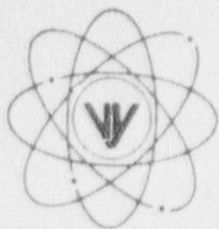


VERMONT YANKEE NUCLEAR POWER CORPORATION



Ferry Road, Brattleboro, VT 05301-7002

REPLY TO
ENGINEERING OFFICE

580 MAIN STREET
BOLTON, MA 01740
(508) 779-6711

June 28, 1989
BVY 89-59

United States Nuclear Regulatory Commission
Washington, DC 20555

Attention: Document Control Desk

Reference: License No. DPR-28 (Docket No. 50-271).

Subject: Request to Routinely Dispose of Slightly Contaminated Septic
Waste in Accordance with 10 CFR 20.302(a)

Dear Sir:

In accordance with the criteria of the Code of Federal Regulations, Title 10, Section 20.302(a) (10CFR20.302(a)), enclosed please find the subject application for the disposal of very low level radioactive waste materials. Vermont Yankee Nuclear Power Corporation (Vermont Yankee) hereby requests NRC approval of the proposed procedures for the disposal of slightly contaminated septic waste generated at the Vermont Yankee Nuclear Power Plant in Vernon, Vermont.

This application specifically requests approval to dispose of septic tank waste, contaminated at minimal levels, which have been or might be generated through the end of station operations at the Vermont Yankee Nuclear Power Plant. The proposed method of disposal is for the on-site land spreading in designated areas in compliance with State of Vermont health code requirements for septic waste. Disposal of this waste in the manner proposed, rather than at a 10CFR Part 61 licensed facility would save Vermont Yankee not only substantial cost, but also valuable disposal site space which would then be available for wastes of higher radioactivity levels. Disposal as radioactive waste would require treatment of the biological aspects of the septage and solidification to a stable waste form, thereby increasing the volume substantially.

A radiological assessment and proposed operational controls, based upon the continued on-site disposal of septic waste as presently contained in the plant's septic tanks, are detailed in Attachments 1 and 2. Based upon this analysis, Vermont Yankee requests approval to dispose of septic tank waste on-site by land spreading in such a manner that the radioactivity concentration limit in any batch of septage to be spread does not exceed one-tenth of the MPC values listed in 10CFR20, Appendix B, Table II; and the combined radiological impact for all disposal operations shall be limited to a total body or organ dose of a maximally exposed member of the public of less than one mrem/year (less than 5 mrem/year to an inadvertent intruder).

8907050464 890628
PDR ADDCK 05000271
P PNU

A009
1/1

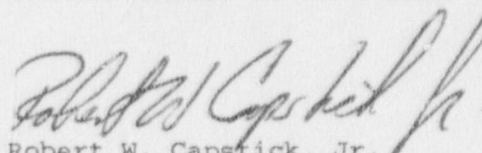
United States Nuclear Regulatory Commission
June 28, 1989
Page 2

Due to our expected need to utilize the proposed methodology of land application of septic waste on-site during the spring of 1990, we request your review and approval of this proposed disposal method by the end of the first quarter of 1990.

We trust that the information contained in the submittal is sufficient; however, should you have any questions or require further information concerning this matter, please contact this office

Very truly yours,

VERMONT YANKEE NUCLEAR POWER CORPORATION



Robert W. Capstick, Jr.
Licensing Engineer

MSS/emd

Enclosures

cc: USNRC - Region I
USNRC - Resident Inspector, VTNPS

ATTACHMENT 1

VERMONT YANKEE NUCLEAR POWER PLANT

APPLICATION FOR APPROVAL TO ROUTINELY DISPOSE OF
SEPTIC WASTE WITH MINIMAL LEVELS OF RADIOACTIVITY

ATTACHMENT 1

VERMONT YANKEE NUCLEAR POWER PLANT

Application for Approval to Routinely Dispose of Septic Waste With Minimal Levels of Radioactivity

1.0 INTRODUCTION

Vermont Yankee Nuclear Power Corporation (Vermont Yankee) requests approval, pursuant to 10CFR20.302(a), of a method proposed herein for the routine disposal of slightly contaminated septic tank waste. Vermont Yankee proposes to dispose of this waste by spreading it on designated areas within the plant's site boundary fence. This application addresses specific information requested in 10CFR20.302(a).

2.0 WASTE STREAM DESCRIPTION

The waste involved in this application consists of residual solids and water associated with the sewage collection system at Vermont Yankee. The plant's sewage systems are of the septic tank and disposal field type. The two systems servicing the majority of the plant's sanitary waste are identified as (1) main septic system and (2) the south sewage disposal system.

The main septic system (design flow capacity 4,950 gallons/day) consists of a wastewater lift station, septic tank, and dual alternating disposal fields located on the north side of the plant. This system services the main complex of buildings central to the plant and processes approximately 3,500 gallons of wastewater per day. The septic tank, shown in Figure (1), will typically contain 9,250 gallons of septage.

The south sewage disposal system is a newly-installed (January 1989) pressurized mound system, which is used in lieu of the construction office building (COB) holding tank that had previously serviced the lavatory facilities on the south end of the plant. The new system is composed of a septic tank (5,700 gallon capacity, see Figure 2), pumping station, and pressurized mound disposal field. When dosing the field, a force main pressurizes the disposal field's piping system with the septic tank effluent, which distributes throughout the field. The south sewage disposal system has

the design flow capacity to process 4,607 gallons of wastewater per day. The system is typically loaded at approximately 2,500 gallons per day during normal plant operations. Figure (3) indicates diagrammatically the flow of both potable and wastewater throughout Vermont Yankee.

Both the main septic system and the south sewage disposal system's septic tanks collect waste from the plant's lavatories, showers, kitchens, and janitorial facilities outside the Radiological Control Area (RCA). No radioactivity is intentionally discharged to either of the septic systems. However, plant investigations into the source of low levels of contamination found in septic waste have identified that very small quantities of radioactive materials, which are below detection limits for radioactivity releases from the RCA, are carried out of the control area on individuals and accumulate in the septic waste collection tanks by way of floor wash water, showers, and hand washing. As a means of minimizing the transport of radioactive materials into the septic collection tanks, the primary source of the radioactivity (i.e., floor wash water) is now poured through a filter bag to remove suspended solids and dirt before the water is released into a janitorial sink.

The majority of the radioactivity found in waste sludge has been associated with the main septic tank. Grab samples of sludge from the bottom of the COB and main septic tank were analyzed by gamma spectroscopy with the following results of plant-related radionuclides:

	<u>Isotope</u>	<u>Activity Concentration ±1 Sigma (pCi/kg Wet)</u>
COB Sludge (June 8, 1988)	Cs-137	10.3 ± 1.8
	Co-60	45.4 ± 3.1
Main Tank Sludge (June 8, 1988)	Mn-54	39.3 ± 4.3
	Co-60	853.0 ± 12.0
	Zn-65	52.7 ± 8.2
	Cs-134	13.0 ± 2.2
	Cs-137	120.7 ± 5.2

The principle radionuclide is Cobalt-60, which accounts for 79% of the plant related activity in the septage samples. In comparison to in-plant smear samples taken for 10CFR61 waste characterizations, the septage sample from the main tank correlates very close with the distribution of radionuclides identified in-plant as shown below:

Relative Isotopic Distributions

<u>Isotope</u>	<u>In-Plant Smears</u>	<u>Main Tank Sludge</u>
Mn-54	3.6%	3.6%
Co-60	81.5	79.1
Zn-65	3.8	4.9
Cs-134	0.4	1.2
Cs-137	10.3	11.2

Additional analyses of the main tank septage showed that the liquid portion of the collected sample did not contain any plant-related activation or fission products, and that essentially all of the activity in the waste was associated with the solid sludge fraction. The average density of the collected sludge was found to be approximately equal to that of water, with a wet to dry ratio of 25.4 to 1.

Both the liquid and solid fractions of the main tank septic waste were also analyzed for strontium with no detectable activity found. The liquid portion of the waste sample was also analyzed for tritium with no activity above the minimum detectable levels found. Appendix A to Attachment 2 contains the laboratory analysis reports of the samples taken from the COB and main septic tanks.

Prior to identification of the plant-related radioactivity in septage waste, the COB holding tank was being pumped on the average of twice per week, with the sludge and waste liquid transported off-site primarily to the Brattleboro, Vermont, sewage treatment facility. Waste from the main septic tank was being pumped and transported off-site for disposal on the average of twice per year.

With the replacement of the COB holding tank by the new south sewage disposal system, and the requested implementation of on-site land disposal of accumulated septic waste, the frequency of collection tank pump-outs with land application of the waste is expected to be once per year. With the past pump-out frequency of the main tank being every six months, the accumulation of sludge at the bottom of the tank was well below its design capacity. During the 1988 sample collections, it was estimated that the sludge thickness was less than 1 foot of its 6-foot depth. However, for conservatism in the radiological evaluations, it is assumed that the sludge layer in the main septic tank and south disposal tank occupies 30% of their combined design volume, and that the frequency of pump-outs is semiannual as opposed to the expected annual cycle. Also, as noted above from laboratory analyses of the sludge layer taken from the bottom of the main tank, the average density of the tank contents is approximately equal to that of water, with a wet-to-dry ratio of 25.4 to 1. Hence, the weight of solids (W_{sol}) being disposed of is estimated, for purposes of this bounding dose assessment, to be approximately:

$$\begin{aligned} W_{sol} &= 14,950 \text{ [gal]} \times 3,785.4 \text{ [cc/gal]} \times 10^{-3} \text{ [kg/cc]} \\ &\quad \times 0.30 \text{ [solids fraction]} \times (1/25.4) \text{ [dry/wet ratio]} \\ &\sim 700 \text{ [kg]} \text{ per pump-out of both tanks} \end{aligned}$$

or, 1,400 kg of dry solids per year.

3.0 DISPOSAL METHOD

Approval of this application will allow Vermont Yankee to dispose of septage by utilization of a technique of land spreading or surface injection in a manner consistent with all applicable state of Vermont health regulations regarding disposal of septic waste. Details of the chemical and biological controls necessary to satisfy state health code requirements are provided in Reference 5.

The septage will be spread or surface injected on land areas owned by Vermont Yankee and situated within the plant's site boundary. Transportation of the septage waste to the disposal areas will involve pumping from one of the septic waste collection tanks (i.e., main septic tank, COB holding tank,

new replacement COB septic tank, or from any other on-site septic waste collection point) into an enclosed truck-mounted tank. The enclosed tank truck is used to prevent spillage while in transit to the disposal areas. The septage will be transported to one of the two disposal sites designated for land application for septage from Vermont Yankee, and applied at a fixed rate based on either limitations imposed by the state of Vermont for heavy metals or organic content of the waste, or on the radioactivity content such that projected maximum individual doses will not exceed established dose objectives.

3.1 Septic Waste Disposal Procedure

Gamma isotopic analysis of septic waste shall be made prior to each disposal by obtaining a representative sample from each tank prior to pump-out. At least two septic waste samples will be collected from each tank to be pumped by taking a volumetric column of sludge and waste water which allows for analysis of the solid's distribution and content from top to bottom of each tank. The weight percent of solid content of the collected waste will be determined and applied to the gamma isotopic analysis in order to estimate the total radioactivity content of each tank to be pumped and spread on designated disposal fields.

These gamma isotopic analyses of the representative samples will be performed at the environmental Technical Specification lower limit of detection (LLD) requirements for liquids (see Technical Specification Table 4.9.3) in order to document the estimation of radiological impact from septage disposal.

The radionuclide concentrations and total radioactivity identified in the septage will be compared to the concentration and total curie limits established herein prior to disposal. The methodology and limits associated with determining compliance with the disposal dose and activity criteria are described in Attachment 2. If the concentration and total activity limits are met, compliance with the dose assessment criteria will have been demonstrated since the radiological analysis (Section 4.5 and Attachment 2) was based on evaluating the exposure to a maximally exposed individual and inadvertent intruder after the accumulation of twenty years of periodic semiannual

spreading of the septic waste on a single (2 acre) plot within one of the designated disposal areas. If the activity limit per disposal area is projected to be exceeded, the appropriate exposure pathways as described in Section 4.5 will be evaluated prior to each additional application, or a separate plot within the designated disposal area will be utilized.

Annually, for years in which disposal occurs, the potential dose impact from disposal operations conducted during the year, including the impact from previous years, will be performed and results reported in the plant's Semiannual Radioactive Effluent Release Report which is filed after January 1. All exposures will be assessed utilizing the methodology described in Attachment 2.

The established dose criteria requires that all applications of septage within the approved designated disposal areas shall be limited to ensure the dose to a maximally-exposed individual be maintained less than 1 mrem/year to the whole body and any organ, and the dose to the inadvertent intruder be maintained less than 5 mrem/year. The total activity based on the measured radionuclide distribution for any single disposal plot is not expected to exceed the following:

Isotope	Maximum Accumulated Radioactivity Allowed Per Acre
	Q_d^{lim} [μ Ci]
Mn-54	1.4
Co-60	120.0
Zn-65	1.4
Cs-134	0.7
Cs-137	46.5

If any of the above radionuclides are projected to exceed the indicated activity values, then dose calculations will be performed prior to spreading, in accordance with the methods detailed in Section 4.2.2 of Attachment 2, to make the determination that the dose limit criteria will not be exceeded.

The concentration of radionuclides in any tank of septic waste to be disposed of will also be limited to a combined Maximum Permissible Concentration of Water (MPC) (as listed in 10CFR, Part 20, Appendix B, Table II, Column 2) ratio of less than or equal to 0.1.

For radiological control, each application of septage will be applied on the designated land area by approved plant procedure which adheres to the following assumptions which were used in developing the dose impact:

- o During surface spreading or injection, the septage, and any precipitation falling onto or flowing onto the disposal field, shall not overflow the perimeter of the designated area.
- o Septage shall not be surface spread or injected into the top 6-inch soil layer within 300 feet from any drinking water well supply.
- o Septage shall not be surface spread closer than 300 feet from the nearest dwelling or public building (or within 100 feet if injected into the top 6-inch surface layer).
- o Septage shall not be surface spread closer than 50 feet (or within 25 feet if injected into the top 6-inch surface layer) from any roads or site boundary adjacent to land areas.
- o Septage shall not be surface spread within 100 feet (or within 50 feet if injected into the top 6-inch surface layer) of any surface water (rivers, streams, drainage ditches).
- o Low areas of the approved fields, subject to seasonally high groundwater levels, are excluded from the septage application.

In addition to the radiological controls to limit the total accumulation of radioactive materials released by septic waste spreading, state of Vermont health code requirements will be followed to ensure the protection of the public and environment from chemical and biological hazards. The application rate and acreage will be determined prior to each

disposal operation. This will vary with the chemical composition of the septage, the percent solids, and the radioactive concentrations.

3.2 Administrative Procedures

Complete records of each disposal will be maintained. These records will include the concentration of radionuclides in the septage, the total volume of septic waste disposed, the total activity in each batch as well as total accumulated on the disposal plot at time of spreading, the plot on which the septage was applied, and the results of any dose calculations required.

The annual disposal of septage on each of the approved plot areas will be limited to within the established dose, activity, and concentration criteria noted above, in addition to limitations dictated by chemical and biological conditions. Dose guidelines, and concentration and activity limits, will be maintained within the appropriate values as detailed in Attachment 2.

Any farmer using land which has been used for the disposal of septic waste will be notified of any applicable restrictions placed on the site due to the land spreading or injection of waste.

4.0 EVALUATION OF ENVIRONMENTAL IMPACT

4.1 Site Characteristics

4.1.1 Site Topography

The proposed disposal sites consist of two fields located on the Vermont Yankee Nuclear Power Plant site, which is located on the west bank of the Connecticut River in southwestern Vermont at latitude 42 degrees, 47 minutes north and longitude 72 degrees 31 minutes west. Both fields are on plant property within the site boundary and surrounded by a chain link fence.

Site A contains an approximate eight-acre parcel of usable land centered approximately 2,200 feet northwest of the Reactor Building. Site B contains about two acres and is centered approximately 1,700 feet south of the Reactor Building. The usable acreage of both the north and south disposal fields is restricted to those areas which have no slopes greater than five percent to limit surface runoff. A radiological assessment based on the 1988 measured radioactivity concentrations in sludge has determined that a single two-acre plot would be sufficient for the routine disposal of septage for twenty years without exceeding the dose criteria to maximum exposed individual or inadvertent intruder. As a result, the eight-acre field to the northwest could be divided into four disposal plots, with the two-acre site at the south end of the plant site, providing a fifth plot. A portion of the United States Geological Survey topographic map (Brattleboro quadrangle), showing the plant site, is presented in the Final Safety Analysis Report (FSAR) as Figure 2.5-1. A plan map showing the plant site and the disposal sites is given on Figure 4.

The sites are located along a glacial terrace on the west side of the Connecticut River. This terrace extends about 3,000 feet west rising gently and then more abruptly to a higher terrace and then to dissected uplands. Distance to the east from the disposal sites to the river is at least 100 feet if septage is disposed of by surface spreading within the designated areas, or 50 feet if septage is injected directly into the soil.

Relief of the proposed disposal sites is low, with elevation ranging between 250 feet and 265 feet (msl). Mean water surface elevation of the adjacent river is about 220 feet.

The topographic character of the site and surrounding area is compatible with this use. The spreading of septage at these locations will have no effect on the topography of the area.

4.1.2 Site Geology

Profiles of site exploratory borings are shown in the FSAR in Figures 2.5-8 through 2.5-11. Current site characteristics as determined from a recent detailed site investigation can be found in Reference 5.

Composition of surficial materials is compatible with the proposed use of the site for septic waste disposal.

4.2 Area Characteristics

4.2.1 Meteorology

The site area experiences a continental-type climate with some modification due to the marine climate which prevails at the Atlantic seacoast to the east. Annual precipitation averages 43 inches and is fairly evenly distributed in each month of the year.

Potential impacts on septic waste disposal include occasional harsh weather: ice storms, severe thunderstorms, heavy rains due to hurricanes, the possibility of a tornado, and annual snowfall of from 30 to 118 inches per year. In addition, frozen ground can occur for up to 4 months of the year.

Septage spreading will be managed by written procedure such that material which is spread or a mix of that material with precipitation will not overflow the perimeter of the disposal site.

Additional information on meteorology of the site can be found in Section 2.3 of the Final Safety Analysis Report.

4.2.2 Hydrology

Hydrology of the site and local area is tied closely to flow in the adjacent Connecticut River. River flow is controlled by a series of hydroelectric and flood-control dams including the Vernon Dam which is about 3,500 feet downstream of the site.

All local streams drain to the Connecticut River and the site is in the direct path of natural groundwater flow from the local watershed easterly toward the river. Site groundwater level is influenced by both precipitation and changes in the level of ponding of the Connecticut River behind the Vernon Dam due to natural flow or dam operation.

Flood flows on the Connecticut are controlled by numerous dams including five upstream of the site. Elevation of the 100-year flood is about 228 ft (msl); and, thus, well below the elevation of the proposed site which ranges from about 250 to 265 feet (msl). The 100-year flood level is based on information presented in References (1) and (2).

Septage disposal by means of land spreading on the proposed site will have no adverse impact on area hydrology.

Further information about site hydrology is in Section 2.4 of the FSAR.

4.3 Water Usage

4.3.1 Surface Water

The adjacent Connecticut River is used for hydroelectric power, for cooling water for the Vermont Yankee plant, as well as for a variety of recreational purposes such as fishing and boating. The Connecticut River is not used as a potable water supply within 50 miles downstream of the plant.

Locally, water from natural springs are used for domestic and farm purposes. FSAR Table 2.4.5 and Figure 2.4-2 show springs used within a 1-mile radius of the site. FSAR Table 2.4.4 and Figure 2.4-1 show water supplies with surface water sources which are within a ten-mile radius of the site.

There will be no impact on surface water usage or quality as a result of septage disposal due to the required separation distances between surface waters and the disposal plots.

4.3.2 Groundwater

Based on a review of groundwater measurements in various site borings presented in the FSAR and References 3 and 5, an upper estimate of groundwater levels at the plant is about 240 feet. Considering the proximity of the Connecticut River and Vernon Pond, with a mean water surface elevation of 220 feet, this estimate for the groundwater level appears to be reasonable. Given the topography of the proposed disposal sites, it is highly unlikely that the groundwater level will be within 3 feet of the disposal area surface elevation. Prior to each application of septic waste to a disposal plot, the groundwater level in nearby test wells will be determined and no application will be allowed if the groundwater level in the vicinity of the disposal plot is found to be less than 3 feet.

Groundwater provides potable water for public wells as shown in FSAR Table 2.4.5 and Figure 2.4-1. Groundwater flow in the vicinity of the proposed disposal sites is towards the Connecticut River. There are no drinking-water wells located between the site and the river. Therefore, it is highly unlikely that any drinking water wells could be affected by septage disposal. FSAR Figure 2.4-2 and Table 2.4-5 present information on private wells near the plant.

The Vermont Yankee on-site wells provide water for plant use. This supply is routinely monitored for radioactive contamination.

To quantify the impact of septage disposal on the Connecticut River, a conservative groundwater/radionuclide travel time analysis was performed. For an assumed average travel distance of 200 feet from the disposal site to the river, a groundwater travel time of 408 days was estimated from Darcy's Law. This estimate is based on a permeability for the glacial till of 10 gpd/ft^2 , a hydraulic gradient of 0.11 ft/ft, and a soil porosity of 0.3. This analysis conservatively assumed that the septage placed on the ground was immediately available to the groundwater. In practice, a minimum of 3 feet separation between groundwater and the surface will be required at time of application of the septic waste.

Due to ionic adsorption of the radionuclides on solid particles in the groundwater flow regime, most radionuclides travel at only a small fraction of the groundwater velocity. For the radionuclides present in the sludge, retardation coefficients were developed from NUREG/CR-3130 (Reference 4). Retardation coefficients for Co-60, Cs-137, and Cs-134 were directly obtained from NUREG/CR-3130. The coefficients for Zn-65 and Mn-54 were conservatively estimated using NUREG/CR-3130 as a guide. The radionuclides, their half-lives, retardation coefficients, and their travel time to the river are summarized in Table 1.

TABLE 1

Radionuclide Travel Times

<u>Radionuclide</u>	<u>Half Life</u>	<u>Retardation Coefficient</u>	<u>Travel Time to River</u>
Co-60	5.3 years	860	961 years
Cs-137	30.2 years	173	193 years
Cs-134	2.1 years	173	193 years
Zn-65	244 days	3	1,224 days
Mn-54	312 days	3	1,224 days

The radiological impact on the river for the radionuclides reaching the river under this conservative analysis is discussed in Attachment 2. Water usage of the Connecticut River downstream from the disposal area is limited to drinking water for dairy cows, irrigation of vegetable crops, and irrigation of cow and cattle fodder.

Based on the assessments noted above, it is concluded that groundwater sources will not be adversely impacted as a result of septage disposal on the proposed site.

4.4 Land Use

Both the eight-acre and two-acre sites proposed for the disposal areas are currently part of the Vermont Yankee Nuclear Power Plant Site inside the plant's site boundary which is enclosed by a chain link fence. It is

undeveloped except for transmission line structures which traverse a portion of the northern disposal area. Development potential is under the control of Vermont Yankee. At present, the eight-acre site on the north end of the plant property is used by a local farmer for the growing of feed hay for use with his dairy herd. No curtailment of this activity as a result of the low levels of radioactivity in septage will be necessary.

Utilization of the proposed sites for septic waste disposal will result in no impact on adjacent land or properties because of the separation of the disposal plots from off-site properties, the general movement of groundwater toward the river and away from adjacent land areas, and the very low levels of radioactive materials contained in the waste. Administrative controls on spreading and the monitoring of disposal area conditions will provide added assurance that this proposed practice will not impact adjacent properties.

4.5 Radiological Impact

In addition to state of Vermont limits imposed on septage spreading, based on nutrient and heavy metal content, the amount of septage applied on each of the proposed disposal plots will also be procedurally controlled to insure doses are maintained within the stated limits. These limits are based on NRC Nuclear Reactor Regulation (NRR) staff proposed guidance (described in AIF/NESP-037, August 1986). The proposed dose criteria require that the maximally exposed member of the general public receive a dose less than 1 mrem/year to the whole body or any organ due to the disposal material, and less than 5 mrem/year to an inadvertent intruder.

