



TU ELECTRIC

Log # TXX-89441
File # 10010
915

June 30, 1989

William J. Cahill, Jr.
Executive Vice President

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)
DOCKET NOS. 50-441 AND 50-446
SUBMITTAL OF VIPRE-01 CORE THERMAL-HYDRAULIC ANALYSIS
METHODS TOPICAL REPORT RXE-89-002

- REF: 1. Letter from W. J. Cahill, (TU Electric) to USNRC,
TXX-89047, dated January 31, 1989
2. Letter from C. E. Rossi (NRC) to J. A. Blaisdell (UGRA),
"Acceptance for Referencing of Licensing Topical Report, EPRI
NP-2511-CCM, 'VIPRE-01: A Thermal Hydraulic Analysis Code for
Reactor Cores; Volumes 1, 2, 3 and 4,'" dated May 1, 1986.
3. NRC Generic Letter 83-11, February 8, 1983.

Gentlemen:

TU Electric has developed the capability to perform core thermal-hydraulic analyses using the VIPRE-01 computer code. The VIPRE-01 analysis capability represents one component of the complete TU Electric reload analysis program described in Reference 1. TU Electric intends to use this capability to perform Departure from Nucleate Boiling (DNB) Calculations for core safety limit evaluations, setpoint analyses, and safety analyses for normal operation and plant transients.

The VIPRE-01 code has been approved previously by the NRC (Reference 2) for PWR licensing applications.

The enclosed two copies of TU Electric Topical Report No. RXE-89-002 are submitted to satisfy the VIPRE-01 model documentation and user qualification requirements stated in References 2 and 3. Your timely review of this topical report is appreciated.

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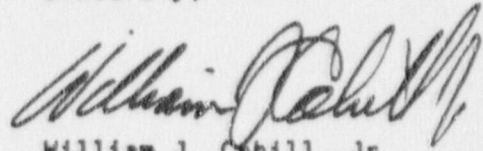
400 North Olive Street LB 81 Dallas, Texas 75201

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If you have any questions on this material, please contact Steve Maier or Ray Ashley of my staff.

Sincerely,

A handwritten signature in cursive script, appearing to read "William J. Cahill, Jr.", written in dark ink.

William J. Cahill, Jr.

RLA/ddm
Enclosures (2)

c - Mr. R. D. Martin, Region IV
Resident Inspectors, CPSES (3)



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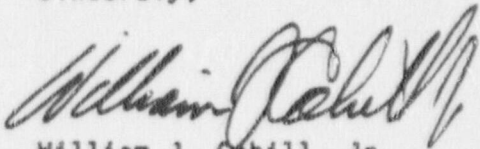
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