  
this 27<sup>th</sup> day of June 1989