To assess the doses received by the maximally-exposed individual and the inadvertent intruder, six potential pathways have been identified. These include:

- (a) Standing on contaminated ground,
- (b) Inhalation of resuspended radioactivity,

- (c) Ingestion of leafy vegetables,
- (d) Ingestion of stored vegetables,
- (f) Ingestion of meat, and
- (g) Ingestion of milk.

The liquid pathway was also evaluated and determined to be insignificant. Both the maximum individual and inadvertent intruder are assumed to be exposed to these pathways with difference between the two related to the occupancy time. The basic assumptions used in the radiological analyses include:

- (a) Exposure to the ground contamination and to resuspended radioactivity is for a period of 104 hours per year during Vermont Yankee active control of the disposal sites, and continuous thereafter. The 104-hour interval being representative of a farmer's time on a plot of land (4 hours per week for 6 months).
- (b) The septic tanks are emptied every 6 months. (Expected practice is to pump septic tanks once per year.)
- (c) The tank radioactivity remains constant at the currently determined level. To account for the uncertainty associated with the counting statistics, the measured activity concentrations listed in Section 2 were increased by 3 sigmas. That is, the activity concentrations employed in dose assessment and the total radioactivity content per pump-out (at 700 kg of solids per batch) are as follows:

<u>Isotope</u>	<u>Upper-Bound Activity Concentration [pCi/kg dry]</u>	<u>Upper-Bound Activity Content [Ci/tankful]</u>
Mn-54	1,348	9.436E-07
Co-60	23,060	1.614E-05
Zn-65	1,620	1.134E-06
Cs-134	322	2.254E-07
Cs-137	4,100	2.870E-06

- (d) The radiation source corresponds to the accumulation of radioactive material on a single plot (two-acre) within the proposed disposal sites over a period of 20 years (40 applications at 6-month intervals). (In actuality, the proposed sites will accommodate more than one disposal plot, and, in practice, more than one plot will most probably be used with an application frequency of once per year.)
- (e) For the analysis of the radiological impact during Vermont Yankee active control of the disposal sites, all dispersed radioactive material remains on the surface and forms a source of unshielded radiation. (In practice, the septic waste will be either surface spread or directly injected within the top 6 inches of the disposal plot, in which case, the radioactive material will be mixed with the soil. This, in effect, would reduce the ground plane source of exposure by a factor of about four due to self-shielding.)
- (f) No radioactive material is dispersed directly on crops for human or animal consumption, crop contamination being only through root uptake.
- (g) The deposition on crops of resuspended radioactivity is insignificantly small.

- (h) Pathway data and usage factors used in the analysis are the same as those used in the plant's ODCM assessment of the off-site radiological impact from routine releases, with the exception that the fraction of stored vegetables grown on the disposal plots was conservatively increased from 0.76 to 1.0 (at present no vegetable crops for direct human consumption are grown on any of the proposed disposal plots).
- (i) It is conservatively assumed that Vermont Yankee relinquishes control of the disposal sites after the fortieth pump-out (i.e., the above source term applies also for the inadvertent intruder).
- (j) For the analysis of the impact after Vermont Yankee control of the sites is relinquished, the radioactive material is plowed under and forms a uniform mix with the top six inches of soil; but, nonetheless, undergoes resuspension at the same rate as surface contamination.

From radiological impact assessments associated with the disposal of septage on different plot sizes (Attachment 2), it was determined that a single two-acre plot within the disposal sites would accommodate the 1 mrem/year prescribed dose to the critical organ of the maximally exposed individual for a period of up to 20 years, as well as the 5 mrem/year prescribed dose to the inadvertent intruder after control is assumed to be relinquished. The calculated potential radiation exposures following the spreading of 40 combined (main septic system and south disposal system) tankfuls (at six-month intervals) on a single two-acre plot are as follows:

<u>Control of Disposal Sites</u>	<u>Radiation Exposure</u>	<u>Individual/Organ</u>
Controlled by VYNPS (Maximum Exposed Individual)	0.1 mrem/yr 0.2 mrem/yr Maximum	Child/Whole Body Child/Liver
Uncontrolled (Inadvertent Intruder)	1.3 mrem/yr 3.9 mrem/yr Maximum	Adult/Whole Body Teenager/Lung

The individual pathway contributions to the total dose at the end of the 20-year accumulation of waste deposited on a single two-acre plot are as listed below:

Pathway-Dependent Critical Organ Doses

<u>Pathway</u>	<u>Maximally Exposed Individual/Organ (Child/Liver) (mrem/year)</u>	<u>Inadvertent Intruder Critical Individual/Organ (Teenager/Lung) (mrem/year)</u>
Ground Irradiation	0.0576	1.16
Inhalation	0.00122	2.74
Stored Vegetables	0.0913	0.00601
Leafy Vegetable	0.00467	0.00040
Milk Ingestion	0.0421	0.00229
Meat Ingestion	<u>0.00249</u>	<u>0.00012</u>
TOTAL	0.1994	3.909

In addition, an isotopic breakdown of the critical organ dose results listed above is shown in the following table:

Isotopic Breakdown of Maximum Radiation Exposures

<u>Description</u>	<u>Isotope</u>	<u>Radioactivity [μCi/2 Acres]</u>	<u>Exposure [mrem/yr]</u>
During Vermont Yankee control of the disposal sites. Maximally Exposed Individual/Organ: Child/Liver	Mn-54	2.831	0.000436
	Co-60	235.3	0.0559
	Zn-65	2.801	0.0230
	Cs-134	1.457	0.00231
	Cs-137	92.59	<u>0.118</u>
TOTAL			0.199
After Vermont Yankee control of sites is relinquished. Inadvertent Intruder Critical Individual/Organ: Teenager/Lung	Mn-54	2.831	0.0144
	Co-60	235.3	3.76
	Zn-65	2.801	0.00983
	Cs-134	1.457	0.000505
	Cs-137	92.59	<u>0.1247</u>
TOTAL			3.91

Of interest are also derived dose conversion factors which provide a means of ensuring septage disposal operations within the prescribed radiological guidelines. The critical-organ (worst-case) all-pathway values per acre are as follows:

All-Pathway Critical-Organ Dose Conversion Factors
During Vermont Yankee Control of Disposal Sites

<u>Isotope</u>	<u>Individual/Organ</u>	<u>Exposure</u> <u>[mrem/yr-μCi/acre]</u>
Mn-54	Adult/GE-LLI	3.74E-4
Co-60	Teenager/Lung	7.14E-4
Zn-65	Child/Liver	1.64E-2
Cs-134	Child/Liver	3.18E-3
Cs-137	Child/Bone	2.66E-3

The calculational methodology and details of the radiological assessment and proposed operational controls on total activity and concentration of waste to be disposed are presented in Attachment 2.

5.0 RADIATION PROTECTION

The disposal operation will follow the applicable Vermont Yankee procedures to maintain doses as low as reasonably achievable and within the specified dose and release concentration criteria.

REFERENCES

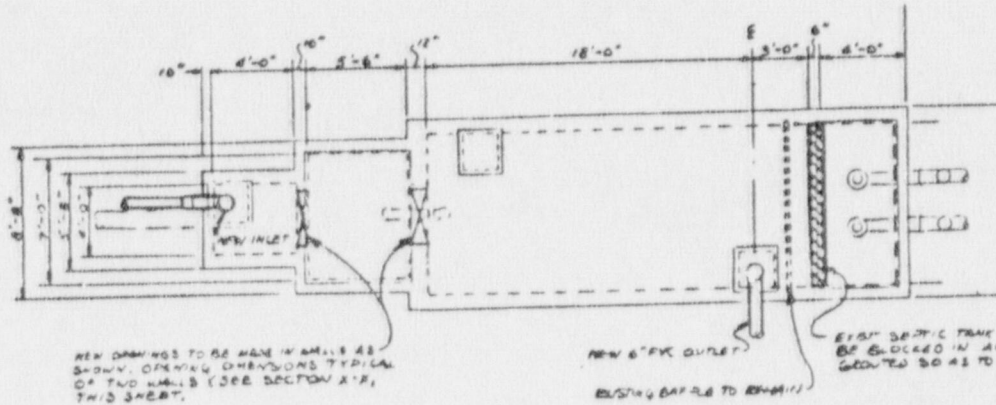
1. Flood Insurance Study, Vernon, Vermont, Windham County, FEMA, Community No. 500137, July 25, 1980.
2. Flood Insurance Study, Town of Hinsdale, New Hampshire, Cheshire County, FEMA, Community No. 330022, October 15, 1980.
3. Vermont Yankee Well Development Evaluation by Wagner, Heindel, and Noyes, Inc. July 10, 1986.
4. NUREG/CR-3130, Influence of Leach Rate and Other Parameters on Groundwater Migration, by Dames & Moore, February 1983.
5. Vermont Yankee Nuclear Power Corporation On-Site Septage Disposal Plan, by Wagner, Heindel, and Noyes, Incorporated, June 1989.

FIGURE 1

MAIN SEPTIC TANK

ENLARGED SYSTEM PLAN

SCALE 1/4" = 1'-0"

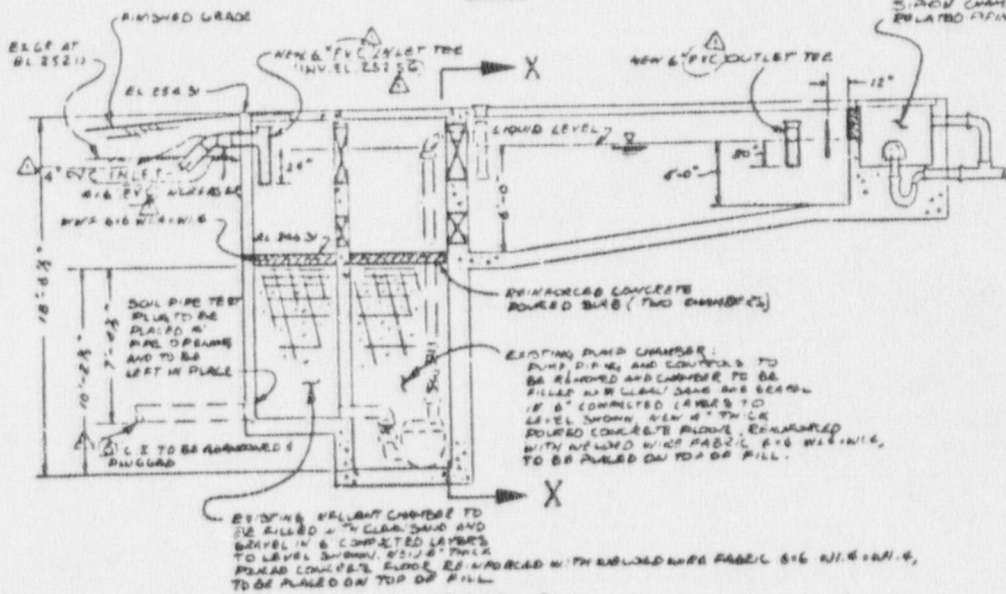


NEW DRAWINGS TO BE MADE IN SMALL AS SHOWN. OPENING DIMENSIONS TYPICAL OF TWO WALLS (SEE SECTION X-X, THIS SHEET).

NEW 6" PVC OUTLET
EXISTING 6" PVC TO REMAIN

EXISTING SEPTIC TANK OUTLET TO BE BLOCKED IN AND TO BE COVERED SO AS TO BE WATER-TIGHT

PLAN



EXISTING PUMP CHAMBER, PUMPS AND RELATED PIPING TO BE ABANDONED

EDGE AT EL 252.11

FINISHED GRADE

NEW 6" PVC INLET TEE (INV. EL. 252.50)

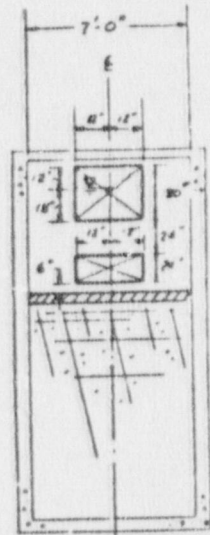
NEW 6" PVC OUTLET TEE

REINFORCED CONCRETE ROUNDED BARS (TWO CHAMBERS)

EXISTING PUMP CHAMBER: PUMP, PIPING, AND CONNECTIONS TO BE REMOVED AND CHAMBER TO BE FILLED WITH LEAD SAND AND SEALS. 12" x 8" CONCRETE LAMBS TO LEVEL SURFACE. NEW 4" THICK POLURED CONCRETE FLOOR, REINFORCED WITH WELDED WIRE FABRIC 6"x6" WELM-6, TO BE PLACED ON TOP OF FILL.

EXISTING VALVE CHAMBER TO BE FILLED WITH LEAD SAND AND GRAVEL IN 6" COMPACTED LAYERS TO LEVEL SURFACE. 12" x 8" THICK POLURED CONCRETE FLOOR, REINFORCED WITH WELDED WIRE FABRIC 6"x6" WELM-6, TO BE PLACED ON TOP OF FILL.

PROFILE

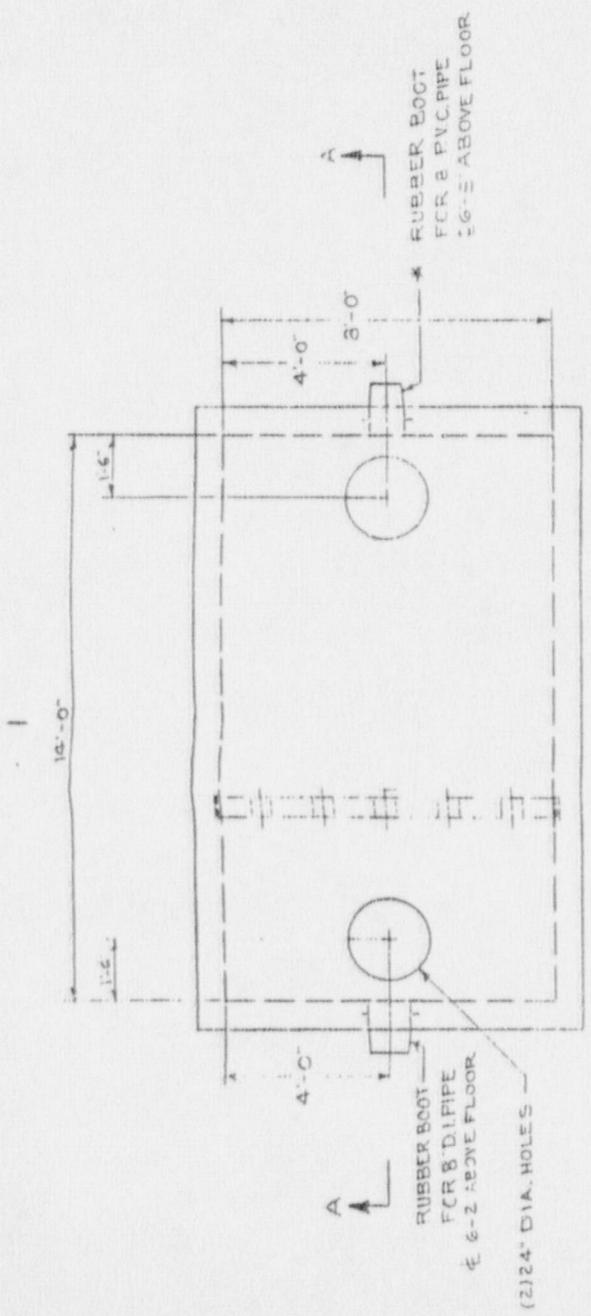


SECTION X-X
NOT TO SCALE

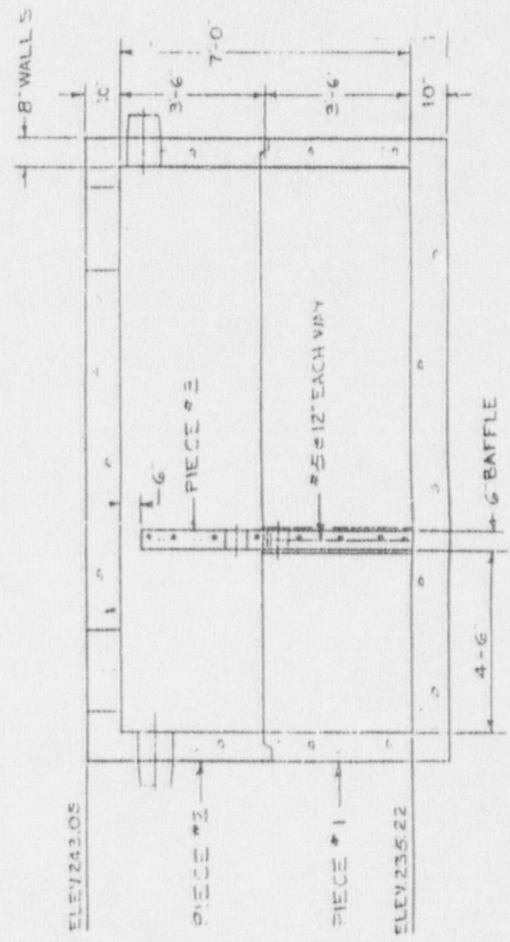
MODIFICATIONS TO EXISTING SEPTIC TANK AND CHAMBERS

NOT TO SCALE
SEE DWG. G191607 FOR DETAILS OF ORIGINAL SEPTIC TANK

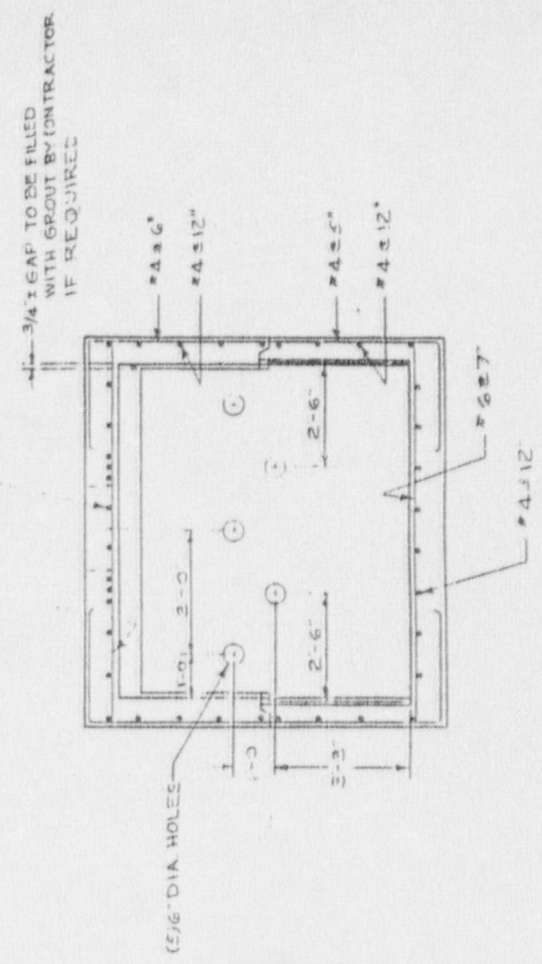
FIGURE 2
SOUTH DISPOSAL SYSTEM
SEPTIC TANK



— #6 @ 7" - (2) ADD'L. BARS, EACH
SIDE OF OPENINGS
— #5 @ 12"



SECTION A-A



SECTION B-B

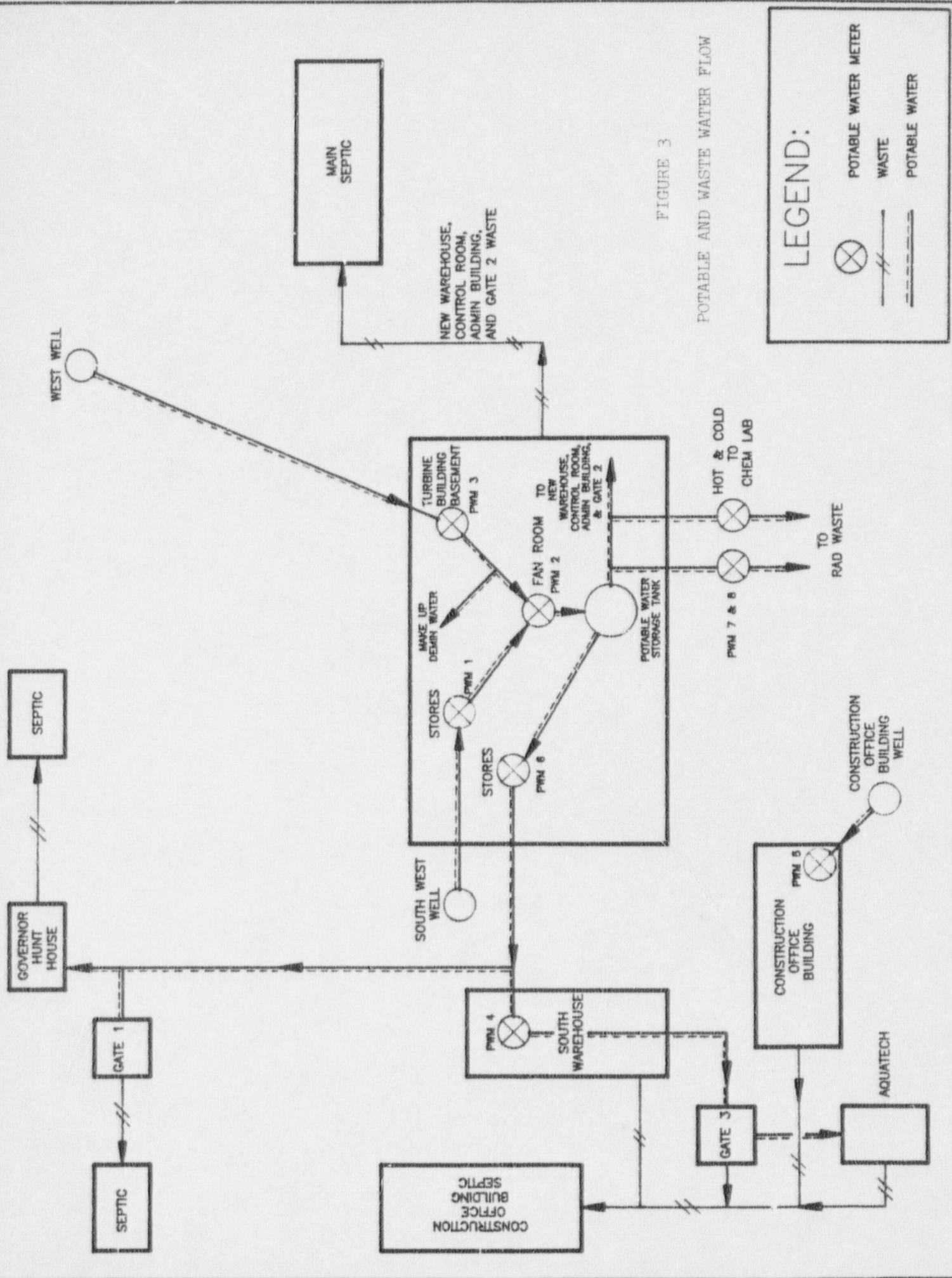


FIGURE 3

POTABLE AND WASTE WATER FLOW

LEGEND:

- ⊗ POTABLE WATER METER
- // WASTE
- // POTABLE WATER

STATE LINE - VT. N.H.

CONNECTICUT RIVER

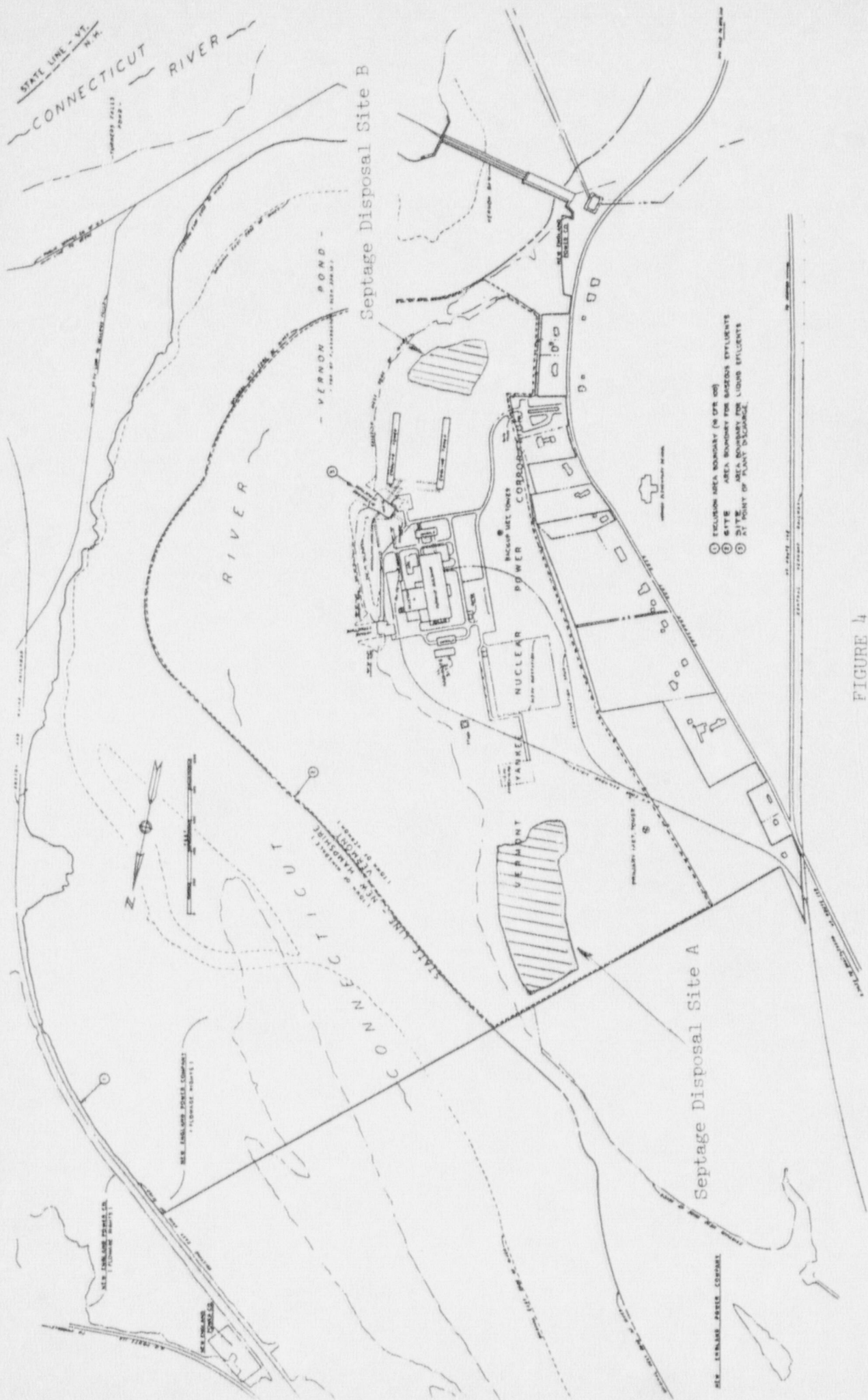


FIGURE 4

SEPTIC WASTE DISPOSAL AREAS

ATTACHMENT 2

VERMONT YANKEE NUCLEAR POWER PLANT

RADIOLOGICAL ASSESSMENT OF
ON-SITE DISPOSAL OF SEPTIC WASTE

and

PROPOSED PROCEDURAL CONTROLS TO ENSURE
COMPLIANCE WITH RADIOLOGICAL LIMITS

TABLE OF CONTENTS

	<u>Page</u>
1.0 INTRODUCTION.....	2-1
2.0 SOURCE TERM AND OTHER BASIC DATA.....	2-3
2.1 Septic Tank Specifics.....	2-3
2.2 Measured and Adjusted Septic Waste Radioactivities.....	2-3
2.3 Limiting Concentration Guidelines.....	2-4
2.4 Disposal Sites.....	2-6
2.5 Radioactivity at Disposal Plot After 20 Years.....	2-6
2.6 Land-Spreading, Resuspension and Occupancy Factors.....	2-7
2.7 Site-Specific Pathway Data and Usage Factors.....	2-9
2.8 Liquid Pathways.....	2-12
3.0 RADIOLOGICAL ASSESSMENT.....	2-15
3.1 Dose Reduction as a Result of Plowing the Radioactive Material into the Soil.....	2-15
3.2 Data Manipulation for Use with ATMADOS.....	2-17
3.2.1 Radioactivity Release Rate.....	2-17
3.2.2 Atmospheric Dispersion.....	2-18
3.2.3 Occupancy Factors.....	2-19
3.3 Land-Spreading Exposure Pathways.....	2-20
3.3.1 Impact During Vermont Yankee Control of the Disposal Sites.....	2-24
3.3.2 Radiological Impact After Termination of Vermont Yankee Control of the Disposal Sites.....	2-26
3.3.3 Isotopic Dose Conversion Factors.....	2-27
4.0 RECOMMENDED PROCEDURAL CONTROLS TO ENSURE COMPLIANCE WITH RADIOLOGICAL LIMITS.....	2-29
4.1 Total Radioactivity Dispersed per Disposal Plot.....	2-29
4.2 Operational Limits.....	2-30
4.2.1 Maximum Activity Concentrations.....	2-30
4.2.2 Potential Radiation Exposures.....	2-31
5.0 REFERENCES.....	2-35

TABLE OF CONTENTS
(Continued)

	Page
APPENDIX A - LABORATORY ANALYSES OF SEPTIC WASTE.....	A-1
APPENDIX B - COMPUTER CODE OUTPUTS.....	B-1
B.1 ALLEGRA - Gamma Ray Spectra.....	B.1-1
B.2 DIDOS-V - Dose Reduction as a Result of Plowing.....	B.2-1
B.3 ATMODOS - Radiological Impact During Vermont Yankee Control of the Disposal Sites.....	B.3-1
B.3.1 Impact due to Mn-54 in the Septage.....	B.3.1-1
B.3.2 Impact due to Co-60 in the Septage.....	B.3.2-1
B.3.3 Impact due to Zn-65 in the Septage.....	B.3.3-1
B.3.4 Impact due to Cs-134 in the Septage.....	B.3.4-1
B.3.5 Impact due to Cs-137 in the Septage.....	B.3.5-1
B.3.6 Impact due to All Nuclides in the Septage.....	B.3.6-1
B.4 ATMODOS - Radiological Impact After Termination of Vermont Yankee Control of the Disposal Sites (All Nuclides).....	B.4-1
B.5 ATMODOS - Unplowed-Land Dose Conversion Factors for Radiological Impact Assessment.....	B.5-1
B.5.1 Impact due to 1 uCi of Mn-54.....	B.5.1-1
B.5.2 Impact due to 1 uCi of Co-60.....	B.5.2-1
B.5.3 Impact due to 1 uCi of Zn-65.....	B.5.3-1
B.5.4 Impact due to 1 uCi of Cs-134.....	B.5.4-1
B.5.5 Impact due to 1 uCi of Cs-137.....	B.5.5-1

ATTACHMENT 2

VERMONT YANKEE NUCLEAR POWER PLANT

Radiological Assessment of
On-Site Disposal of Septic Waste
and
Proposed Procedural Controls to Ensure
Compliance With Radiological Limits

1.0 INTRODUCTION

This calculation is in support of Vermont Yankee's application to the Nuclear Regulatory Commission for the on-site disposal of slightly radioactive septic waste in accordance with the provisions of 10CFR20.302 for very-low-level waste disposal. Specifically, the main purposes of the calculation were as follows:

- (a) Determination of an optimal plot size for septage disposal (based on measured 1988 radioactivity concentrations in septic waste) which would accommodate both the radiological guidelines and the needed flexibilities for a smooth operation of the disposal program.
- (b) Preparation of procedural controls to ensure compliance with the radiological guidelines.

Guidance for obtaining regulatory approval to dispose of very-low-level waste is presented in AIF/NESP-037 (Reference 6). According to this reference, the NRR staff personnel have proposed a number of draft dose guidelines regarding the impact of low-level waste disposal on the public health and safety for use in the preparation of 10CFR20.302(a) requests. Of these, the following two are pertinent to the present calculation:

- (a) Doses to the total body and any body organ of a maximally exposed individual (a member of the general public or a worker who is not

classified as a radiation worker) from the probable pathways of exposure to the disposed material should be less than 1 mrem/yr.

- (b) Doses to the total body and any body organ of an inadvertent intruder, from the probable pathways of exposure, should be less than 5 mrem/yr.

In either case, consideration should be given to all possible exposure pathways, while allowing for land-usage restrictions which may be in effect. It is on these guidelines that the optimum disposal plot size was selected and the procedural controls prepared.

In addition to the dose guidelines listed above, the procedural controls recommended in this calculation also include MPC checks on the septage to be disposed. As stated in the above AIF report, the total activity concentration in the waste is expected to be below 50 pCi/gram (Reference (6), Page 4-1). This guideline is approximately equivalent to the MPC limits specified in 10CFR20, Appendix B, Table II, Column 2, for the release of radioactive material to unrestricted areas, and to also be approximately 50 times higher than the activities measured in the Vermont Yankee septic waste in 1988. A lower MPC ratio appears to be more appropriate for better control. As a result, and in addition to the prescribed dose limits, a combined MPC ratio of less than or equal to 0.1 was also included in the procedures to regulate the disposal of septic waste. With respect to the measured septage radioactivity, spectroscopic analyses of samples taken in 1988 from the Vermont Yankee main septic tank showed that the liquid portion of the collected samples did not contain any activation or fission products, and that the following plant-related radionuclides were found in the solids:

Isotope	Activity Concentration <u>±1 Sigma [pCi/kg dry]</u>
Mn 54	1,126 ± 74
Co-60	22,400 ± 220
Zn-65	1,200 ± 140
Cs-134	166 ± 52
Cs-137	3,824 ± 92

2.0 SOURCE TERM AND OTHER BASIC DATA

2.1 Septic Tank Specifics

The effective capacity of the main septic tank, when filled to its maximum depth of 6 feet, is approximately 9,250 gallons. The south sewage disposal system is newly installed (January 1989) and replaces the construction office building (COB) holding tank that had previously serviced the lavatory facilities on the south end of the plant. This new system contains a 5,700 gallon septic tank. The total design capacity of both main system and new south system septic tanks is approximately 14,950 gallons.

Prior to 1988, the main tank was usually emptied every 6 months. Due to this high pump-out frequency, the accumulation of sludge at the bottom is well below the design capacity of the tank. During the 1988 sample collections, it was estimated that the sludge thickness was less than 1 foot. For conservatism in this radiological evaluation, it was assumed that the sludge occupies 30% of the design liquid volume of both the main septic tank and new south systems tank. Also, from laboratory analyses of the septic waste, the average density of the tank contents is approximately equal to that of water, and the wet to dry ratio of the sludge is 25.4 to 1. Hence, the weight of solids (W_{sol}) being disposed of is estimated for purposes of bounding dose analyses to be approximately:

$$\begin{aligned} W^{sol} &= 14,950 \text{ gal} \times 3,785.4 \text{ cc/gal} \times 10^{-3} \text{ kg/cc} \\ &\quad \times 0.30 \text{ solids fraction} \times (1/25.4) \text{ dry/wet ratio} \\ &\approx 700 \text{ kg} \end{aligned}$$

2.2 Measured and Adjusted Septic Waste Radioactivities

Gamma spectroscopic analyses of septage samples from Vermont Yankee were carried out at the Yankee Environmental Laboratory in Westborough, Massachusetts (see Appendix A). For the main septic tank, no activation or fission products were found in the liquid portion of the collected samples. In the dry solids, on the other hand, the following man-made radionuclides were found to be statistically positive at the 99.9 percent confidence level:

<u>Isotope</u>	<u>Activity Concentration</u> <u>±1 Sigma (pCi/kg Dry)</u>
Mn-547	1126 ± 74
Co-60	22400 ± 220
Zn-65	1200 ± 140
Cs-134	166 ± 52
Cs-137	3824 ± 92

To account for the uncertainty associated with the counting statistics, the measured activity concentrations listed above were increased by 3 sigmas. That is, the activity concentrations employed in this calculation, and the total radioactivity content per combined tankful of both south and main septic tanks) (at approximately 700 kg of solids per batch, from Section 2.1 of this calculation) are as follows:

<u>Isotope</u>	<u>Upper-Bound Activity</u> <u>Concentration (pCi/kg dry)</u>	<u>Upper-Bound Activity</u> <u>Content (Ci/Batch)</u>
Mn-54	1,348	9.436E-07
Co-60	23,060	1.614E-05
Zn-65	1,620	1.134E-06
Cs-134	322	2.254E-07
Cs-137	4,100	2.870E-06

2.3 Limiting Concentration Guidelines

The AIF Report (AIF/NESP-037) provided draft guidance on total activity concentration in waste stating that it is expected to be below 50 pCi/gram. As shown below, this guideline appears to be approximately equivalent to the MPC limits specified in 10CFR20, Appendix B, Table II, Column 2, for the release of radioactive material to unrestricted areas.

For the major radionuclides identified in the Vermont Yankee septic waste, the individual MPC limits are as follows:

Maximum Permissible Concentrations in Water

<u>Isotope</u>	<u>Soluble</u> <u>(uCi/ml)</u>	<u>Insoluble</u> <u>uCi/ml</u>
Mn-54	1.0E-4	1.0E-4
Co-60	5.0E-5	3.0E-5
Zn-65	1.0E-4	2.0E-4
Cs-134	9.0E-6	4.0E-5
Cs-137	2.0E-5	4.0E-5

For a mix of radionuclides, 10CFR20 specifies that, in addition to the above individual limits, the following condition must also be met:

$$\sum (C_i / MPC_i) \leq 1.0$$

where: C_i is the measured concentration for Isotope i , and the summation is over all radionuclides in the mix.

As indicated in Section 2.2, the 1988 spectroscopic analyses of Vermont Yankee septage samples showed that there was no radioactivity in the septic water samples. That is, the limits which are currently applicable are those listed above for insoluble compounds. Using the activity data from Section 2.2, along with the main septic tank volume of 9,250 gallons, the current upper-bound activities and MPC ratios are approximately:

<u>Isotope</u>	<u>Upper-Bound Activity</u> <u>Content (Ci/tankful)</u>	<u>Upper-Bound Activity</u> <u>Concentration (uCi/ml)</u>	<u>MPC Ratio</u>
Mn-54	9.44E-07	2.67E-08	2.67E-04
Co-60	1.61E-05	4.57E-07	1.52E-02
Zn-65	1.13E-06	3.21E-08	1.60E-04
Cs-134	2.25E-07	6.38E-09	1.59E-04
Cs-137	2.87E-06	8.13E-08	2.03E-03
TOTAL	2.13E-05	6.03E-07	1.78E-02

It is seen that the overall MPC ratio is approximately 1.8 % of the regulatory limit, and that the total concentration is 1.2 % of the 50 pCi/g guideline. Thus, the sludge activity concentration can be at least 50 times higher without exceeding either limit. Obviously, if the MPC ratio of 1 or the 50 pCi/g guideline are not revised, the on-site disposal of septic waste will be regulated solely by the prescribed radiation exposure limits. For better control, therefore, it is hereby proposed that, in addition to the prescribed dose limits, a combined MPC ratio of less than or equal to 0.1 be also included in the procedures to regulate the disposal of septage. Refer to Section 4 for more details.

2.4 Disposal Sites

There are two sites on Vermont Yankee site property which are currently designated for on-site septic waste disposal, as follows:

- (a) Site A, a 8-acre site approximately 2,200 feet northwest of the Reactor Building.
- (b) Site B, a 2-acre site approximately 1,700 feet south of the Reactor Building.

Both sites are within the plant's site boundary and surrounded by a chain link fence, and under direct control of Vermont Yankee for all access.

2.5 Radioactivity at Disposal Plot After 20 Years

It is clear that, due to the longevity of the two primary isotopes identified in the sludge (Co-60 and Cs-137), the amount of radioactivity at the disposal plot will be increasing with each disposal application. However, since the content of radioactivity in septic waste is very low, and since it is neither practical nor necessary to carry out a new dose analysis prior to each disposal, the approach employed in this calculation was to assess the potential radiological impact at approximately the end of plant life. That is, the radiation source was assumed to correspond to the accumulation of

radioactive material on a given plot within the proposed disposal sites over a period of 20 years (40 applications at an assumed 6-month interval).

Analytically, if Q_0 is the amount of radioactivity per batch for a given isotope, then the total accumulated radioactivity Q_e at the disposal plot after 40 applications is given by:

$$Q_e = Q_0 (1 + E + E^2 + E^3 + E^4 + \dots + E^{39}) \quad (2.1)$$

$$= Q_0 (1 - E^{39}) / (1 - E) \quad (2.2)$$

where: $E = \exp(-\lambda \Delta t)$ (2.3)

λ = is the decay constant for the selected isotope (1/year)

and

Δt = time interval between applications = 0.5 year

For the isotopes of interest, the results are as follows:

Isotope	Half Life	(1/yr)	Q_0 (Ci/batch)	Q_e/Q_0	Q_e (Ci)
Mn-54	312.2 d	0.8109	9.436E-7	3.000	2.831E-06
Co-60	5.272 y	0.1315	1.614E-5	14.58	2.353E-04
Zn-65	243.8 d	1.038	1.134E-6	2.470	2.801E-06
Cs-134	2.065 y	0.3357	2.254E-7	6.464	1.457E-06
Cs-137	30.17 y	0.02297	2.870E-6	32.26	9.259E-05

2.6 Land-Spreading, Resuspension and Occupancy Factors

As pointed out above, even though the proposed sites can accommodate more than one disposal plot, only a single disposal plot will be assumed in assessing the potential radiological impact. If this plot has a surface area

of N acres, then the surface area deposition S_e (Ci/m²) following 40 disposal applications will be equal to:

$$S_e = Q_e \text{ (Ci)} / (N \text{ (acres)} \times 4046.9 \text{ (m}^2\text{/acre)}) \quad (2.4)$$

The denominator of this equation is equivalent to the (D/Q) deposition factor normally employed in the impact assessment of deposited radionuclides. That is:

$$\begin{aligned} (D/Q) &= 1 / (N \text{ (acres)} \times 4046.9 \text{ (m}^2\text{/acre)}) \\ &= 2.471\text{E-}04 / N \text{ (m}^{-2}\text{)} \end{aligned} \quad (2.5)$$

Following the application of septage on the disposal plot, some of the radioactivity may become airborne as a result of resuspension effects. The model used to estimate the radionuclide concentration in air above the disposal plot was taken from WASH-1400, Appendix VI (Reference 7). According to that model, the relationship between the airborne concentration A_e (Ci/m³) and the surface deposition is:

$$A_e = S_e \text{ (Ci/m}^2\text{)} \times K \text{ (1/m)} \quad (2.6)$$

where: K is the resuspension factor and is equal to 1.0E-05 (1/m) for semi-arid/grassland terrains (from Reference 1).

In actual practice, septage waste will be either surface spread at a controlled rate per acre, or directly injected into the top 6 inch surface soil layer, at a precalculated rate, in order to control the limiting factor. The assumptions made for analytical purposes are as follows:

- (a) For the analysis of the radiological impact during Vermont Yankee active control of the disposal sites, no injection will be assumed to take place; all dispersed radioactive material will be assumed to remain on the surface and to form a source of unshielded radiation.

- (b) For the analysis of the impact after Vermont Yankee control of the sites is assumed to be relinquished, the radioactive material will be assumed to be plowed under and to form a uniform mix with the top 6 inches of soil (to account for the shielding provided by the soil), but, nonetheless, to undergo resuspension at the same rate as surface contamination.

Analysis of preliminary results, based on the measured radioactivity concentration found in sludge during 1988, showed that a 2-acre disposal plot would meet the radiation criteria given in Section 2.3. This is the plot size, therefore, used in the final analyses.

As for the occupancy factors for direct exposure to the ground deposition and for immersion in the resuspended radioactivity, 104 hours were used for the radiological impact analysis during active Vermont Yankee control of the disposal sites, and continuous exposure was assumed thereafter. The 104-hour interval is expected to be an upper bound of a farmer's time spent on a plot of land, which is assumed to be 4 hours per week for 6 months while he plows, plants, and harvests his crop.

2.7 Site-Specific Pathway Data and Usage Factors

The following exposure pathways were addressed in this calculation for both the maximally exposed individual (i.e., during Vermont Yankee control of the disposal sites) and for the inadvertent intruder (i.e., after control is assumed to be relinquished):

- (a) Standing on contaminated ground.
- (b) Inhalation of resuspended radioactivity.
- (c) Ingestion of leafy vegetables.
- (d) Ingestion of stored vegetables.
- (f) Ingestion of meat.
- (g) Ingestion of milk.
- (h) Liquid pathways.

Radiation exposures were computed for all pathways, with one exception. As shown in Section 2.8 below, the radiological impact from the liquid pathway was determined to be insignificant without the need of a detailed analysis. It should be noted that current agricultural activities permitted on the designated disposal sites are limited to the growing of feed crops (hay) for dairy animals. As such, the ingestion of leafy and stored vegetables are not existing exposure pathways, but have been included to demonstrate that these could also be accommodated within the proposed dose criteria for septic waste disposal.

Pathway data and usage factors as applicable to the area in the vicinity of the Vermont Yankee Nuclear Power Station are shown in the tables which follow. These are the same factors as used in the plant's ODCM assessment of the off-site radiological impact due to routine releases from the plant, with the following exceptions:

- (a) The soil exposure time for spreading of the radioactivity content of the septage to cover each period of measured deposition was changed from a standard 15 years (given in Regulatory Guide 1.109) to 1 year.
- (b) The fraction of stored vegetables grown on the contaminated land was conservatively increased from 0.76 to 1.0.
- (c) The crop exposure time was changed from 2160 hours to 0 hours to reflect the condition that no radioactive material will be dispersed directly on crops for human or animal consumption, the deposition on crops of resuspended radioactivity being insignificantly small; that is, crop contamination is only through root uptake.

USAGE FACTORS

<u>Individual</u>	<u>Vegetables (kg/yr)</u>	<u>Leafy Veg. (kg/yr)</u>	<u>Milk (l/yr)</u>	<u>Meat (kg/yr)</u>	<u>Inhalation (m³/yr)</u>
Adult	520	64	310	110	8,000
Teen	630	42	400	65	8,000
Child	520	26	330	41	3,700
Infant	---	---	330	---	1,400

VEGETABLE PATHWAY

	<u>Stored Vegetables</u>	<u>Leafy Vegetables</u>
Agricultural productivity (kg/m ²)	2.0	2.00
Soil surface density (kg/m ²)	240.0	240.0
Transport time to user (hours),	0.0	0.0
Soil exposure time (hours)	8,766.0	8,766.0
Crop exposure time to plume (hours)	.0	.0
Holdup after harvest (hours)	1,440.0	24.0
Fraction of stored vegetables grown in garden	1.0	
Fraction of leafy vegetables grown in garden		1.0

COW-MILK PATHWAY

	<u>Pasture Feed</u>	<u>Stored Feed</u>
Agricultural productivity (kg/m ²)	.7	2.0
Soil surface density (kg/m ²)	240.0	240.0
Transport time to user (hours)	48.0	48.0
Soil exposure time (hours)	8,766.0	8,766.0
Crop exposure time to plume (hours)	.0	.0
Holdup after harvest (hours)	.0	2,160.0
Animals daily feed (kg/day)	50.0	50.0
Fraction of year on pasture	.5	
Fraction pasture when on pasture	1.0	

MEAT PATHWAY

	<u>Pasture Feed</u>	<u>Stored Feed</u>
Agricultural productivity (kg/m ²)	.7	2.0
Soil surface density (kg/m ²)	240.0	240.0
Transport time to user (hours)	480.0	480.0
Soil exposure time (hours)	8,766.0	8,766.0
Crop exposure time to plume (hours)	.0	.0
Holdup after harvest (hours)	.0	2,160.0
Animals daily feed (kg/day)	50.0	50.0
Fraction of year on pasture	.5	
Fraction pasture when on pasture	1.0	

2.8 Liquid Pathways

There are three potential routes through which septic waste radioactivity may enter into the liquid pathway, as follows:

- (a) Surface water runoff.
- (b) Ground water pathway.
- (c) Accidental releases into the Connecticut River.

Since there are no potable water wells between the disposal site and the river, it is evident that the only way for septic waste radioactivity to enter the liquid pathway is via the Connecticut River.

Even though surface water runoff may be a credible pathway into the river, the fraction of disposed radioactivity which may thus be transported to the river is very small for the following reasons:

- (1) The selected disposal sites are set back from the river.
- (2) Procedural controls will ensure that during surface spreading of all the septage and any precipitation falling onto or flowing onto the disposal plot will not overflow the perimeter of the disposal site.
- (3) The disposal plots have slopes of 5% or less in order to limit surface runoff.

With respect to septage radionuclides reaching the Connecticut River via the ground-water pathway, the critical parameter is the total transport time from the field to the river. Should this transport time (which is element dependent) be large in comparison to the half-life of the radionuclide of interest, then decay in transit will remove the said radionuclide from the pathway. For the case on hand, the conservative travel times to go an average 200 feet to the river, and the fractions of land-spread radioactivity which are expected to reach the river are as follows:

Isotope	Half Life	Decay Constant (1/yr)	Travel Time to River (years) (Ground Water Path)	Fraction of Initial Activity Entering River
Mn-54	312.2 d	0.8109	3.35	6.61E-02
Co-60	5.272 y	0.1315	961.	0.0
Zn-65	243.8 d	1.038	3.35	3.09E-02
Cs-134	2.065 y	0.3357	193.	0.0
Cs-137	30.17 y	0.02297	193.	1.19E-02

Thus, only small fractions of Mn-54, Zn-65, and Cs-137 may make it to the river via the ground-water pathway; and, since the initial activities of these isotopes are relatively insignificant, it is clear that the ground-water pathway is not a credible one.

We proceed, then, with the analysis of an accidental release of the entire contents of a septic waste spreading truck directly into the Connecticut River. Following such an accident, the released radioactive material is expected to first mix with part of the water in Vernon Pond and then gradually flow downstream of the Vernon Dam. The storage volume in Vernon Pond, excluding the volume below the crest, is approximately 6.0E+9 gallons (2.3E+13 cc). Since no use is made of the river between the plant and the Vernon dam, the only potential exposure pathway is downstream of the dam; and from Reference (2), the river flow through the dam is typically 10,000 cfs, and no less than 1,200 cfs during the dry season.

As a conservative condition, assume that the septic waste mixes with just one thousandth of the Vernon pond storage volume, i.e., with 2.3E+10 cc. This amount of water will pass through the dam in about 11 minutes if the river flow is 1,200 cfs, and in about 1.3 minutes if the flow is 10,000 cfs. Using the upper-bound activities given in Section 2.3, the expected concentrations in the pond, and the corresponding MPCs are as follows:

Isotope	Upper-Bound Activity Content (Ci/Batch)	Expected Concentration in Pond (uCi/ml)	MPC Ratio
Mn-54	9.44E-07	4.10E-11	4.10E-07
Co-60	1.61E-05	7.02E-10	2.34E-05
Zn-65	1.13E-06	4.93E-11	2.47E-07
Cs-134	2.25E-07	9.80E-12	2.45E-07
Cs-137	2.87E-06	1.25E-10	3.13E-06
TOTAL	2.13E-05	9.27E-10	2.74E-05

It is seen that the concentrations are negligibly small to pose any radiological concern.

In summary, as demonstrated above, the liquid pathway is not credible.

3.0 RADIOLOGICAL ASSESSMENT

The radiological impact associated with the on-site disposal of radioactive septage at Vermont Yankee was carried out using the dose assessment models in Regulatory Guide 1.109, and is consistent with the methodology employed by the Vermont Yankee ODCM. However, since the computer code used (ATMADOS; Reference (3)) is primarily for use with atmospheric releases, it was necessary to manipulate the input to obtain the desired results for direct deposition of radioactivity on soil due to land spreading of septic waste. In particular, special consideration was given to the following:

- (a) The computation of an effective shielding factor to account for the effect provided by the soil after the waste is plowed under, or if it is directly injected into the top 6 inch surface layer.
- (b) The definition of an annual activity release rate, which following a year's time of continuous release, would yield the ground deposition expected to prevail after 40 combined tank pump-outs, as calculated in Section 2.5.
- (c) The definition of an effective atmospheric dispersion factor to represent the resuspended radioactivity.
- (d) The proper representation of partial occupancy factors.

These are discussed in Sections 3.1 and 3.2 which follow.

The results of the radiological impact assessment are presented in Sections 3.3 and 3.4.

3.1 Dose Reduction as a Result of Plowing the Radioactive Material into the Soil

As pointed out in Section 2.6 of this calculation, the impact analysis after control of the disposal sites is relinquished, was based on the

assumption that the radioactive material will be plowed to form a uniform mix with the top 6 inches of soil. To account for the gamma attenuation provided by the soil, it was necessary to carry out an appropriate shielding calculation. This was accomplished through use of the ALLEGRA and DIDOS-V computer codes (References 4 and 5). The ALLEGRA code was used to define the gamma spectrum (in MeV/sec) associated with the selected radionuclide mix. This spectrum was then entered into DIDOS-V to compute the radiation levels from the two following source/receptor geometries:

- (a) A circular disk source with a radius of 150 m (represented by a cylindrical volume with a height equal to 0.001 m), the receptor location being along the disk axis, 1 m from the disk.
- (b) A cylindrical volume source with a radius of 150 m and a height of 0.15 m, with the receptor located along the axis, 1 m above the source.

In the latter case, the source density was set equal to 1.6 g/cc; this is equivalent to the Reg. Guide 1.109 value of 240 kg/m^2 for the effective surface density of soil within a 15 cm plow layer. The source radii were assumed to be large so as to approximate semi-infinite conditions, thus, permitting a direct comparison of the DIDOS-V and ATMOS results for the unplowed land. The source intensity (in MeV/sec-m^3 , as required for input into DIDOS-V) was computed by distributing the radioactive material over a 2-acre surface, and within 0.001 m for the disk source and 0.15 m for the second case.

Copies of the ALLEGRA and DIDOS-V outputs appear in Appendix B, which should be referred to for more details. The DIDOS-V results are as follows:

Dose to air from the disk source = $1.085\text{E-}06 \text{ rad/hr}$
Dose to air from cylinder source = $2.629\text{E-}07 \text{ rad/hr}$
Overall soil shielding factor = $2.629\text{E-}07/1.085\text{E-}06 = 0.243$

At this point, it is of interest to compare the DIDOS-V and ATMOS exposure results from standing on contaminated ground. From the ATMOS output in Appendix B (Section E.3.6), where the source term was the same as used in DIDOS-V, the skin dose due to exposure to contaminated ground for 104 hours is given as $6.78E-02$ mrem. This is equivalent to a dose rate of $6.52E-4$ mrem/hr, or $(6.52E-4/1.11) = 5.87E-4$ mrad/hr to air, 1.11 being the average ratio of tissue-to-air energy absorption coefficients (from Regulatory Guide 1.109). It is seen that ATMOS underestimates the dose by a factor of 2, approximately; the reason for this is the slightly outdated set of dose conversion factors in the guide, as can be verified by inspecting the data in WASH-1400, for instance.

3.2 Data Manipulation for Use with ATMOS

3.2.1 Radioactivity Release Rate

There are two parameters in the input to ATMOS which affect the buildup of radioactivity at an off-site location, namely, the activity release rate and the accumulation period. To simulate this process, and to also account for the effect of the 40 applications described earlier, the accumulation period was set equal to 1 year, and the release rate was selected to be such that, at the end of one year, the total accumulated radioactivity at the disposal plot would be equal to the Q_e values given in Section 2.5. That is, if we define by Q_r the activity release rate (Ci/yr) which is required as input to ATMOS, then the relationship between this parameter and Q_e is as follows:

$$Q_e = Q_r (1 - E)/\lambda \quad (3.1)$$

$$E = \exp(-\lambda\Delta t) \quad (3.2)$$

λ = is the decay constant for the selected isotope (1/year) and

Δt = time interval between applications = 1 yr.

Using the information given for Q_e in Section 2.5, the desired values for Q_r are as follows:

Isotope	Q_e (Ci)	Q_r (Ci/yr)*	Ratio of $(Q_r \times 1 \text{ year})/Q_e$
Mn 54	2.831E-06	4.132E-06	1.460
Co 60	2.353E-04	2.511E-04	1.067
Zn 65	2.801E-06	4.502E-06	1.607
Cs 134	1.457E-06	1.715E-06	1.177
Cs 137	9.259E-05	9.366E-05	1.012

*For input to ATMADOS only.

3.2.2 Atmospheric Dispersion

What is of interest at this point is to provide a means of calculating the air immersion dose due to resuspension using the ATMADOS code (under the assumption that the resuspended material is due to an atmospheric release). To accomplish this, we proceed as follows. By definition, in the analysis of releases of gaseous effluents to the atmosphere, the airborne concentration at a receptor of interest is given by:

$$A_e = Q_r (\text{Ci/yr}) \times (X/Q) (\text{sec/m}^3) / 3.1536\text{E}+7 (\text{sec/yr}) \quad (3.3)$$

Where: (X/Q) is the atmospheric dispersion factor.

Combining Equations (2.4), (2.6) and (3.3), it is seen that, for long-lived radionuclides (where the total accumulated radioactivity at the end of one year is numerically equal to the annual release rate, i.e. $Q_e = Q_r \times 1 \text{ year}$), the airborne concentration at the disposal plot due to resuspension effects can be accommodated by the following atmospheric dispersion factor:

$$\begin{aligned} (X/Q) &= K (1/m) 3.1536\text{E}+7 (\text{sec/yr}) / (N (\text{acres}) \times 4046.9 (\text{m}^2/\text{acre})) \\ &= 7,792.6 (K/N) (\text{sec/m}^3) \end{aligned} \quad (3.4)$$

With $K = 1.0E-5$ (1/m), and $N = 2$ acres, the last equation reduces to:

$$(X/Q) = 3.896E-02 \text{ (sec/m}^3\text{)}.$$

At this point it is important to note that this method of analysis is slightly conservative since the receptor is assumed to be immersed in a cloud of undecayed radioactivity. From the (Q_r/Q_e) ratios given in the last table in Section 3.2.1, it is seen that inhalation exposures will be overestimated by the following factors:

Isotope	Inhalation Exposure Overestimation Factor
Mn-54	1.460
Co-60	1.067
Zn-65	1.607
Cs-134	1.177
Cs-137	1.012

3.2.3 Occupancy Factors

As indicated in Section 2.6, the occupancy factor for exposure to ground deposition and for immersion in the resuspended radioactivity was set equal to 104 hours during control of the disposal sites, and was assumed to be continuous thereafter. Since occupancy factors cannot be entered directly into the ATMADOS code, the partial occupancy situation was accommodated as follows:

- (1) The exposure to resuspended radioactivity was handled by multiplying the effective (X/Q) , as given by Equation (3.4), by $(104/8760)$, 8,760 being the number of hours in one year; this leads to a X/Q value of $4.626E-4 \text{ sec/m}^3$.
- (2) The exposure to radioactivity deposited on the ground was handled by setting the shielding correction factor equal to the occupancy factor (i.e., equal to $104/8760 = 0.012$).

It should be noted that the (X/Q) adjustment described above is appropriate in this case since radioactive material will not be dispersed on crops for human or animal consumption. The only pathway through which crop contamination can take place is through root uptake.

3.3 Land-Spreading Exposure Pathways

Three sets of ATMODOS computer runs were carried out, for the following:

- (a) Assessment of the radiological impact during Vermont Yankee control of the disposal sites.
- (b) Assessment of the radiological impact after control of the sites is assumed to be relinquished.
- (c) Development of dose conversion factors providing a correlation between pathway exposures per soil activity for each isotope of interest.

The results for each case are presented in the subsections which follow. Briefly, note that they correspond to a disposal plot size of 2 acres, which was determined to be the appropriate size to meet both the radiation exposure criteria listed in Section 2.3, and the desired flexibilities listed in Section 2.4. The whole body and critical-organ radiation exposures (after 40 pump-outs on the same plot at a concentration level equivalent to the measured 1988 concentrations in septic waste) are as follows:

<u>Control of Disposal Sites</u>	<u>Radiation Exposure</u>	<u>Individual/Organ</u>
Controlled by VYNPS (Maximum Exposed Individual)	0.1 mrem/yr 0.2 mrem/yr Maximum	Child/Whole Body Child/Liver
Uncontrolled (Inadvertent Intruder)	1.3 mrem/yr 3.9 mrem/yr Maximum	Adult/Whole Body Teenager/Lung

The individual pathway contributions to the total dose are as follows:

Pathway-Dependent Critical Organ Doses

<u>Pathway</u>	<u>Maximally Exposed Individual/Organ (Child/Liver) (mrem/year)</u>	<u>Inadvertent Intruder Critical Individual/Organ (Teenager/Lung) (mrem/year)</u>
Ground Irradiation	0.0576	1.16
Inhalation	0.00122	2.74
Stored Vegetables	0.0913	0.00601
Leafy Vegetable	0.00467	0.00040
Milk Ingestion	0.0421	0.00229
Meat Ingestion	<u>0.00249</u>	<u>0.00012</u>
TOTAL	0.1994	3.909

In addition, an isotopic breakdown of the critical organ dose results listed above is shown in the following table:

Isotopic Breakdown of Maximum Radiation Exposures

<u>Description</u>	<u>Isotope</u>	<u>Radioactivity (μCi/2 Acres)</u>	<u>(mrem/yr)</u>
During Vermont Yankee control of the disposal sites.	Mn-54	2.831	0.000436
	Co-60	235.3	0.0559
	Zn-65	2.801	0.0230
	Cs-134	1.457	0.00231
	Cs-137	92.59	<u>0.118</u>
Maximally Exposed Individual/Organ: Child/Liver	TOTAL		0.199
After Vermont Yankee control of sites is relinquished.	Mn-54	2.831	0.0144
	Co-60	235.3	3.76
	Zn-65	2.801	0.00983
	Cs-134	1.457	0.000505
	Cs-137	92.59	<u>0.1247</u>
Inadvertent Intruder Critical Individual/Organ: Teenager/Lung	TOTAL		3.91

As for the dose conversion factors during active plant control of the disposal sites, the critical-organ all-pathway values for a 2-acre disposal plot are:

All-Pathway Worst-Case Dose Conversion Factors
During Vermont Yankee Control of Disposal Sites

<u>Isotope</u>	<u>Individual/Organ</u>	<u>Exposure</u> <u>(mrem/yr-μCi)</u>
Mn-54	Adult/GE-LLI	1.87E-4
Co-60	Teenager/Lung	3.57E-4
Zn-65	Child/Liver	8.21E-3
Cs-134	Child/Liver	1.59E-3
Cs-137	Child/Bone	1.33E-3

In all cases, the exposure pathways are direct shine from shielded/unshielded ground deposition, inhalation of resuspended radioactivity, and ingestion of contaminated food (stored vegetables, leafy vegetables, milk and meat); exposure to the ground deposition and to resuspended radioactivity is for a period of 104 hours during control of the disposal sites, and continuous thereafter. Refer to Appendix B for copies of the ATMADOS outputs, and to the following list of assumptions employed in the calculations.

Briefly, the following basic assumptions were used in the calculational analyses:

- (a) The septic tanks are emptied every 6 months (expected future practice is to pump tanks once per year).
- (b) The tank radioactivity remains constant (at the main septic tank 1988 determined level plus 3 sigma).
- (c) The radiation source corresponds to the accumulation of radioactive material on a single plot within the proposed disposal sites over a period of 20 years (40 applications at 6 month intervals). (In actuality, the proposed sites will accommodate more than one disposal plot, and, in practice, more than one plot will most probably be used.)

- (d) For the analysis of the radiological impact during Vermont Yankee control of the disposal sites, no plowing or direct injection of septage takes place and all dispersed radioactive material remains on the surface and forms a source of unshielded radiation. (In practice, the waste will be either surface spread or directly injected into the top 6 inch layer of the disposed plot, in which case the radioactive material will be mixed with the soil. This in effect would reduce the ground plane source of exposure by a factor of about four due to self-shielding.)
- (e) No radioactive material is dispersed directly on crops for human or animal consumption, crop contamination being only through root uptake.
- (f) The deposition on crops of resuspended radioactivity is insignificantly small.
- (g) Pathway data and usage factors used in the analysis are the same as those used in the plant's ODCM assessment of the off-site radiological impact from routine releases, with the exception that the fraction of stored vegetables grown on the disposal plots was conservatively increased from 0.76 to 1.0. (At present, no vegetable crops for direct human consumption are grown on any of the disposal sites.)
- (h) It is assumed that Vermont Yankee relinquishes control of the disposal sites after the fortieth pump-out (i.e., the above source term applies also for the inadvertent intruder).
- (i) For the analysis of the impact after Vermont Yankee control of the sites is relinquished, the radioactive material is plowed under and forms a uniform mix with the top 6 inches of soil, but, nonetheless, undergoes resuspension at the same rate as surface contamination.

- (k) Exposure to the ground deposition and to resuspended radioactivity is for a period of 104 hours during Vermont Yankee control of the disposal sites, and continuous thereafter, the 104-hour interval being representative of a farmer's time on a plot of land (4 hours per week for 6 months).

3.3.1 Impact During Vermont Yankee Control of the Disposal Sites

The tables which follow present summaries of the ATMADOS results for the radiological impact during Vermont Yankee control of the disposal sites. The first table presents the results for the entire mix of radionuclides, and the second table shows the contributions by each isotope.

Total Accumulated Radioactivity on 2-Acre Plot After 40 Disposal Applications

<u>Isotope</u>	<u>Curies</u>
Mn 54	2.831E-06
Co 60	2.353E-04
Zn 65	2.801E-06
Cs 134	1.457E-06
Cs 137	9.259E-05

Dose Delivered to Each Organ From all Radionuclides in the Mix and From All Pathways Combined* (Adult, Teenager, Child, and Infant) (mrem/yr)

	<u>Bone</u>	<u>Liver</u>	<u>Kidney</u>	<u>Lung</u>	<u>GI-LLI</u>	<u>Thyroid</u>	<u>Whole Body</u>	<u>Skin</u>
A	9.20E-02	1.13E-01	7.92E-02	8.46E-02	9.66E-02	5.76E-02	9.42E-02	6.78E-02
T	1.13E-01	1.44E-01	9.08E-02	9.90E-02	9.65E-02	5.76E-02	9.41E-02	6.78E-02
C	1.86E-01	1.99E-01	1.10E-01	9.73E-02	8.22E-02	5.76E-02	9.94E-02	6.78E-02
I	1.14E-01	1.38E-01	8.36E-02	8.08E-02	7.59E-02	5.76E-02	7.19E-02	6.78E-02

*Each pathway includes unshielded exposure to ground contamination for 104 hours, with all radioactivity assumed to be on the surface of the ground; exposure to resuspended radioactivity is also for a period of 104 hours.

Isotope-Specific Contributions to the Dose Delivered to Each Organ
From a 1 Pathways Combined*
(Adult, Teenager, Child, and Infant)
(mrem/yr)

	<u>Bone</u>	<u>Liver</u>	<u>Kidney</u>	<u>Lung</u>	<u>GI-LLI</u>	<u>Thyroid</u>	<u>Whole Body</u>	<u>Skin</u>
Source: Mn-54, 2.831E-06 Ci (2-acre plot)								
A	2.13E-04	3.18E-04	2.44E-04	2.98E-04	5.30E-04	2.13E-04	2.33E-04	2.50E-04
T	2.13E-04	3.66E-04	2.59E-04	3.34E-04	5.25E-04	2.13E-04	2.44E-04	2.50E-04
C	2.13E-04	4.36E-04	2.76E-04	3.09E-04	4.00E-04	2.13E-04	2.73E-04	2.50E-04
I	2.13E-04	2.18E-04	2.14E-04	2.74E-04	2.15E-04	2.13E-04	2.14E-04	2.50E-04
Source: Co-60, 2.353E-04 Ci (2-acre plot)								
A	5.20E-02	5.37E-02	5.20E-02	7.40E-02	8.32E-02	5.20E-02	5.56E-02	6.12E-02
T	5.20E-02	5.44E-02	5.20E-02	8.41E-02	8.30E-02	5.20E-02	5.73E-02	6.12E-02
C	5.20E-02	5.55E-02	5.20E-02	7.80E-02	7.16E-02	5.20E-02	6.24E-02	6.12E-02
I	5.20E-02	5.22E-02	5.20E-02	6.86E-02	5.26E-02	5.20E-02	5.25E-02	6.12E-02
Source: Zn-65, 2.801E-06 Ci (2-acre plot)								
A	3.43E-03	1.06E-02	7.13E-03	2.03E-04	6.72E-03	1.46E-04	4.87E-03	1.68E-04
T	4.61E-03	1.57E-02	1.01E-02	2.28E-04	6.72E-03	1.46E-04	7.38E-03	1.68E-04
C	8.72E-03	2.30E-02	1.45E-02	2.11E-04	4.16E-03	1.46E-04	1.44E-02	1.68E-04
I	6.18E-03	2.08E-02	1.02E-02	1.88E-04	1.76E-02	1.46E-04	9.69E-03	1.68E-04
Source: Cs-134, 1.457E-06 Ci (2-acre plot)								
A	5.89E-04	1.09E-03	5.06E-04	3.20E-04	2.42E-04	2.27E-04	9.31E-04	2.65E-04
T	7.92E-04	1.56E-03	6.50E-04	3.89E-04	2.44E-04	2.27E-04	8.44E-04	2.65E-04
C	1.50E-03	2.31E-03	8.74E-04	4.59E-04	2.38E-04	2.27E-04	6.67E-04	2.65E-04
I	7.74E-04	1.25E-03	4.89E-04	3.35E-04	2.30E-04	2.27E-04	3.30E-04	2.65E-04
Source: Cs-137, 9.259E-05 Ci (2-acre plot)								
A	3.57E-02	4.70E-02	1.93E-02	9.79E-03	5.86E-03	5.06E-03	3.25E-02	5.90E-03
T	5.52E-02	7.18E-02	2.78E-02	1.39E-02	6.00E-03	5.06E-03	2.83E-02	5.90E-03
C	1.23E-01	1.18E-01	4.19E-02	1.83E-02	5.76E-03	5.06E-03	2.18E-02	5.90E-03
I	5.50E-02	6.35E-02	2.07E-02	1.14E-02	5.24E-03	5.06E-03	9.20E-03	5.90E-03

*Each pathway includes unshielded exposure to ground contamination for 104 hours, with all radioactivity assumed to be on the surface of the ground; exposure to resuspended radioactivity is also for a period of 104 hours.

3.3.2 Radiological Impact After Termination of Active Control of the Disposal Sites

The table which follows presents a summary of the ATMADOS results for the radiological impact after control of the disposal sites is assumed to be relinquished after 20 years of septic waste disposal. Tables showing the contributions by the various isotopes were not prepared as they were determined to be of little significance.

Total Accumulated Radioactivity on 2-Acre Plot
After 40 Disposal Applications

<u>Isotope</u>	<u>Curies</u>
25 Mn-54	2.831E-06
27 Co-60	2.353E-04
30 Zn-65	2.801E-06
55 Cs-134	1.457E-06
55 Cs-137	9.259E-05

Dose Delivered to Each Organ From all Radionuclides in the Mix
and From all Pathways Combined*
(Adult, Teenager, Child, and Infant)
(mrem/yr)

	<u>Bone</u>	<u>Liver</u>	<u>Kidney</u>	<u>Lung</u>	<u>GI-LLI</u>	<u>Thyroid</u>	<u>Whole Body</u>	<u>Skin</u>
A	1.25E+00	1.29E+00	1.21E+00	3.04E+00	1.29E+00	1.16E+00	1.25E+00	1.37E+00
T	1.30E+00	1.35E+00	1.23E+00	3.91E+00	1.28E+00	1.16E+00	1.24E+00	1.37E+00
C	1.40E+00	1.41E+00	1.25E+00	3.39E+00	1.22E+00	1.16E+00	1.23E+00	1.37E+00
I	1.28E+00	1.32E+00	1.21E+00	2.58E+00	1.19E+00	1.16E+00	1.19E+00	1.37E+00

*Each pathway includes continuous exposure to ground contamination (uniformly distributed within a 6-inch layer of soil)

3.3.3 Isotopic Dose Conversion Factors

The table which follows presents isotope-dependent dose conversion factors for the various age groups and organs. They were computed using the ATMADOS computer code along with all the assumptions employed in the assessment of the radiological impact during Vermont Yankee control of the disposal sites. The source terms were defined using the adjustment ratio $(Q_r \times 1 \text{ yr})/Q_e$ given in Section 3.2.1 to obtain an accumulated radioactivity of 1 uCi for each isotope of interest at the end of one year. These conversion factors form part of one of the procedural controls described in Section 4 for ensuring that the disposed contaminated septage does not lead to radiation exposures in excess of the specified limits.

Dose Conversion Factors
For Radioactive Material Spread over Two Acres
For all Pathways Combined*
(Adult, Teenager, Child, and Infant)
(mrem/yr-uCi)

	<u>Bone</u>	<u>Liver</u>	<u>Kidney</u>	<u>Lung</u>	<u>GI-LLI</u>	<u>Thyroid</u>	<u>Whole Body</u>	<u>Skin</u>
Source: Mn-54								
A	7.54E-05	1.12E-04	8.63E-05	1.05E-04	1.87E-04	7.54E-05	8.24E-05	8.84E-05
T	7.54E-05	1.29E-04	9.15E-05	1.18E-04	1.85E-04	7.54E-05	8.61E-05	8.84E-05
C	7.54E-05	1.54E-04	9.74E-05	1.09E-04	1.41E-04	7.54E-05	9.63E-05	8.84E-05
I	7.54E-05	7.71E-05	7.58E-05	9.68E-05	7.60E-05	7.54E-05	7.58E-05	8.84E-05

Source: Co-60								
A	2.21E-04	2.28E-04	2.21E-04	3.14E-04	3.54E-04	2.21E-04	2.36E-04	2.60E-04
T	2.21E-04	2.31E-04	2.21E-04	3.57E-04	3.53E-04	2.21E-04	2.43E-04	2.60E-04
C	2.21E-04	2.36E-04	2.21E-04	3.32E-04	3.04E-04	2.21E-04	2.65E-04	2.60E-04
I	2.21E-04	2.22E-04	2.21E-04	2.92E-04	2.24E-04	2.21E-04	2.23E-04	2.60E-04

Source: Zn-65								
A	1.22E-03	3.78E-03	2.55E-03	7.24E-05	2.40E-03	5.20E-05	1.74E-03	5.98E-05
T	1.65E-03	5.59E-03	3.60E-03	8.12E-05	2.40E-03	5.20E-05	2.64E-03	5.98E-05
C	3.11E-03	8.21E-03	5.19E-03	7.55E-05	1.48E-03	5.20E-05	5.12E-03	5.98E-05
I	2.21E-03	7.44E-03	3.63E-03	6.72E-05	6.29E-03	5.20E-05	3.46E-03	5.98E-05

Source: Cs-134								
A	4.04E-04	7.46E-04	3.47E-04	2.19E-04	1.66E-04	1.56E-04	6.39E-04	1.82E-04
T	5.44E-04	1.07E-03	4.46E-04	2.67E-04	1.67E-04	1.56E-04	5.79E-04	1.82E-04
C	1.03E-03	1.59E-03	6.00E-04	3.15E-04	1.64E-04	1.56E-04	4.58E-04	1.82E-04
I	5.31E-04	8.55E-04	3.36E-04	2.30E-04	1.58E-04	1.56E-04	2.26E-04	1.82E-04

Source: Cs-137								
A	3.86E-04	5.07E-04	2.09E-04	1.06E-04	6.33E-05	5.46E-05	3.52E-04	6.37E-05
T	5.97E-04	7.75E-04	3.00E-04	1.50E-04	6.48E-05	5.46E-05	3.06E-04	6.37E-05
C	1.33E-03	1.28E-03	4.53E-04	1.98E-04	6.23E-05	5.46E-05	2.35E-04	6.37E-05
I	5.94E-04	6.86E-04	2.24E-04	1.23E-04	5.66E-05	5.46E-05	9.94E-05	6.37E-05

*Each pathway includes unshielded exposure to ground contamination for 104 hours, with all radioactivity assumed to be on the surface of the ground; exposure to resuspended radioactivity is also for a period of 104 hours.

4.0 RECOMMENDED PROCEDURAL CONTROLS TO ENSURE COMPLIANCE WITH RADIOLOGICAL LIMITS

Once an on-site septage disposal permit has been secured, implementation of the disposal program must be accompanied with procedural controls to ensure that the applicable radiological limits are not violated. This section presents a list of proposed controls to this effect.

4.1 Total Radioactivity Dispersed per Disposal Plot

As pointed out in Section 2.5, since the content of radioactivity in septic waste is very low, and since it is neither practical nor necessary to carry out a new analysis prior to each disposal, assessment of the radiological impact was based on an assumed source corresponding to the expected accumulation of radioactive material on a given 2-acre disposal plot over a period of 20 years (40 applications at 6-month intervals). As such, it will be necessary to keep accurate records of the time and location of septage disposal and of the ensuing buildup and decay of radioactivity on each disposal plot. The basic equation to be employed is as follows:

$$Q_i^{\text{tot}} = Q_i^{\text{new}} + Q_i^{\text{old}} \exp(-\lambda_i \Delta t) \quad (4.1)$$

Where: Q_i^{tot} = total accumulated radioactivity at the selected 2-acre disposal plot after the current disposal (uCi).

Q_i^{new} = radioactivity added to the plot as a result of the current disposal (uCi).

Q_i^{old} = radioactivity accumulated at the selected disposal plot prior to the current disposal (uCi), as determined at the time of the previous disposal.

λ_i = radioactive decay constant (1/year).

Δt = time lapse since the previous disposal on the same disposal plot (years).

Q_i^{new} can be calculated using the following equation:

$$\begin{aligned} Q_i^{new} &= 14,950 \text{ gallons} \times 3,785.4 \text{ cc/gallon} \times 1.0E-3 \text{ kg/cc} \\ &\quad \times F_s \text{ (solids fraction)} \times C_i^{wet} \text{ (pCi/kg wet)} \\ &\quad \times 1.0E-6 \text{ (uCi/pCi)} \\ &= 0.0566 F_s C_i^{wet} \end{aligned} \quad (4.2)$$

Where: C_i^{wet} is the measured or estimated radionuclide concentration in the septic waste on a wet basis, and F_s is the fraction of solids in the septage per tankful. F_s was conservatively set equal to 0.3 in this calculation. 14,950 gallons equals the volume of both the main septic tank and the south disposal system collection tank.

4.2 Operational Limits

The disposal operating procedures to be established should address both the activity concentration and the potential radiation exposure. Should the activity concentration be in excess of the specified limit, then the sewage mix would not be suitable for on-site disposal and would have to be processed accordingly; this situation, however, is not likely to occur. On the other hand, approaching the exposure guideline is a possibility; but this can be easily accommodated by switching to a different plot within the disposal sites. The subsections which follow present pertinent information recommended for inclusion in the operating procedures.

4.2.1 Maximum Activity Concentrations

In line with the discussion presented in Section 2.3 of this calculation, the radionuclide concentrations in the septic waste must not exceed the following limits:

- (a) One tenth of the MPC values listed in 10 CFR 20, Appendix B, Table II, Column 2.

(b) An overall MPC ratio of less than or equal to 0.1.

For the major radionuclides identified in the Vermont Yankee septic waste, the individual MPC limits are as follows:

Maximum Permissible Concentrations in Water
(10CFR20, Appendix B, Table II)

<u>Isotope</u>	<u>Soluble</u> <u>(μCi/ml)</u>	<u>Insoluble</u> <u>(μCi/ml)</u>
Mn 54	1.0E-4	1.0E-4
Co 60	5.0E-5	3.0E-5
Zn 65	1.0E-4	2.0E-4
Cs 134	9.0E-6	4.0E-5
Cs 137	2.0E-5	4.0E-5

For a mix of radionuclides in the sewage mix, the condition to be met is:

$$\sum (C_i/MPC_i) \leq 0.1$$

Where: C_i is the measured concentration for Isotope i, and the summation is over all radionuclides in the mix.

From the 1988 spectroscopic analysis of septic waste samples, all radioactivity is expected to be in insoluble form, and no radioactivity is expected in the liquid above the sludge. Should the situation change, use should be made of both the soluble and insoluble MPCs listed above, as appropriate.

4.2.2 Potential Radiation Exposures

As described in Section 2.3 of this calculation, the NRR draft guidelines for radiation exposure from all probable pathways due to the disposal of low-level waste are 1 mrem/yr to the total body and any body organ of a maximally exposed individual, and 5 mrem/yr to an inadvertent intruder. The maximally exposed individual is identified as a member of the general public or a worker who is not classified as a radiation worker.

Since the proposed septage disposal sites are within VYNPS property and under VYNPS control, occupancy of the disposal sites by an inadvertent intruder is only possible after plant decommissioning. That is, during the on-site septic waste disposal program, only the specified exposure guideline for the maximally exposed individual would be in effect.

To ensure proper operation of the on-site disposal program, a set of checkpoints was prepared as guidance. The action levels were based on the following results from Section 3:

Isotopic Breakdown of Maximum Radiation Exposures

<u>Description</u>	<u>Isotope</u>	<u>Radioactivity (μCi/2 Acres)</u>	<u>Exposure (mrem/yr)</u>
During Vermont Yankee control of the disposal sites.	Mn-54	2.831	0.000436
	Co-60	235.3	0.0559
	Zn-65	2.801	0.0230
	Cs-134	1.457	0.00231
Maximally Exposed Individual/Organ: Child/Liver	Cs-137	92.59	<u>0.118</u>
	TOTAL		0.199
After Vermont Yankee control of sites is relinquished. Inadvertent Intruder Critical Individual/Organ: Teenager/Lung	Mn-54	2.831	0.0144
	Co-60	235.3	3.76
	Zn-65	2.801	0.00983
	Cs-134	1.457	0.000505
	Cs-137	92.59	<u>0.1247</u>
	TOTAL		3.91

It is seen that, whereas the exposure to the maximally exposed individual is approximately 20% of the 1 mrem/year guideline, the inadvertent intruder exposure is almost 80% of the 5 mrem/year limit. Thus, to ensure that both guidelines are met at all times, it is intended that the operational guideline for the maximally-exposed individual be set at 0.2 mrem/year. This is a conservative approach since the likelihood of intruder occupancy of the sites coinciding with the end of the on-site disposal program is nil; substantial decay of the radioactive material is expected by the time the sites are released to the general public. Of course, future reassessment of this operational guideline is not precluded. However, an operational limit close to the guideline is not recommended since it eliminates all flexibilities.

Based on the operational guideline of 0.2 mrem/yr to the maximally exposed individual, two checkpoints were prepared which would ensure that the radiation exposure limit will not be exceeded. They are as follows:

(a) Action Level 1 - Gross Radioactivity Limit

The up-to-date total radioactivity dispersed per disposal plot (Q_i^{tot}) is calculated for each isotope using Equation (4.1).
If the condition:

$$Q_i^{tot} < Q_i^{lim}$$

is met for each isotope, where Q_i^{lim} represents the limiting values listed in the following table (from Section 3, rounded off to 2 significant figures), then disposal of the septic waste will not violate the exposure limit; otherwise, proceed to Action Level 2.

Isotope	Maximum Accumulated Radioactivity Allowed Per Acre
	Q_i^{lim} (uCi)
Mn-54	1.4
Co-60	120.0
Zn-65	1.4
Cs-134	0.7
Cs-137	46.5

(b) Action Level 2 - Radiation Exposure

If Action Level 1 fails, determine the potential radiation exposure using the equation:

$$\text{Dose (mrem/yr)} = \sum Q_i^{tot} DCF_i$$

Where: Q_i^{tot} is the up-to-date total radioactivity dispersed per disposal plot calculated for each isotope using Equation (4.1), DCF_i is the dose conversion factor for isotope i, and the summation is over all the radionuclides in the mix. The dose conversion factors are as follows:

<u>Isotope</u>	<u>Individual/Organ</u>	<u>DCF_i</u> <u>(mrem/year-uCi/acre)</u>
Mn-54	Adult/GI-LLI	3.74E-04
Co-60	Teenager/Lung	7.14E-04
Zn-65	Child/Liver	1.64E-02
Cs-134	Child/Liver	3.18E-03
Cs-137	Child/Bone	2.66E-03

The model overestimates the exposure by approximately 20% because the above DCF's correspond to the most restrictive exposure to any individual and any organ from all pathways, independently selected for each radionuclide. The exposure pathways are direct shine from unshielded ground deposition, inhalation of resuspended radioactivity, and ingestion of contaminated food (stored vegetables, leafy vegetables, milk and meat); exposure to the ground deposition and to resuspended radioactivity is for a period of 10⁴ (hours/year).

If the calculated dose is in excess of 0.2 (mrem/yr), a different disposal plot would have to be selected.

5.0 REFERENCES

1. U.S. Nuclear Regulatory Commission, Reactor Safety Study, Appendix VI, Calculation of Reactor Accident Consequences, WASH-1400 (NUREG 75/014), October 1975.
2. Vermont Yankee Nuclear Power Station, FSAR, Section 2.4.4 Uses of River.
3. ATMODOS, A YAEC Computer Code for the Calculation of Off-Site Doses from Iodines and Particulates Discharged to the Atmosphere in line with the Models In Regulatory Guide 1.109
4. J. N. Hamawi, ALLEGRA - A Computer Code Making Use of the ORIGEN-2 Data Bases for the Analysis of Radioactive Decay Chains and the Computation of Gamma Spectra, ENTECH Engineering, Inc., Marlboro, MA, Technical Report P100-R15 (technical report in preparation).
5. J. N. Hamawi, DIDOS-III - A Three-Dimensional Point-Kernel Shielding Code for Cylindrical Sources, ENTECH Engineering, Inc, Technical Report P100-R2, December 1982 (an upgraded version of the code, DIDOS-V, suitable for the analysis of infinitely large cylindrical sources, is currently in preparation).
6. Atomic Industrial Forum, National Environmental Studies Program, A Guide for Obtaining Regulatory Approval to Dispose of Very Low Level Wastes by Alternative Means, prepared by D. W. Chan, J. P. Davis & R. W. Wofford, General Physics Corporation, Columbia, Maryland, Technical Report No. AIF/NESP-037, August 1986.

APPENDIX A

LABORATORY ANALYSES OF
SEPTIC WASTE

MAILED

YANKEE ATOMIC ELECTRIC COMPANY
ENVIRONMENTAL LABORATORY

JUN 09 1988

Initial Analysis Report

Customer: YAFEC Vermont Yankee Nuclear Power Corp.
Attention: MS. ELAINE KEEGAN
MR. EDWARD CUMMING

Report Date: 06/09/88
Analysis Date: 6 / 8 / 88
Date Received: 6 / 8 / 88
Reference Date: 6 / 8 / 88

MR. STEPHEN SKIBNIEWSKY

Sludge

Sample Amount: 1.01 Kg.
Elapsed Time : 0.65 days

Lab Sample No.: G72970
Sample Submission Code: VSL 02 2388
Other Analysis Requested: None
Comment: COB TANK BOTTOM

NUCLIDE	DECAY CORRECTION	ACTIVITY		MDC
		CONC. +- 1 SIGMA [Pico Curie / Kilogram - WET]		
Np-239	8.24E-01	(-15 +- 11)	E 0	35 E 0
Co-57	9.98E-01	(5 +- 94)	E-2	310 E-2
Ce-144	9.98E-01	(-9 +- 73)	E-1	240 E-1
Ce-141	9.86E-01	(26 +- 17)	E-1	55 E-1
Mo-99	8.49E-01	(11 +- 20)	E 0	65 E 0
Se-75	9.96E-01	(-6 +- 15)	E-1	49 E-1
Cr-51	9.84E-01	(41 +- 93)	E-1	310 E-1
I -131	9.45E-01	(-15 +- 11)	E-1	37 E-1
Be-7	9.92E-01	(102 +- 94)	E-1	310 E-1
Ku-103	9.89E-01	(-11 +- 12)	E-1	38 E-1
I -133	5.95E-01	(22 +- 17)	E-1	58 E-1
Ba-140	9.65E-01	(-175 +- 68)	E-1	230 E-1
Cs-134	9.99E-01	(16 +- 16)	E-1	53 E-1
Ru-106	9.99E-01	(4 +- 13)	E 0	43 E 0
*+ Cs-137	1.00E 00	(103 +- 18)	E-1	54 E-1
Ag-110M	9.98E-01	(-2 +- 19)	E-1	6 E-1
Zr-95	9.93E-01	(-19 +- 24)	E-1	79 E-1
Co-58	9.94E-01	(7 +- 13)	E-1	42 E-1
Mn-54	9.99E-01	(-7 +- 14)	E-1	50 E-1
*+ AcTh228	1.00E 00	(287 +- 68)	E-1	230 E-1
TeI-132	8.70E-01	(117 +- 99)	E-1	330 E-1
Fe-59	9.90E-01	(-33 +- 27)	E-1	91 E-1
Zn-65	9.98E-01	(89 +- 36)	E-1	120 E-1
*+ Co-60	1.00E 00	(454 +- 31)	E-1	82 E-1
*+ K -40	1.00E 00	(87 +- 23)	E 0	75 E 0
Sb-124	9.92E-01	(-31 +- 31)	E-1	100 E-1

Notes:

- * Activity greater than 3*standard deviation
- ⚡ Peak is found

Approved by

D E McCurdy
D.E. McCurdy

The quoted one-sigma uncertainty terms do not represent the propagation of all possible errors associated with the radioactive decay process (counting statistics). Estimates of the additional systematic and random uncertainties, use the calibration curve, ± 5 percent, sample positioning (source to detector), ± 2 percent, sample non-homogeneity, ± 10 percent, and sample self-absorption, ± 10 percent.

MAILED

YANKEE ATOMIC ELECTRIC COMPANY
ENVIRONMENTAL LABORATORY

JUN 09 1988

Initial Analysis Report

YAEC

Customer: Vermont Yankee Nuclear Power Corp.
Attention: MS. ELAINE KEEGAN
MR. EDWARD CUMMING
MR. STEPHEN SKIBNIOWSKY

Report Date: 06/09/88
Analysis Date: 6 / 8 / 88
Date Received: 6 / 8 / 88
Reference Date: 6 / 8 / 88

Sludge

Sample Amount: 1.02 Kg.

Lab Sample No.: G72971

Sample Submission Code: VSL 03 2388

Elapsed Time : 0.63 days

Other Analysis Requested: None

Comment: COB TANK-LIQUID

NUCLIDE	DECAY CORRECTION	ACTIVITY		MDC
		CONC. +/- 1 SIGMA [Pico Curie / Kilogram-WET]		
Np-239	8.29E-01	(6 +- 10)	E 0	34 E 0
Co-57	9.98E-01	(65 +- 93)	E-2	310 E-2
Ce-144	9.98E-01	(20 +- 67)	E-1	220 E-1
Ce-141	9.87E-01	(0 +- 16)	E-1	52 E-1
Mo-99	8.54E-01	(-25 +- 19)	E 0	63 E 0
Se-75	9.96E-01	(4 +- 14)	E-1	48 E-1
Cr-51	9.84E-01	(81 +- 86)	E-1	290 E-1
I -131	9.47E-01	(8 +- 11)	E-1	38 E-1
Be-7	9.92E-01	(1 +- 10)	E 0	35 E 0
Ru-103	9.89E-01	(17 +- 12)	E-1	41 E-1
I -133	6.05E-01	(-1 +- 19)	E-1	65 E-1
Ba-140	9.66E-01	(33 +- 72)	E-1	240 E-1
Cs-134	9.99E-01	(-17 +- 14)	E-1	48 E-1
Ru-106	9.99E-01	(-1 +- 12)	E 0	40 E 0
Cs-137	1.00E 00	(-5 +- 13)	E-1	44 E-1
Ag-110M	9.98E-01	(-6 +- 16)	E-1	54 E-1
Zr-95	9.93E-01	(20 +- 22)	E-1	75 E-1
Co-58	9.94E-01	(13 +- 12)	E-1	39 E-1
Mn-54	9.99E-01	(11 +- 12)	E-1	39 E-1
AcTh228	1.00E 00	(-21 +- 66)	E-1	260 E-1
TeI-132	8.73E-01	(16 +- 91)	E-1	300 E-1
Fe-59	9.90E-01	(8 +- 28)	E-1	93 E-1
Zn-65	9.98E-01	(72 +- 34)	E-1	110 E-1
Co-60	1.00E 00	(-6 +- 18)	E-1	76 E-1
+ K -40	1.00E 00	(55 +- 21)	E 0	76 E 0
Sb-124	9.93E-01	(6 +- 34)	E-1	110 E-1

Notes:

+ Peak is found

Approved by

D.E. McCurdy
D.E. McCurdy

The quoted one-sigma uncertainty limits do not represent the propagation of all possible errors associated with the radioactive decay process (including statistics). Estimates of the additional systematic and random uncertainties are the calibration curve, ± 3 percent, sample positioning (source to detector), ± 2 percent, sample non-homogeneity, ± 10 percent, and sample self-absorption, ± 10 percent.

MAILED

YANKEE ATOMIC ELECTRIC COMPANY
ENVIRONMENTAL LABORATORY

Initial Analysis Report

Customer: Vermont Yankee Nuclear Power Corp.
Attention: MSX-ELAINE KEEGAN
MR. EDWARD CUMMING
MR. STEPHEN SKIBNIOWSKY

Report Date: 06/09/88
Analysis Date: 6 /8 /88
Date Received: 6 /8 /88
Reference Date: 6 /8 /88

Sludge

Sample Amount: 0.99 Kg.
Elapsed Time : 0.59 days

Lab Sample No.: G72972
Sample Submission Code: VSL 04 2388
Other Analysis Requested: None
Comment: MAIN TANK BOTTOM

NUCLIDE	DECAY CORRECTION	ACTIVITY		MDC
		CONC. +/- 1 SIGMA [Pico Curie / Kilogram -WET]		
Np-239	8.40E-01	(2 +- 21)	E 0	70 E 0
Co-57	9.98E-01	(-10 +- 19)	E-1	62 E-1
Ce-144	9.99E-01	(11 +- 14)	E 0	45 E 0
Ce-141	9.87E-01	(-12 +- 32)	E-1	110 E-1
Mo-99	8.63E-01	(20 +- 38)	E 0	120 E 0
Se-75	9.97E-01	(34 +- 29)	E-1	97 E-1
Cr-51	9.85E-01	(-25 +- 18)	E 0	61 E 0
I -131	9.50E-01	(3 +- 23)	E-1	78 E-1
Be-7	9.92E-01	(-20 +- 19)	E 0	65 E 0
Ru-103	9.90E-01	(-10 +- 24)	E-1	81 E-1
I -133	6.26E-01	(-25 +- 37)	E-1	120 E-1
Ba-140	9.68E-01	(-23 +- 11)	E 0	37 E 0
*+ Cs-134	9.99E-01	(130 +- 22)	E-1	48 E-1
Ru-106	9.99E-01	(-1 +- 28)	E 0	93 E 0
*+ Cs-137	1.00E 00	(1207 +- 52)	E-1	130 E-1
Ag-110M	9.98E-01	(-8 +- 53)	E-1	180 E-1
Zr-95	9.94E-01	(-77 +- 59)	E-1	200 E-1
Co-58	9.94E-01	(-11 +- 34)	E-1	110 E-1
*+ Mn-54	9.99E-01	(393 +- 43)	E-1	120 E-1
*+ AcTh228	1.00E 00	(39 +- 11)	E 0	32 E 0
TeI-132	8.81E-01	(-7 +- 39)	E 0	98 E 0
Fe-59	9.91E-01	(68 +- 73)	E-1	240 E-1
*+ Zn-65	9.98E-01	(527 +- 82)	E-1	220 E-1
*+ Co-60	1.00E 00	(853 +- 12)	E 0	14 E 0
*+ K -40	1.00E 00	(223 +- 35)	E 0	110 E 0
Sb-124	9.93E-01	(-12 +- 35)	E-1	120 E-1

Notes:

- * Activity greater than 3*standard deviation
- + Peak is found

Approved by

D.E. McCurdy
D.E. McCurdy.

The quoted one-sigma uncertainty terms do not represent the propagation of all possible errors associated with the radioactive decay process (counting statistics). Estimates of the additional systematic and random uncertainties are the calibration curve, ± 3 percent, sample positioning (source to detector), ± 3 percent, sample non-homogeneity, ± 10 percent, and sample self-absorption, ± 10 percent.

MAILED

YANKEE ATOMIC ELECTRIC COMPANY
ENVIRONMENTAL LABORATORY

Initial Analysis Report

Customer : Vermont Yankee Nuclear Power Corp.
Attention: MS/ELA/ELAINE KEEGAN
MR. EDWARD CUMMING
MR. STEPHEN SKUBNIEWSKY

Report Date: 06/20/88
Analysis Date: 6/15/88
Date Received: 6/14/88
Reference Date: 6/8/88

Septic-Solid

Sample Amount: 0.06 Kg.
Elapsed Time : 7.57 days

Lab Sample No.: G73075
Sample Submission Code: VSLs04 2388
Other Analysis Requested: None

Station No.: 04 Main Tank Bottom
ACTIVITY

NUCLIDE	DECAY CORRECTION	CONC. +- 1 SIGMA		MDC
		[Pico Curie / Kilogram]	
Np-239	1.07E-01	(55 +- 25)	E 2	85 E 2
Co-57	9.81E-01	(-9 +- 30)	E 0	99 E 0
Ce-144	9.82E-01	(3 +- 22)	E 1	74 E 1
Ce-141	8.51E-01	(137 +- 60)	E 0	190 E 0
Mo-99	1.51E-01	(-60 +- 29)	E 2	97 E 2
Se-75	9.57E-01	(85 +- 51)	E 0	170 E 0
Cr-51	8.27E-01	(14 +- 36)	E 1	120 E 1
I -131	5.21E-01	(-6 +- 74)	E 0	250 E 0
Be-7	9.06E-01	(12 +- 38)	E 1	130 E 1
Ru-103	8.75E-01	(-2 +- 46)	E 0	150 E 0
XI -133	2.49E-03	----		----
Ba-140	6.64E-01	(-86 +- 40)	E 0	130 E 0
** Cs-134	9.93E-01	(166 +- 52)	E 0	150 E 0
Ru-106	9.86E-01	(12 +- 49)	E 1	160 E 1
** Cs-137	1.00E 00	(3824 +- 92)	E 0	200 E 0
Ag-110M	9.79E-01	(76 +- 96)	E 0	320 E 0
Zr-95	9.22E-01	(-2 +- 11)	E 1	36 E 1
Co-58	9.29E-01	(12 +- 60)	E 0	200 E 0
** Mn-54	9.83E-01	(1126 +- 74)	E 0	200 E 0
** AcTh-228	1.00E 00	(76 +- 17)	E 1	49 E 1
TeI-132	1.99E-01	(-14 +- 22)	E 2	75 E 2
Fe-59	8.90E-01	(7 +- 14)	E 1	48 E 1
** Zn-65	9.79E-01	(120 +- 14)	E 1	40 E 1
** Co-60	9.37E-01	(2240 +- 22)	E 1	23 E 1
** K -40	1.00E 00	(472 +- 53)	E 1	160 E 1
Sb-124	9.17E-01	(69 +- 61)	E 0	200 E 0

Notes:

- * Activity greater than 3*standard deviation
- + Peak is found
- x Decay correction less than .01

Approved by

D. E. McCurdy

D. E. McCurdy.

The quoted one-sigma terms include only counting statistics and do not represent the propagation of all possible errors associated with the radioactive decay process. Estimates of the additional systematic and random uncertainties are: calibration curve, ± 5 percent, sample positioning, ± 2 percent, sample non-homogeneity, ± 10 percent, and sample self-absorption, ± 10 percent.

MAILED

YANKEE ATOMIC ELECTRIC COMPANY
ENVIRONMENTAL LABORATORY

JUL 20 1988

Initial Analysis Report

Customer: Vermont Yankee Nuclear Power Corp.
Attention: MS. ELAINE KEEGAN
MR. EDWARD CUMMING
MR. STEPHEN SKIBNIOWSKY

Report Date: 06/20/88
Analysis Date: 6 /15/88
Date Received: 6 /14/88
Reference Date: 6 /8 /88

Septic-Liquid Portion

Sample Amount: 1.00 Kg.
Elapsed Time : 7.57 days

Lab Sample No.: G73074
Sample Submission Code: VSL104 2388
Other Analysis Requested: None

Station No.: 04 Main Tank Bottom
ACTIVITY

NUCLIDE	DECAY CORRECTION	CONC. +- 1 SIGMA [Pico Curie / Kilogram]	MDC
Np-239	1.07E-01	(142 +- 88) E 0	290 E 0
Co-57	9.81E-01	(-16 +- 99) E-2	330 E-2
Ce-144	9.82E-01	(60 +- 73) E-1	240 E-1
Ce-141	8.51E-01	(15 +- 19) E-1	63 E-1
Mo-99	1.51E-01	(-123 +- 94) E 0	310 E 0
Se-75	9.57E-01	(-6 +- 15) E-1	50 E-1
Cr-51	8.27E-01	(3 +- 11) E 0	36 E 0
I -131	5.21E-01	(-6 +- 20) E-1	67 E-1
Be-7	9.06E-01	(37 +- 98) E-1	330 E-1
Ru-103	8.75E-01	(-5 +- 13) E-1	44 E-1
xI -133	2.48E-03	-----	-----
Ba-140	6.64E-01	(-23 +- 23) E-1	77 E-1
Cs-134	9.93E-01	(-9 +- 15) E-1	49 E-1
Ru-106	9.86E-01	(1 +- 12) E 0	41 E 0
Cs-137	1.00E 00	(21 +- 14) E-1	47 E-1
Ag-110M	9.79E-01	(-20 +- 18) E-1	69 E-1
Zr-95	9.22E-01	(29 +- 23) E-1	85 E-1
Co-58	9.29E-01	(20 +- 13) E-1	63 E-1
Mn-54	9.83E-01	(3 +- 13) E-1	42 E-1
AcTh228	1.00E 00	(28 +- 61) E-1	240 E-1
TeI-132	1.99E-01	(25 +- 37) E 0	120 E 0
Fe-59	8.90E-01	(-1 +- 25) E-1	82 E-1
Zn-65	9.79E-01	(31 +- 26) E-1	86 E-1
Co-60	9.97E-01	(-13 +- 22) E-1	86 E-1
+ K -40	1.00E 00	(34 +- 21) E 0	75 E 0
Sb-124	9.16E-01	(68 +- 30) E-1	100 E-1

Notes:

- + Peak is found
- x Decay correction less than .01

Approved by

D.E. McCurdy

D.E. McCurdy.

The quoted one-sigma terms include only counting statistics and do not represent the propagation of all possible errors associated with the radioactive decay process. Estimates of the additional systematic and random uncertainties are: calibration curve, ± 5 percent, and sample positioning, ± 2 percent.

MAILED

YANKEE ATOMIC ELECTRIC COMPANY
ENVIRONMENTAL LABORATORY --

Initial Analysis Report

JUL 14 1988

Customer: Vermont Yankee Nuclear Power Corp.
Attention: MS/ELAINE KEEGAN
ENVIRONMENTAL
MR. EDWARD CUMMING

Report Date: 07/11/88
Date Received: 6 /14/88

Septic-Solid

LAB. No. SAMPLE CODE	DATE		VOLUME Kg	NUCLIDE	ACTIVITY		MDC
	of REFERENCE ANALYSIS				CONC. +- 1 SIGMA [Pico Curie / KG - DRY]		
S73075	6 /8	7 /7	0.022	Sr-90	(-14 +- 37)E 0		40E 0
VSLs04 2388	Main Tank	Bottom		Sr-89	(52 +- 46)E 0		62E 0

Notes:

One-sigma terms include only counting statistics and do not represent the propagation of all possible errors associated with the radioactive decay process. Estimates of the additional systematic and random uncertainties are: calibration curve, ± 5 percent, and sample positioning, ± 2 percent.

Approved by

D E McCurdy

D.E. McCurdy.

MAILED

YANKEE ATOMIC ELECTRIC COMPANY
ENVIRONMENTAL LABORATORY

Initial Analysis Report

JUL 11 1988

Customer: Vermont Yankee Nuclear Power Corp.
Attention: MS ELAINE KELAN
ENVIRONMENTAL LAB.
MR. EDWARD CUMMING

Report Date: 07/11/88
Date Received: 6 /14/88

Septic-Liquid Portion

LAB. No. SAMPLE CODE	DATE		VOLUME Kg	NUCLIDE	ACTIVITY		MDC
	of REFERENCE ANALYSIS	of ANALYSIS			CONC. +- 1 SIGMA [Pico Curie / Kilogram]		
S73074	6 /8	6 /19	1.002	Sr-90	(113 +- 98)	E-2	200E-2
VSL104 2388	Main Tank	Bottom		Sr-89	(-10 +- 11)	E-1	22E-1

Notes:

Approved by

D E McCurdy

D. E. McCurdy.

The quoted one-sigma terms include only counting statistics and do not represent the propagation of all possible errors associated with the radioactive decay process. Estimates of the additional systematic and random uncertainties are: calibration curve, ± 5 percent, and sample positioning, ± 2 percent.

MAILED

YANKEE ATOMIC ELECTRIC COMPANY
ENVIRONMENTAL LABORATORY --

JUL 11 1988

Initial Analysis Report

YAEC

Customer: Vermont Yankee Nuclear Power Corp.
Attention: MS. ELAINE KEEGAN
MR. EDWARD CUMMING

Report Date: 07/11/88
Date Received: 6 /14/88

Septic-Liquid Portion

LAB. No. SAMPLE CODE	DATE		VOLUME Kg	NUCLIDE	ACTIVITY	
	of REFERENCE ANALYSIS				CONC. +- 1 SIGMA [Pico Curie / Kilogram]	MDC
H73074 VSL104 2388	6 /8	6 /20	0.003	H-3	(26 +- 15)E 1	49E 1
	Main Tank Bottom					

Notes:

The quoted one-sigma terms include only counting statistics and do not represent the propagation of all possible errors associated with the radioactive decay process. Estimates of the additional systematic and random uncertainties are: calibration curve, ± 5 percent, and sample positioning, ± 2 percent.

Approved by .

D. E. McCurdy

D. E. McCurdy.

APPENDIX B

Computer Code Outputs

This section contains copies of the computer code outputs employed in the calculation, as follows:

- B.1 ALLEGRA - Gamma Ray Spectra
- B.2 DIDOS-V - Dose Reduction as a Result of Plowing
- B.3 ATMODOS - Radiological Impact During VY Control of the Disposal Site
 - B.3.1 Impact due to Mn-54 in the Septage
 - B.3.2 Impact due to Co-60 in the Septage
 - B.3.3 Impact due to Zn-65 in the Septage
 - B.3.4 Impact due to Cs-134 in the Septage
 - B.3.5 Impact due to Cs-137 in the Septage
 - B.3.6 Impact due to All Nuclides in the Septage
- B.4 ATMODOS - Radiological Impact After Termination of Vermont Yankee Control of the Disposal Site (All Nuclides)
- B.5 ATMODOS - Unplowed-Land Dose Conversion Factors for Radiological Impact Assessment
 - B.5.1 Impact due to 1 uCi of Mn-54
 - B.5.2 Impact due to 1 uCi of Co-60
 - B.5.3 Impact due to 1 uCi of Zn-65
 - B.5.4 Impact due to 1 uCi of Cs-134
 - B.5.5 Impact due to 1 uCi of Cs-137

B.1 ALLEGRA - Gamma Ray Spectra

ALLEGRA (RADIOACTIVITY : GAMMA SPECTRA - ORIDEN-2 DATA BASE - ENTECH ENGINEERING, INC. - MOD 01 06/22/88) BB/06/30. PAGE 1

TAPE 4 IMPU... YA LISTING

CARD SEQ.	1	2	3	4	5	6	7	8
1	1234567890123456789012345678901234567890123456789012345678901234567890							
2	BY - DISPOSAL OF CONTAMINATEL. SEWAGE - FIELD ACTIVITY AFTER 40 TANK PUMFINGS							
3	1.0	1.0	0.0	0.0	1.0	0.0		
4	MR54	250540	2.831E-06					
5	CD60	270600	2.353E-04					
6	ZM65	300450	2.801E-06					
7	CS134	531340	1.457E-06					
8	CS137	531370	9.259E-05					
9	PA137M	561371	8.759E-05					
10								

USER-SPECIFIED PRINTOUT CONTROL FLAGS:

NUCLIDES IN LIBRARY : KPRINT(1) = 0
 DECAY DATA IN LIBRARY : KPRINT(2) = 1
 GAMMA SPECTRA IN LIBRARY : KPRINT(3) = 1
 CALCULATED ACTIVITIES : KPRINT(4) = 1
 ISOTOPE-SPECIFIC SPECTRA : KPRINT(5) = 1
 TOTAL GAMMA SPECTRA : KPRINT(6) = 1
 TAPE 11 CONTENTS : KPRINT(7) = 6
 TAPE 12 CONTENTS : KPRINT(8) = 0
 INTERMEDIATE DECAY RESULTS : KPRINT(9) = 2

DATA LIBRARY SELECTION OPTION = 2
 INPUT ACTIVITY UNIT CONTROL = 1
 GAMMA SPECTRA CONTROL FLAG = 1

SOURCE VOLUME (CUBIC METERS) - 1.0000E+00
 SOURCE INTENSITY INPUT MULTIPLIER - 1.0000E+00
 MIN. ISOTOPIC ACTIVITY FOR INCLUSION IN THE OUTPUT TABLES * .0000E+00

TOTAL NUMBER OF NUCLIDES IN THE INPUT * 6

LIST OF INPUT NUCLIDES AND ACTIVITIES (CURIES):

250540	2.831E-06	270600	2.353E-04	300650	2.801E-06	551340	1.457E-06	551370	9.259E-05	561371	8.759E-05
--------	-----------	--------	-----------	--------	-----------	--------	-----------	--------	-----------	--------	-----------

THERE IS CS137 AND/OR RA137M IN THE INPUT.
 CHECK IF BOTH NUCLIDES ARE IN THE INPUT AND THAT THE RA137M ACTIVITY IS 0.946 TIMES THAT OF CS137.

UY - DISPOSAL OF CONTAMINATED SEWAGE - FIELD ACTIVITY AFTER 40 TANK PUMPINGS

DECAY CONSTANTS (1/HR), BRANCHING FRACTIONS AND DAUGHTER PRODUCTS - TAFE 10 PREPARATION DATE: 88/06/22.

NUCLIDE	DECAY CONST	BETA-GRND	BETA-META	FOSI-GRND	FOSI-META	ISOMER.TR	ALPHA EM.	RETAINTRN	SFONT.FIS
HN 54	9.24198E-05	.000E+00	.000E+00	1.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00
CO 60	1.50050E-05	1.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00
ZN 65	1.18430E-04	NI 60 .000E+00	.000E+00	1.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00
CS134	3.83484E-05	1.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00
CS137	2.63582E-06	BA134 5.400E-02	9.460E-01	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00
BA137H	1.62967E+01	BA137 .000E+00	.000E+00	.000E+00	.000E+00	1.000E+00	.000E+00	.000E+00	.000E+00

TOTAL NUMBER OF RADIONUCLIDES IN THE DATA LIBRARY = 1030

UY - DISPOSAL OF CONTAMINATED SEWAGE - FIELD ACTIVITY AFTER 40 TANK PUMFINGS

PHOTON SPECTRA (PHOTONS/PIS AT GIVEN ENERGY) - TAPE 10 PREPARATION DATE: BB/06/22.

NUCLIDE	.0100 MEV	.0250 MEV	.0375 MEV	.0650 MEV	.1250 MEV	.2250 MEV	.3750 MEV	.5750 MEV
	.850 MEV	1.250 MEV	1.750 MEV	2.750 MEV	3.500 MEV	5.000 MEV	7.000 MEV	9.000 MEV
HN 54	1.380E-01	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00
	9.820E-01	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00
CO 60	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00
	7.370E-03	2.000E+00	.000E+00	3.280E-08	.000E+00	.000E+00	.000E+00	.000E+00
ZN 65	3.160E-01	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	2.590E-02
	2.720E-05	4.530E-01	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00
CS134	3.970E-04	.000E+00	6.580E-03	.000E+00	.000E+00	2.270E-04	1.250E-04	1.270E+00
	8.820E-01	5.830E-02	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00
CS137	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00
	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00
BA137H	4.740E-03	.000E+00	6.510E-02	.000E+00	.000E+00	.000E+00	.000E+00	1.030E+00
	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00

TOTAL NUMBER OF GAMMA SPECTRA IN THE DATA LIBRARY = 435

UY - DISPOSAL OF CONTAMINATED SEWAGE - FIELD ACTIVITY AFTER 40 TANK PUMPINGS

DECAYED RADIOACTIVITY (CURIES) AS A FUNCTION OF DECAY TIME (HRS)

NUCLIDE	.0000E+00
MN 54	2.8310E-06
CO 60	2.3530E-04
ZN 65	2.8010E-06
CS134	1.4570E-06
CS137	9.2590E-05
RA137M	8.7590E-05

ALLEGRA (RADIOACTIVITY & GAMMA SPECTRA - ORIGEN-2 DATA BASE - ENTECH ENGINEERING, INC. - MOD 01 06/27/88)
 VY - DISPOSAL OF CONTAMINATED SEWAGE - FIELD ACTIVITY AFTER 40 TANK PUMFINGS

RADIOACTIVITY TOTALS (CURIES) AS A FUNCTION OF DECAY TIME (HRS)

T (HRS)	NUCLIDES WITH ACTIVITY > .000E+00 (USER-SPECIFIED LIMIT)			ALL NUCLIDES ENCOUNTERED IN THE ANALYSIS		
	OTHER	TOTAL	GIVEN MIX	OTHER	TOTAL	GIVEN MIX
.000E+00	.0000E+00	4.2257E-04	4.2257E-04	.0000E+00	4.2257E-04	4.2257E-04

NUCLIDE MIX CONSISTS OF .0 % NORLE BASES, .0 % HALOGENS, AND 100.0 % OTHERS

U-235 - DISPOSAL OF CONTAMINATED SEWAGE - FIELD ACTIVITY AFTER 40 TANK PUMPINGS

NUCLIDE	ACTIVITY (CURIES)	18-GROUP GAMMA ENERGY RELEASE RATES (MEV/SEC) AT T - .000E+00 HOURS										TOTAL	
		.010 MEV	.025 MEV	.0375 MEV	.0575 MEV	.0850 MEV	.1250 MEV	.1750 MEV	.2500 MEV	.3500 MEV	.5000 MEV		
HM 54	2.831E-06	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	8.728E+04
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
CD 60	2.353E-04	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	2.177E+07
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
IN 65	2.801E-04	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	6.056E+04
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
CS134	1.457E-04	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	8.373E+04
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
CS137	9.259E-05	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
BA137M	8.759E-05	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	1.927E+06
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	
		.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	.000E+00	

UY - DISPOSAL OF CONTAMINATED SEWAGE - FIELD ACTIVITY AFTER 40 TANK PUMFINGS
 GAMMA ENERGY RELEASE RATES (MEV/SEC) - ALL NUCLIDES OTHER THAN NOBLE GASES AND HALOGENS - (0, 0, 100)
 AS A FUNCTION OF DECAY TIME (HOURS) - DRIGEN-2 GAMMA ENERGY GROUPS

E (MEV)	.0000E+00
.0100	6.2587E+02
.0250	.0000E+00
.0375	7.9250E+03
.0575	.0000E+00
.0850	.0000E+00
.1250	.0000E+00
.1750	2.2534E+00
.3750	2.3311E+02
.5750	1.9603E+06
.8500	1.2840E+05
1.7500	2.1828E+07
1.7500	.0000E+00
2.2500	2.0764E+02
2.7500	7.8529E-01
3.5000	.0000E+00
5.0000	.0000E+00
7.0000	.0000E+00
9.0000	.0000E+00
TOTAL	2.3926E+07

UY - DISPOSAL OF CONTAMINATED SEWAGE - FIELD ACTIVITY AFTER 40 TANK PUMPINGS

GRAND TOTAL GAMMA ENERGY RELEASE RATES (REV/SEC) - NOBLE GASES, HALOGENS AND OTHERS - (100,100,100)

AS A FUNCTION OF DECAY TIME (HOURS) - ORIGEN-2 GAMMA ENERGY GROUPS

E (HEV)	
.0100	.0000E+00
.0250	6.2557E+02
.0375	.0000E+00
.0575	7.9250E+03
.0850	.0000E+00
.1250	.0000E+00
.2250	2.7534E+00
.3750	2.3311E+02
.5750	1.9603E+06
.8500	1.2840E+05
1.2500	2.1878E+07
1.7500	.0000E+00
2.2500	2.0764E+02
2.7500	7.8529E-01
3.5000	.0000E+00
5.0000	.0000E+00
7.0000	.0000E+00
9.0000	.0000E+00
TOTAL	2.3927E+07

BY - DISPOSAL OF CONTAMINATED SEWAGE - FIELD ACTIVITY AFTER 40 TANK PUMPINGS

GAMMA ENERGY RELEASE RATES (REV/SEC) - USER-SPECIFIED MIX - (MOD. HALF TIME) = (.0, .0+100.0)
 AS A FUNCTION OF DECAY TIME (HOURS) - ORIGEN-2 GAMMA ENERGY GROUPS

E (MEV)	
.0100	.0000E+00
.0250	6.2587E+02
.0375	.0000E+00
.0575	7.9250E+03
.0850	.0000E+00
.1250	.0000E+00
.1750	2.7534E+00
.2500	2.3311E+02
.3750	1.9603E+06
.5000	1.2840E+05
.7500	2.1828E+07
1.0000	.0000E+00
1.5000	2.0764E+02
2.0000	7.8529E-01
3.0000	.0000E+00
5.0000	.0000E+00
7.0000	.0000E+00
9.0000	.0000E+00
TOTAL	2.3926E+07

***** END OF ANALYSIS *****

B.2 DIDOS-V - Dose Reduction as a Result of Plowing

21005-V (CYLINDRICAL RADIATION SOURCE DOSIMETRY) - ENTECH ENGINEERING/YANKEE ATOMIC - (MOD 01 - 10/15/64) 68/07/02, PAGE 1

INPUT DATA LISTING - TAPE 4

CARD	1	2	3	4	5	6	7	8
NO.	123456789012345678901234567890123456789012345678901234567890							
	UY - DISP. OF CONTAIN. SEWAGE -	ACTIV. FROM 30 FUMFOUTS/2	ACRES - UNFLOWED LAND					
1	1	0	0	0	0.1236	3.0	50.0	
2	1	0	0	0	0.001			
3	1	0	0	0	0.001			
4	1	0	0	0	0.001			
5	1.001	-1.0						

INPUT DATA LISTING - TAPE 11

CARD	1	2	3	4	5	6	7	8
SEQ.	123456789012345678901234567890123456789012345678901234567890							
1	VY - DISPOSAL OF CONTAMINATED SEMAGE - FIELD EQUILIBRIUM ACTIVITY (MEV/SEC)							
2	TIME (HRS)							
3	3.750E-02	3.750E-01	3.750E-01	8.000E-01	1.250E+00	2.500E+00		
4	7.925E+03	2.331E+02	1.560E+06	1.284E+03	2.183E+07	2.076E+02		

UY - DISP. OF CONTAM. SEWAGE - ACTIV. FROM 40 FEET/FOOTS/2 ACRES - UNFLOUED LAND

RECEPTOR 1 TIME (HRS) = .00

** SOURCE DESCRIPTION

UFRIGHT CIRCULAR CYLINDER	MATERIAL	AIR
RADIUS (METERS) = 1.500E+02	HEIGHT (METERS) =	1.000E-03
VOLUME (CUB.H.) = 7.069E+01	DENSITY (G/CC) =	1.293E-03
MINIMUM SOURCE INTENSITY FOR INDEPENDENT ANALYSIS =		.000E+00
SOURCE INTENSITY INFUT MULTIPLIER =		1.236E-01
MAX ENERGY DIFF FOR INDEPENDENT ANALYSIS (PERCENT) =		3.000E+00

** CONTAINER	MATERIAL	DENSITY (G/CC)	RADIUS (M)	HEIGHT (M)	RAD. THICKNESS (M)
	NONE				

** SHIELD SLABS	MATERIAL	DENSITY (G/CC)	THICKNESS (M)	ANGLE (DEG)	DELTA
AIR		1.293E-03			

** DOSE POINT DESCRIPTION

RECEPTOR ON AXIS

2-DIMENSIONAL ANALYSIS

ELEVATION RELATIVE TO LOWER END OF SOURCE (METERS) = 1.001E+00

** RESULTS

GAMMA GROUP	ENERGY (MEV)	ADJ. INTENS (REV/S-R3)	SOURCE ATT COEF (1/M)	CONTAINER MUST RADIAL - AXIAL	SHIELD MUST	OVERALL BUILDUP	INTRNL VAL WITH PLDUP	DOSE TO AIR (R/HR)
1	.038	9.795E+02	3.423E-02	.000	.000	1.737E+00	2.443E-03	1.009E-09
2	.375	2.681E+01	1.266E-02	.000	.000	1.330E+00	2.484E-03	1.211E-11
3	.575	2.423E+05	1.040E-02	.000	.000	1.241E+00	2.408E-03	9.941E-08
4	.850	1.587E+04	8.887E-03	.000	.000	1.183E+00	2.375E-03	6.232E-09
5	1.250	2.698E+06	7.355E-03	.000	.000	1.139E+00	2.367E-03	9.783E-07
6	2.220	2.566E+01	5.598E-03	.000	.000	1.072E+00	2.367E-03	8.052E-12
TOTAL								1.085E-06

MAXIMUM PATH-LENGTH USED IN ANALYSIS (MFF) = 50.0

***** END OF PROBLEM *****

CARD
 SEQ. 1 2 3 4 5 6 7 8
 1234567890123456789012345678901234567890123456789012345678901234567890

INPUT DATA LISTING - TAPE 4
 UY - DISP. OF CONTACT. SEWAGE - ACTIV. FROM 40 PUFFOUTS/2 ACRES - FLOWED LAND
 1 1 0 6 1 2 1 0.0 6.237E-04 3.0 50.0
 2 3 1.6 150.0 .15
 3 3 1.6 150.0 .15
 4 1.15 -1.0

INPUT DATA LISTING - TAPE 11

CARD	1	2	3	4	5	6	7	8
NO.	1134567890123456789012345678901234567890123456789012345678901234567890							
1	UY - DISPOSAL OF CONTAMINATED SEWAGE - FIELD EQUILIBRIUM ACTIVITY (MEV/SEC)							
2	TIME (HRS) .000E+00							
3	3.750E-02	3.750E-01	5.750E-01	8.500E-01	1.250E+00	2.750E+00		
4	7.925E+03	2.831E+02	1.966E+06	1.289E+05	2.187E+07	2.078E+02		

WY - DISP. OF CONTAIN. SEWAGE - ACTIV. FROM 40 PUMPOUTS/2 ACRES - FLOWED LAND

RECEPTOR 1 TIME (HRS) = .00

** SOURCE DESCRIPTION

UPRIGHT CIRCULAR CYLINDER	MATERIAL	CONCRETE
RADIUS (METERS) = 1.500E+02	HEIGHT (METERS)	1.500E+01
VOLUME (CUB.M.) = 1.060E+04	DENSITY (G/CC)	1.600E+00
MINIMUM SOURCE INTENSITY FOR INDEPENDENT ANALYSIS		.000E+00
SOURCE INTENSITY INPUT MULTIPLIER		8.237E-03
MAX ENERGY DIFF FOR INDEPENDENT ANALYSIS (PERCENT)		3.000E+00

** CONTAINER MATERIAL RADIUS (M) HEIGHT (M) RAD. THICKNESS (M)

NONE

** SHIELD SLABS MATERIAL THICKNESS (M) ANGLE (DEG) DELTA

AIR 1.295E-03

** DOSE POINT DESCRIPTION RECEPTOR ON AXIS Z-DIMENSIONAL ANALYSIS

ELEVATION RELATIVE TO LOWER END OF SOURCE (METERS) = 1.150E+00

** RESULTS

GAMMA GROUP	ENERGY (MEV)	ARJ. INTENS (MEV/S-M3) COEF (1/M)	SOURCE ATT COEF (1/M)	CONTAINER MUET RADIAL - AXIAL	SHIELD MUET	OVERALL RUTLDUP	INTGR VAL WITH BLDUP	DOSE TO AIR (R/HR)
1	.338	6.528E+00	1.114E+02	.000	.000	1.571E+00	6.181E-03	1.701E-11
2	.375	1.920E+01	1.585E+01	.000	.000	2.598E+00	7.577E-02	2.465E-12
3	.575	1.618E+03	1.321E+01	.000	.000	2.274E+00	7.660E-02	2.165E-08
4	.850	1.052E+02	1.102E+01	.000	.000	2.014E+00	8.198E-02	1.434E-09
5	1.250	1.798E+04	9.109E+00	.000	.000	1.812E+00	8.689E-02	2.398E-07
6	2.250	1.710E-01	6.745E+00	.000	.000	1.576E+00	9.613E-02	2.179E-12
TOTAL								2.629E-07

MAXIMUM PATH-LENGTH USED IN ANALYSIS (MFP) = 50.0

***** END OF PROBLEM *****

B.3 ATMODOS - Radiological Impact During VY Control of the Disposal Site

Presented below is a partial listing of one of the ATMODOS Tape 5 inputs used in this portion of the calculation. It corresponds to the output in Sec. B.3.6 below. With the exception of the nuclide data library, a large portion of which was deleted due to space limitations, the listing is complete.

The Tape 5 listings corresponding to the outputs in Sec. B.3.1 through B.3.5 are similar, except that a single isotope was analyzed in each case. Also, the tables with the detailed pathway/isotope/organ exposures were excluded from the output.

```
1 1VYPI VY STANDARD PROGRAM INFORMATION FILE
VY - SEWAGE CONTAM - SOLIDS 1% BY WT - 2 ACRES - SHIELD F = 0.012 - 104 HR OCCUP
1. .200 44.56FRESH .500YESNO NO 0..012 1.NO NO NO 0.1.00
```

--EOR--

--EOF--

```
0 2VYGASRLGAS STK VY 87 1 1 0 87063023:
MN54 4.132E-6 25 MN 54
CO60 2.511E-4 27 CO 60
ZN65 4.502E-6 30 ZN 65
CS134 1.715E-6 55 CS 134
CS137 9.366E-5 55 CS 137
```

--EOR--

--EOF--

```
1 5QUERY PATHWAYS: SHORE(1,2,3),BND,ROAD,RES1,RAD,MEAT,GOAT,COW
SHORE1 YESYES
SHORE2 YESYES
SHORE3 YESYES
BOUND YESYES
ROAD YESYES
RES1 YESYESYESYES
RADIUS YESYES
MEAT YESYESYESYES YESYESYES
GOAT YESYESYESYES YES YESYES
COW YESYESYESYESYES YESYES
```

--EOR--

--EOF--

1 6VYUFMAXVY MAXIMUM INDIVIDUAL USAGE FACTORS FOR STANDARD PROBLEMS

520.00	64.00	310.00	110.00	21.00	.00	.00	12.00	.00	8000.00
630.00	42.00	400.00	65.00	16.00	.00	.00	67.00	.00	8000.00
520.00	26.00	330.00	41.00	6.90	.00	.00	14.00	.00	3700.00
.00	.00	330.00	.00	.00	.00	.00	.00	.00	1400.00

--EOR--

--EOF--

1 7VYGS D VY MAX INDIVIDUAL GAS SITE DATA FILE FOR STANDARD PROBLEMS

2.00	2.00	.70	2.00	.70	2.00	.70	2.00	2.00
240.00	240.00	240.00	240.00	240.00	240.00	240.00	240.00	240.00
.00	.00	48.00	48.00	48.00	48.00	48.00	480.00	480.00
8766.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00
0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
1440.00	24.00	.00	2160.00	.00	2160.00	.00	2160.00	2160.00
.00	.00	50.00	50.00	6.00	6.00	50.00	50.00	50.00
.00	.00	.50	.00	.50	.00	.50	.00	.00
.00	.00	1.00	.00	1.00	.00	1.00	.00	.00
1.0	1.00	.00	.00	.00	.00	.00	.00	.00
5.60	.00	.00	.00	.00	.00	.00	.00	.00

--EOR--

--EOF--

1 81STPNBLNUCLIDE LIBRARY FOR ALL DOSE PROGRAMS

89

1 3 1.78E-090.00E+000.00E+00 SOLUBLE
H 9.0E-01 9.0E-01 9.0E-01 9.0E-01 9.3E-01 9.3E-01 4.8E-00 1.0E-02 1.2E-02
0. 1.05E-071.05E-071.05E-071.05E-071.05E-071.05E-07
0. 1.58E-071.58E-071.58E-071.58E-071.58E-071.58E-07
0. 1.06E-071.06E-071.06E-071.06E-071.06E-071.06E-07
0. 1.59E-071.59E-071.59E-071.59E-071.59E-071.59E-07
0. 2.03E-072.03E-072.03E-072.03E-072.03E-072.03E-07
0. 3.04E-073.04E-073.04E-073.04E-073.04E-073.04E-07
0. 3.08E-073.08E-073.08E-073.08E-073.08E-073.08E-07
0. 4.62E-074.62E-074.62E-074.62E-074.62E-074.62E-07

6 14 3.83E-12 SOLUBLE
C 4.6E+03 9.1E+03 4.6E+03 1.8E+03 1.4E+03 1.8E+03 5.5E+00 1.2E-02 3.1E-02
2.84E-065.68E-075.68E-075.68E-075.68E-075.68E-075.68E-07
2.27E-064.26E-074.26E-074.26E-074.26E-074.26E-074.26E-07
4.06E-068.12E-078.12E-078.12E-078.12E-078.12E-078.12E-07
3.25E-066.09E-076.09E-076.09E-076.09E-076.09E-076.09E-07
1.21E-052.42E-062.42E-062.42E-062.42E-062.42E-062.42E-06
9.70E-061.82E-061.82E-061.82E-061.82E-061.82E-061.82E-06
2.37E-055.06E-065.06E-065.06E-065.06E-065.06E-065.06E-06
1.89E-053.79E-063.79E-063.79E-063.79E-063.79E-063.79E-06
11 24 1.28E-052.50E-08 2.90E-08 SOLUBLE

ETC (FOR A TOTAL OF 89 NUCLIDES)

--BOR--

--BOF--

1 3VYXQF VY X/QFILE - SPECIAL VALUES - SEWAGE CONTAM. PROBLEM - 2 ACRE PLOTS
FLD 0 COW MEAT 4.626E-04 4.626E-04 1.236E-04 4.626E-04

--BOR--

--BOF--

B.3.1 Impact due to Mn 54 in the Septage

PROGRAM ATHODOS
YANKEE ATOMIC ELECTRIC COMPANY
DEC, 1983 REV. 7

REGULATORY GUIDE 1.109, APPENDIX C
MODELS FOR CALCULATING DOSE VIA
ADDITIONAL PATHWAYS FROM RADIOIODINES
AND OTHER RADIONUCLIDES DISCHARGED
TO THE ATMOSPHERE NOV, 1977 LIBRARY

SECTOR FLD 0 (METERS)
DISTANCE - 4.63E-04 (SEC/H-3)
X/D - 4.63E-04 (SEC/H-3)
Z/D DEPLETED - 4.63E-04 (SEC/H-3)
DELTA - 1.24E-04 (1/H-2)

WY - SEWAGE CONTAIN - SOLIDS 12 BY WT - 2
ACRES - SHIELD F - 0.012 - 104 HR OCCUP

THE FOLLOWING 1 NUCLIDES WERE USED IN THIS CALCULATION

NUCLIDE RELEASE
CURIES
25 MN 54 4.13E-06

6 PATHWAYS CONSIDERED

GROUND FLARE YES
INHALATION YES
STORED VEGETABLES YES
LEAFY VEGETABLES YES
COW MILK YES
GOAT MILK NO
MEAT YES

VARIABLE	VEGETABLES STORED	LEAFY	COW MILK PASTURE	GOAT MILK PASTURE	MEAT STORED
YV AGRICULTURAL PRODUCTIVITY (KG/H-2)	2.00	2.00	.70	.70	2.00
F SOIL SURFACE DENSITY (KG/H-2)	240.00	240.00	240.00	240.00	240.00
T TRANSPORT TIME TO USER (HRS)	8766.00	8766.00	8766.00	8766.00	8766.00
TR SOIL EXPOSURE TIME (HRS)	.00	.00	.00	.00	.00
TE CROP EXPOSURE TIME TO FLUME (HRS)	1440.00	24.00	.00	.00	.00
TH HOLDUP AFTER HARVEST (HRS)			50.00	50.00	50.00
DF ANIMALS DAILY FEED (KG/PAY)			.50	.50	.50
FF FRACTION OF YEAR ON PASTURE			1.00	1.00	1.00
FS FRACTION PASTURE WHEN ON PASTURE					
FG FRACTION OF STORED VEG GROWN IN GARDEN	1.00				
FL FRACTION OF LEAFY VEG GROWN IN GARDEN		1.00			
F1 FRACTION ELEMENTAL IODINE - .500					
H ABSOLUTE HUMIDITY - 5.60 (G/H-3)					
FC FRACTIONAL EQUILIBRIUM RATIO FOR C-14 - 1.000					

AGE	VEG (KG/YR)	MILK (L/YR)	MEAT (KG/YR)	INHALATION (H-3/YR)	VEG (KG/YR)	MILK (L/YR)	MEAT (KG/YR)	INHALATION (H-3/YR)	WHOLE BODY	SKIN
ADULT	520.00	68.00	310.00	110.00	2.13E-04	3.18E-04	2.44E-04	2.98E-04	2.13E-04	2.33E-04
TEEN	630.00	42.00	400.00	65.00	2.13E-04	3.66E-04	2.59E-04	3.34E-04	2.13E-04	2.44E-04
CHILD	520.00	26.00	330.00	41.00	2.13E-04	4.36E-04	2.76E-04	3.09E-04	2.13E-04	2.73E-04
INFANT	.00	.00	330.00	.00	2.13E-04	2.18E-04	2.14E-04	2.74E-04	2.13E-04	2.14E-04

B.3.2 Impact due to Co 60 in the ¹³⁷Cs pathway

PROGRAM: YANKEE
 YANKEE ATOMIC ELECTRIC COMPANY
 REC. 1980 REV. 7

REGULATORY GUIDE 1.109, APPENDIX C
 MODELS FOR CALCULATING DOSE VIA
 ADDITIONAL PATHWAYS FROM RADIOIODINES
 AND OTHER RADIONUCLIDES DISCHARGED
 TO THE ATMOSPHERE NOV. 1977 LIBRARY

SECTOR FLD 0 (METERS)
 DISTANCE - 4.63E-04 (SEC/H-3)
 X/D - 4.63E-04 (SEC/H-3)
 X/D DELETED - 4.63E-04 (SEC/H-3)
 DELTA - 1.24E-04 (1/H-2)
 VY - SEWAGE CONTACT - SOLID 12 BY WT - 2
 ACKES - SHIELD F - 6.012 - 104 HR OCCUP

THE FOLLOWING 1 NUCLIDES WERE USED IN THIS CALCULATION

NUCLIDE RELEASE
 27 CO 60 2.31E-04

6 PATHWAYS CONSIDERED
 GROUND FLARE YES
 IMMALATION YES
 STORED VEGETABLES YES
 LEAFY VEGETABLES YES
 COW MILK YES
 GOAT MILK NO
 MEAT YES

VARIABLE	VEGETABLES STORED	LEAFY STORED	FASTURE	COW MILK	GOAT MILK	FASTURE	MEAT STORED
TV AGRICULTURAL PRODUCTIVITY (KG/H-2)	2.00	2.00	2.00	2.00	2.00	2.00	2.00
F SOIL SURFACE DENSITY (KG/H-2)	240.00	240.00	240.00	240.00	240.00	240.00	240.00
T TRANSPORT TIME TO USER (HRS)	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00
TR SOIL EXPOSURE TIME TO FLUME (HRS)	.00	.00	.00	.00	.00	.00	.00
TE CROP EXPOSURE TIME TO FLUME (HRS)	1440.00	24.00	50.00	2160.00	2160.00	2160.00	2160.00
TH HOLDUP AFTER HARVEST (HRS)				.50	.50	.50	.50
QF ANIMALS DAILY FEED (KG/DAY)			1.00			1.00	
FF FRACTION OF YEAR ON PASTURE							
FS FRACTION PASTURE WHEN ON PASTURE							
FG FRACTION OF STORED VEG GROWN IN GARDEN	1.00						
FL FRACTION OF LEAFY VEG GROWN IN GARDEN		1.00					
FI FRACTION ELEMENTAL IODINE - .500							
H ABSOLUTE HUMIDITY - 5.60 (GM/H-3)							
FC FRACTIONAL EQUILIBRIUM RATIO FOR C-14 - 1.000							

AGE	VEG (KG/YR)	LEAFY VEG (KG/YR)	MILK (LI/YR)	MEAT (KG/YR)	IMMALATION (H-3/YR)	BI-OME	LIVER	KIDNEY	LUNG (MREN)	BI-LLI	THYROID	WHOLE BODY	SKIN
ADULT	520.00	64.00	310.00	110.00	8000.00	5.20E-02	5.37E-02	5.20E-02	7.30E-02	8.32E-02	5.20E-02	5.20E-02	6.12E-02
TEEN	630.00	42.00	400.00	65.00	8000.00	5.20E-02	5.45E-02	5.20E-02	8.41E-02	8.30E-02	5.20E-02	5.20E-02	6.12E-02
CHILD	520.00	26.00	330.00	41.00	3700.00	5.20E-02	5.52E-02	5.20E-02	7.80E-02	7.18E-02	5.20E-02	5.20E-02	6.12E-02
INFANT	.00	.00	330.00	.00	1400.00	5.20E-02	5.22E-02	5.20E-02	6.86E-02	5.26E-02	5.20E-02	5.20E-02	6.12E-02

B.3.3 Impact due to Zn 65 in the Septage

PROGRAM ATHOROS
YANKEE ATOMIC ELECTRIC COMPANY
DEC. 1985 REV. 7

REGULATORY GUIDE 1.109, APPENDIX C
MODELS FOR CALCULATING DOSE VIA
ADDITIONAL PATHWAYS FROM RADIOIODINES
AND OTHER RADIONUCLIDES DISCHARGED
TO THE ATMOSPHERE NOV. 1977 LIBRARY

SECTOR = FLD 0 (METERS)
DISTANCE = 4.63E-04 (SEC/M-3)
X/D DEPLETED = 4.63E-04 (SEC/M-3)
DELTA = 1.24E-04 (1/M-2)
VY - SEWAGE CONTAM - SOLIDS 12 BY WT - 2
ACRES - SHIELD F - 0.012 - 104 HR OCCUR

THE FOLLOWING 1 NUCLIDES WERE USED IN THIS CALCULATION

NUCLIDE	RELEASE	6 PATHWAYS CONSIDERED
30 IN 65	CURIES 4.50E-06	GROUND FLAME YES
		INHALATION YES
		STORED VEGETABLES YES
		LEAFY VEGETABLES YES
		COW MILK YES
		GOAT MILK NO
		MEAT YES

VARIABLE	VEGETABLES STORED	VEGETABLES LEAFY	COW MILK PASTURE	COW MILK STORED	GOAT MILK PASTURE	GOAT MILK STORED	MEAT FASTURE	MEAT STORED
YV AGRICULTURAL PRODUCTIVITY (KG/M-2)	2.00	2.00	.70	2.00	.70	2.00	.70	2.00
P SOIL SURFACE DENSITY (KG/M-2)	240.00	240.00	240.00	240.00	240.00	240.00	240.00	240.00
T TRANSPORT TIME TO USER (HRS)	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00
TR SOIL EXPOSURE TIME TO FLUME (HRS)	.00	.00	.00	.00	.00	.00	.00	.00
TE CROP EXPOSURE TIME TO FLUME (HRS)	1440.00	24.00	.00	2160.00	.00	2160.00	.00	2160.00
TH HOLDUP AFTER HARVEST (HRS)	.50	.50	.50	50.00	.50	50.00	.50	50.00
OF ANIMALS DAILY FEED (KG/DAY)	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
FF FRACTION OF YEAR ON PASTURE								
FS FRACTION PASTURE WHEN ON PASTURE								
FG FRACTION OF STORED VEG GROWN IN GARDEN	1.00	1.00						
FL FRACTION OF LEAFY VEG GROWN IN GARDEN								
FI FRACTION ELEMENTAL IODINE = .500								
M ABSOLUTE HUMIDITY = 5.60 (GM/M-3)								
FC FRACTIONAL EQUILIBRIUM RATIO FOR C-14 = 1.000								

AGE	VEG (KG/YR)	LEAFY VEG (KG/YR)	MILK (LI/YR)	MEAT (KG/YR)	INHALATION (M-3/YR)	BONE	LIVER	KIDNEY	LUNG (MMEM)	GI-LLI	THYROID	WHOLE BODY	SKIN
ADULT	520.00	68.00	310.00	110.00	8000.00	3.43E-03	1.04E-02	7.13E-03	2.03E-04	6.72E-03	1.46E-04	4.87E-03	1.68E-04
TEEN	630.00	42.00	400.00	65.00	8000.00	8.41E-03	1.57E-02	1.01E-02	2.28E-04	6.72E-03	1.46E-04	7.38E-03	1.68E-04
CHILD	520.00	26.00	330.00	41.00	3700.00	8.72E-03	2.30E-02	1.45E-02	2.11E-04	4.16E-03	1.46E-04	1.44E-02	1.68E-04
INFANT	.00	.00	330.00	.00	1400.00	6.18E-03	2.08E-02	1.05E-02	1.88E-04	1.75E-02	1.46E-04	9.69E-03	1.68E-04

USAGE FACTORS

DOSE DELIVERED TO EACH ORGAN FROM ALL PATHWAYS COMBINED

B.3.4 Impact due to Cs 134 in the Septage

PROGRAM ATMOS
 YANKEE ATOMIC ELECTRIC COMPANY
 DEC, 1985 REV. 7

REGULATORY GUIDE 1.109, APPENDIX C
 MODELS FOR CALCULATING DOSE VIA
 ADDITIONAL PATHWAYS FROM RADIOIODINES
 AND OTHER RADIONUCLIDES DISCHARGED
 TO THE ATMOSPHERE NOV. 1977 LIBRARY

SECTOR = FLD 0 (METERS)
 DISTANCE = 4.63E-04 (SEC/M-3)
 X/O = 4.63E-04 (SEC/M-3)
 X/O DEPLETED = 4.63E-04 (SEC/M-3)
 DELTA = 1.24E-04 (1/M-2)
 UY - SEWAGE CONTAM - SOLIDS 12 BY WT - 2
 ACRES - SHIELD F = 0.012 - 104 HR OCCUP

THE FOLLOWING 1 NUCLIDES WERE USED IN THIS CALCULATION

NUCLIDE RELEASE
 CS 134 1.72E-06

6 PATHWAYS CONSIDERED
 GROUND FLAME YES
 INHALATION YES
 STORED VEGETABLES YES
 LEAFY VEGETABLES YES
 COW MILK YES
 GOAT MILK NO
 MEAT YES

VARIABLE	VEGETABLES STORED	VEGETABLES LEAFY	COW MILK FASTURE	COW MILK STORED	GOAT MILK FASTURE	GOAT MILK STORED	MEAT FASTURE	MEAT STORED
YU AGRICULTURAL PRODUCTIVITY (KG/M-2)	2.00	2.00	.70	2.00	.70	2.00	.70	2.00
P SOIL SURFACE DENSITY (KG/M-2)	240.00	240.00	240.00	240.00	240.00	240.00	240.00	240.00
T TRANSFERT TIME TO USER (HRS)	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00
TR SOIL EXPOSURE TIME (HRS)	.00	.00	.00	.00	.00	.00	.00	.00
TE CROP EXPOSURE TIME TO FLUME (HRS)	1440.00	24.00	.00	2160.00	.00	2160.00	.00	2160.00
TH HOLDUP AFTER HARVEST (HRS)			50.00	50.00	.50	6.00	50.00	50.00
DF ANIMALS DAILY FEED (KG/DAY)			1.00	1.00	.50	1.00	.50	1.00
FP FRACTION OF YEAR ON PASTURE								
FS FRACTION PASTURE WHEN ON PASTURE	1.00							
FG FRACTION OF STORED VEG GROWN IN GARDEN		1.00						
FL FRACTION OF LEAFY VEG GROWN IN GARDEN								
FI FRACTION ELEMENTAL IODINE = .500								
H ABSOLUTE HUMIDITY = 5.60 (GM/M-3)								
FC FRACTIONAL EQUILIBRIUM RATIO FOR C-14 = 1.000								

AGE	VEG (KG/YR)	LEAFY (KG/YR)	MILK (LI/YR)	MEAT (KG/YR)	INHALATION (M-3/YR)	BONE	LIVER	KIDNEY	LUNG (HREM)	GI-LLI	THYROID	WHOLE BODY	SKIN
ADULT	520.00	84.00	310.00	110.00	8000.00	5.87E-04	1.09E-03	5.04E-04	3.20E-04	2.42E-04	2.27E-04	9.31E-04	2.65E-04
TEEN	630.00	42.00	400.00	65.00	8000.00	7.92E-04	1.58E-03	6.50E-04	3.89E-04	2.44E-04	2.27E-04	8.44E-04	2.65E-04
CHILD	520.00	26.00	330.00	41.00	3700.00	1.50E-03	2.31E-03	8.74E-04	4.59E-04	2.38E-04	2.27E-04	6.67E-04	2.65E-04
INFANT	.00	.00	330.00	.00	1400.00	7.74E-04	1.25E-03	4.89E-04	3.35E-04	2.30E-04	2.27E-04	3.30E-04	2.65E-04

DOSE DELIVERED TO EACH ORGAN FROM ALL PATHWAYS COMBINED

USAGE FACTORS

B.3.5 Impact due to Cs 137 in the Septage

PROGRAM ATMD000
YANKEE ATOMIC ELECTRIC COMPANY
REV. 1985 REV. 7

REGULATORY GUIDE 1.109, APPENDIX C
MODELS FOR CALCULATING DOSE VIA
ADDITIONAL PATHWAYS FROM RADIOIODINES
AND OTHER RADIONUCLIDES DISCHARGED
TO THE ATMOSPHERE NOV. 1977 LIBRARY

SECTOR = FLD 0 (METERS)
DISTANCE = 4.63E-04 (SEC/M-3)
X/D = 4.63E-04 (SEC/M-3)
X/D DEPLETED = 4.63E-04 (SEC/M-3)
DELTA = 1.24E-04 (1/M-2)
VY - SEMARE CONTAM - SOLIDS 12 BY WT - 2
ACRES - SHIELD F = 0.012 - 104 HR OCCUP

THE FOLLOWING 1 NUCLIDES WERE USED IN THIS CALCULATION

NUCLIDE RELEASE
55 CS 137 9.37E-05

6 PATHWAYS CONSIDERED
GROUND PLANE YES
INHALATION YES
STORED VEGETABLES YES
LEAFY VEGETABLES YES
COW MILK YES
GOAT MILK NO
MEAT YES

VARIABLE	VEGETABLES	COW MILK	GOAT MILK	MEAT
	STORED	PASTURE	PASTURE	STORED
YV AGRICULTURAL PRODUCTIVITY (KG/M-2)	2.00	.70	.70	.70
F SOIL SURFACE DENSITY (KG/M-2)	240.00	240.00	240.00	240.00
T TRANSPORT TIME TO USER (HRS)	240.00	48.00	48.00	480.00
TR SOIL EXPOSURE TIME (HRS)	8766.00	8766.00	8766.00	8766.00
TE CROP EXPOSURE TIME TO PLUME (HRS)	.00	.00	.00	.00
TH HOLDUP AFTER HARVEST (HRS)	1440.00	.00	.00	.00
GF ANIMALS DAILY FEED (KG/DAY)		50.00	2160.00	50.00
FP FRACTION OF YEAR ON PASTURE		.50	6.00	.50
FS FRACTION PASTURE WHEN ON PASTURE		1.00	1.00	1.00
FG FRACTION OF STORED VEG GROWN IN GARDEN	1.00			
FL FRACTION OF LEAFY VEG GROWN IN GARDEN				
FI FRACTION ELEMENTAL IODINE = .500				
H ABSOLUTE HUMIDITY = 5.60 (GM/M-3)				
FC FRACTIONAL EQUILIBRIUM RATIO FOR C-14 = 1.000				

AGE	VEG (KG/YR)	MILK (LI/YR)	MEAT (KG/YR)	INHALATION (M-3/YR)	ROSE DELIVERED TO EACH ORGAN FROM ALL PATHWAYS COMBINED
ADULT	520.00	64.00	310.00	110.00	5.06E-03
TEEN	630.00	42.00	400.00	65.00	5.06E-03
CHILD	520.00	26.00	330.00	41.00	5.06E-03
INFANT	.00	.00	330.00	.00	5.06E-03

B.3.6 Impact due to All Nuclides in the Septage

PROGRAM: ATM0005
 YANKEE ATOMIC ELECTRIC COMPANY
 DEC. 1983 REV. 7

REGULATOR: GUIDE 1.105, APPENDIX C
 MODELS FOR CALCULATING DOSE VIA
 ADDITIONAL PATHWAYS FROM RADIOIODINES
 AND OTHER RADIONUCLIDES DISCHARGED
 TO THE ATMOSPHERE NOV. 1977 LIBRARY

SECTOR - FLP 0 (METERS)
 DISTANCE - 4.43E-04 (SEC/H-3)
 X/D DEPLETED - 4.03E-04 (SEC/H-3)
 DELTA - 1.22E-04 (1/H-2)
 VY - SEWAGE CONTAIN - SOLIDS 12 BY WT - 2
 ACRES - CHIEF F - 0.012 - 104 HA OCCUP

THE FOLLOWING 5 NUCLIDES WERE USED IN THIS CALCULATION

NUCLIDE	RELEASE CURIES
25 IIN 54	4.13E-04
27 CO 60	2.51E-04
30 ZN 65	4.50E-04
55 CS 134	1.75E-04
55 CS 137	9.37E-05

6 PATHWAYS CONSIDERED
 GROUND FLAME YES
 INHALATION YES
 STORED VEGETABLES YES
 LEAFY VEGETABLES YES
 COW MILK YES
 GOAT MILK NO
 MEAT YES

VARIABLE	VEGETABLES STORED	LEAFY	PASTURE	STORED	GOAT MILK PASTURE	STORED	MEAT PASTURE	STORED
YV AGRICULTURAL PRODUCTIVITY (KG/M-2)	2.00	2.00	.70	2.00	.70	2.00	.70	2.00
P SOIL SURFACE DENSITY (KG/M-2)	240.00	240.00	240.00	240.00	240.00	240.00	240.00	240.00
T TRANSPORT TIME TO USER (HRS)	8766.00	8766.00	48.00	48.00	48.00	48.00	48.00	48.00
TP SOIL EXPOSURE TIME (HRS)	.00	.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00
TE CROP EXPOSURE TIME TO FLUME (HRS)	1440.00	24.00	.00	.00	.00	.00	.00	.00
TM HOLDUP AFTER HARVEST (HRS)	.00	.00	.00	2160.00	.00	2160.00	.00	2160.00
DF ANIMALS DAILY FEED (KG/DAY)	.00	.00	.00	50.00	.00	50.00	.00	50.00
FF FRACTION OF YEAR ON PASTURE	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
FS FRACTION OF STORED VEG GROWN IN GARDEN	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
FG FRACTION OF STORED VEG GROWN IN GARDEN	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
FL FRACTION OF LEAFY VEG GROWN IN GARDEN	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
FI FRACTION ELEMENTAL IODINE = .500								
H ABSOLUTE HUMIDITY = 5.60 (GM/M-3)								
FC FRACTIONAL EQUILIBRIUM RATIO FOR I-131 = 1.000								

AGE	VEG (KG/YR)	LEAFY VEG (KG/YR)	MILK (L/YR)	MEAT (KG/YR)	INHALATION (H-3/YR)	USAGE FACTORS
ADULT	520.00	64.00	310.00	110.00	8000.00	
TEEN	630.00	42.00	400.00	65.00	8000.00	
CHILD	520.00	26.00	330.00	41.00	3700.00	
INFANT	.00	.00	330.00	.00	1400.00	

DOSE DELIVERED TO EACH ORGAN FROM ALL PATHWAYS COMBINED	WHOLE BODY	SKIN	THYROID	GI-LLI	LUNG (HREM)	KIDNEY	LIVER	BONE
ADULT	9.45E-02	6.78E-02	5.76E-02	9.46E-02	9.45E-02	7.92E-02	1.13E-01	7.20E-02
TEEN	9.45E-02	6.78E-02	5.76E-02	9.45E-02	9.45E-02	9.08E-02	1.13E-01	7.20E-02
CHILD	9.94E-02	6.78E-02	5.76E-02	8.22E-02	6.73E-02	1.10E-01	1.13E-01	1.86E-01
INFANT	7.19E-02	6.78E-02	5.76E-02	7.59E-02	8.08E-02	8.36E-02	1.13E-01	1.14E-01

ADULT
DOSES RECEIVED FROM
VARIOUS PATHWAYS
(MREM)

FLD SECTOR AT A DISTANCE OF 0 METERS

PATHWAY	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN
GROUND PLANE								
54 MN	.00E+00	2.40E-06	5.96E-07	8.48E-05	4.69E-06	.00E+00	2.13E-04	2.50E-04
60 CO	.00E+00	4.24E-05	.00E+00	2.20E-02	1.05E-03	.00E+00	5.20E-02	6.12E-02
65 ZN	2.14E-06	6.81E-06	4.55E-06	5.70E-05	3.53E-06	.00E+00	1.46E-04	1.68E-04
134 CS	9.38E-06	2.13E-05	7.27E-06	2.45E-06	2.62E-07	.00E+00	2.27E-04	2.45E-04
137 CS	6.57E-04	8.53E-04	3.05E-04	1.03E-04	1.15E-05	.00E+00	5.06E-03	5.90E-03
TOTAL FOR PATHWAY	6.69E-04	9.26E-04	3.18E-04	2.22E-02	1.07E-03	.00E+00	5.76E-02	6.78E-02

PATHWAY	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN
INHALATION								
54 MN	.00E+00	2.40E-06	5.96E-07	8.48E-05	4.69E-06	.00E+00	2.13E-04	2.50E-04
60 CO	.00E+00	4.24E-05	.00E+00	2.20E-02	1.05E-03	.00E+00	5.20E-02	6.12E-02
65 ZN	2.14E-06	6.81E-06	4.55E-06	5.70E-05	3.53E-06	.00E+00	1.46E-04	1.68E-04
134 CS	9.38E-06	2.13E-05	7.27E-06	2.45E-06	2.62E-07	.00E+00	2.27E-04	2.45E-04
137 CS	6.57E-04	8.53E-04	3.05E-04	1.03E-04	1.15E-05	.00E+00	5.06E-03	5.90E-03
TOTAL FOR PATHWAY	6.69E-04	9.26E-04	3.18E-04	2.22E-02	1.07E-03	.00E+00	5.76E-02	6.78E-02

PATHWAY	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN
STORED VEGETABLES								
54 MN	.00E+00	8.180E-05	2.62E-05	.00E+00	2.73E-04	.00E+00	1.68E-05	
60 CO	.00E+00	1.24E-03	.00E+00	.00E+00	2.33E-02	.00E+00	2.74E-03	
65 ZN	1.23E-03	3.90E-03	2.61E-03	.00E+00	2.46E-03	.00E+00	1.76E-03	
134 CS	2.30E-04	5.46E-04	1.77E-04	5.87E-05	9.56E-06	.00E+00	4.47E-04	
137 CS	1.97E-02	2.69E-02	9.15E-03	3.04E-03	5.22E-04	.00E+00	1.76E-02	
TOTAL FOR PATHWAY	2.12E-02	3.27E-02	1.20E-02	3.10E-03	2.66E-02	.00E+00	2.26E-02	

PATHWAY	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN
LEAFY VEGETABLES								
54 MN	.00E+00	1.23E-05	3.67E-06	.00E+00	3.78E-05	.00E+00	2.36E-06	
60 CO	.00E+00	1.56E-04	.00E+00	.00E+00	2.93E-03	.00E+00	3.44E-04	
65 ZN	1.78E-04	5.67E-04	3.80E-04	.00E+00	3.57E-04	.00E+00	2.56E-04	
134 CS	2.98E-05	7.10E-05	2.30E-05	7.63E-06	1.24E-06	.00E+00	5.80E-05	
137 CS	2.43E-03	3.33E-03	1.13E-03	3.76E-04	6.44E-05	.00E+00	2.18E-03	
TOTAL FOR PATHWAY	2.64E-03	4.14E-03	1.54E-03	3.83E-04	3.59E-03	.00E+00	2.84E-03	

PATHWAY	ADULT ROSES RECEIVED FROM VARIOUS PATHWAYS (MREM)							FLD SECTOR AT A DISTANCE OF 0 MYLERS				
	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN				
COW MILK												
54 MN	.00E+00	6.78E-07	2.02E-07	.00E+00	2.08E-06	.00E+00	1.29E-07					
60 CO	.00E+00	3.72E-05	.00E+00	.00E+00	6.58E-04	.00E+00	8.20E-05					
65 ZN	1.49E-03	4.74E-03	3.17E-03	.00E+00	2.99E-03	.00E+00	2.14E-03					
134 CS	8.31E-05	1.98E-04	8.40E-05	2.13E-05	3.44E-04	.00E+00	1.62E-04					
137 CS	7.00E-03	9.65E-03	3.27E-03	1.09E-03	1.67E-04	.00E+00	6.32E-03					
TOTAL FOR PATHWAY	8.63E-03	1.46E-02	6.51E-03	1.11E-03	3.88E-03	.00E+00	8.70E-03					
MEAT												
54 MN	.00E+00	7.40E-07	2.20E-07	.00E+00	2.27E-06	.00E+00	1.41E-07					
60 CO	.00E+00	1.70E-04	.00E+00	.00E+00	3.00E-03	.00E+00	3.74E-04					
65 ZN	3.86E-04	1.23E-03	8.22E-04	.00E+00	7.70E-04	.00E+00	5.56E-04					
134 CS	9.67E-04	2.30E-05	7.45E-06	2.47E-06	4.03E-07	.00E+00	1.88E-05					
137 CS	8.33E-04	1.14E-03	3.87E-04	1.29E-04	2.21E-05	.00E+00	7.46E-04					
TOTAL FOR PATHWAY	1.23E-03	2.56E-03	1.22E-03	1.31E-04	4.00E-03	.00E+00	1.70E-03					
TOTAL ALL PATHS	3.43E-02	5.50E-02	2.15E-02	2.69E-02	3.89E-02	.00E+00	9.42E-02					6.78E-02
TOTAL ALL PATHS INCLUDING WHOLE BODY ROSE FROM GROUND PLANE EXPOSURE	9.20E-02	1.13E-01	7.92E-02	6.46E-02	9.66E-02	5.76E-02	9.42E-02					

PATHWAY	TEEN DOSES RECEIVED FROM VARIOUS PATHWAYS (MREM)							FLD SECTOR AT A DISTANCE OF 0 METERS				
	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN				
GROUND PLANE												
54 MN												
60 CO												
65 ZN												
134 CS												
137 CS												
TOTAL FOR PATHWAY												
INHALATION												
54 MN	.00E+00	3.10E-06	7.71E-07	1.20E-04	4.05E-06	.00E+00	5.09E-07	2.13E-04	2.50E-04			
60 CO	.00E+00	5.57E-05	.00E+00	3.21E-02	9.54E-04	.00E+00	7.31E-05	5.20E-02	6.12E-02			
65 ZN	2.55E-06	8.82E-06	5.70E-06	8.19E-05	3.08E-06	.00E+00	4.12E-06	1.46E-04	1.68E-04			
134 CS	1.26E-05	2.84E-05	9.44E-06	3.68E-06	2.45E-07	.00E+00	1.38E-05	2.27E-04	2.65E-04			
137 CS	9.21E-04	1.16E-03	4.18E-04	1.66E-04	1.16E-05	.00E+00	4.27E-04	5.06E-03	5.90E-03			
TOTAL FOR PATHWAY	9.36E-04	1.26E-03	4.33E-04	3.25E-02	9.73E-04	.00E+00	5.19E-04	5.76E-02	6.78E-02			
STORED VEGETABLES												
54 MN	.00E+00	1.58E-04	4.11E-05	.00E+00	2.82E-04	.00E+00	2.73E-05	2.73E-05	2.50E-04			
60 CO	.00E+00	1.97E-03	.00E+00	.00E+00	2.57E-02	.00E+00	4.45E-03	4.45E-03	4.45E-03			
65 ZN	1.77E-03	6.13E-03	3.93E-03	.00E+00	2.60E-03	.00E+00	2.84E-03	2.84E-03	2.84E-03			
134 CS	3.74E-04	8.81E-04	2.80E-04	1.07E-04	1.10E-05	.00E+00	4.09E-04	4.09E-04	4.09E-04			
137 CS	3.55E-02	4.46E-02	1.52E-02	5.90E-03	6.15E-04	.00E+00	1.55E-02	1.55E-02	1.55E-02			
TOTAL FOR PATHWAY	3.57E-02	5.37E-02	1.94E-02	6.01E-03	2.92E-02	.00E+00	2.33E-02	2.33E-02	2.33E-02			
LEAFY VEGETABLES												
54 MN	.00E+00	1.05E-05	3.12E-06	.00E+00	2.15E-05	.00E+00	2.07E-06	2.07E-06	2.07E-06			
60 CO	.00E+00	1.34E-04	.00E+00	.00E+00	1.75E-03	.00E+00	3.03E-04	3.03E-04	3.03E-04			
65 ZN	1.39E-04	4.84E-04	3.10E-04	.00E+00	2.05E-04	.00E+00	2.26E-04	2.26E-04	2.26E-04			
134 CS	2.63E-05	6.20E-05	1.97E-05	7.57E-06	7.71E-07	.00E+00	2.98E-05	2.98E-05	2.98E-05			
137 CS	2.24E-03	2.99E-03	1.05E-03	3.95E-04	4.25E-05	.00E+00	1.04E-03	1.04E-03	1.04E-03			
TOTAL FOR PATHWAY	2.41E-03	3.66E-03	1.35E-03	4.05E-04	2.52E-03	.00E+00	1.66E-03	1.66E-03	1.66E-03			

PATHWAY	TEEN DOSES RECEIVED FROM VARIOUS PATHWAYS (MKEM)							FLD SECTOR AT A DISTANCE OF 0 METERS				
	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN				
COW MILK												
54 MN	.00E+00	1.13E-06	3.37E-07	.00E+00	2.37E-06	.00E+00	2.24E-07					
60 CD	.00E+00	6.30E-05	.00E+00	.00E+00	8.20E-04	.00E+00	1.42E-04					
65 ZN	2.29E-03	7.95E-03	5.09E-03	.00E+00	3.36E-03	.00E+00	3.71E-03					
134 CS	1.44E-04	3.40E-04	1.08E-04	4.15E-05	4.23E-04	.00E+00	1.58E-04					
137 CS	1.28E-02	1.70E-02	5.79E-03	2.25E-03	2.42E-04	.00E+00	5.93E-03					
TOTAL FOR PATHWAY	1.52E-02	2.54E-02	1.10E-02	2.25E-03	4.43E-03	.00E+00	9.23E-03					
MEAT												
54 MN	.00E+00	5.65E-07	1.68E-07	.00E+00	1.18E-06	.00E+00	1.12E-07					
60 CD	.00E+00	1.35E-04	.00E+00	.00E+00	1.72E-03	.00E+00	2.99E-04					
65 ZN	2.72E-04	9.44E-04	6.04E-04	.00E+00	4.09E-04	.00E+00	4.40E-04					
134 CS	7.69E-06	1.81E-05	5.75E-06	2.20E-06	2.25E-07	.00E+00	8.40E-06					
137 CS	6.95E-04	9.20E-04	3.13E-04	1.22E-04	1.31E-05	.00E+00	3.21E-04					
TOTAL FOR PATHWAY	9.71E-04	2.01E-03	9.23E-04	1.24E-04	2.14E-03	.00E+00	1.07E-03					
TOTAL ALL PATHS	5.52E-02	8.61E-02	3.31E-02	4.13E-02	3.69E-02	.00E+00	9.41E-02					6.78E-02
TOTAL ALL PATHS INCLUDING WHOLE BODY DOSE FROM GROUND PLANE EXPOSURE	1.13E-01	1.44E-01	9.08E-02	9.90E-02	9.65E-02	5.76E-02	9.41E-02					

PATHWAY	CHILD DOSES RECEIVED FROM VARIOUS PATHWAYS (MREM)							FLD SECTOR AT A DISTANCE OF 0 METERS			
	BONE	LIVER	KIDNEY	LUNG	BI-LLI	THYROID	WHOLE BODY	SKIN			
GROUND FLAME											
54 MN	.00E+00	2.60E-06	6.08E-07	9.55E-05	1.39E-06	.00E+00	2.13E-04	2.50E-04			
60 CG	.00E+00	4.84E-05	.00E+00	2.60E-02	3.54E-04	.00E+00	5.20E-02	6.12E-02			
65 ZN	2.81E-06	7.47E-06	4.71E-06	6.57E-05	1.08E-06	.00E+00	1.46E-04	1.68E-04			
134 CS	1.64E-05	2.55E-02	8.31E-04	3.04E-06	9.68E-08	.00E+00	2.27E-04	2.65E-04			
137 CS	1.55E-03	1.13E-03	3.88E-04	1.43E-04	4.97E-06	.00E+00	5.06E-03	5.90E-03			
TOTAL FOR PATHWAY	1.26E-03	1.22E-03	4.01E-04	2.65E-02	3.62E-04	.00E+00	5.76E-02	6.78E-02			
INHALATION											
54 MN	.00E+00	2.06E-04	5.78E-05	.00E+00	1.73E-04	.00E+00	5.49E-05	2.50E-04			
60 CG	.00E+00	3.07E-03	.00E+00	.00E+00	1.70E-02	.00E+00	9.02E-03	6.12E-02			
65 ZN	3.47E-03	9.24E-03	5.85E-03	.00E+00	1.65E-03	.00E+00	5.75E-03	5.90E-03			
134 CS	8.63E-04	1.42E-03	4.38E-04	1.58E-04	7.64E-06	.00E+00	2.99E-04	1.14E-02			
137 CS	8.08E-02	7.74E-02	2.52E-02	9.07E-03	4.84E-04	.00E+00	1.14E-02	1.14E-02			
TOTAL FOR PATHWAY	8.52E-02	9.13E-02	3.15E-02	9.23E-03	1.93E-02	.00E+00	2.66E-02	2.66E-02			
STORED VEGETABLES											
54 MN	.00E+00	2.06E-04	5.78E-05	.00E+00	1.73E-04	.00E+00	5.49E-05	2.50E-04			
60 CG	.00E+00	3.07E-03	.00E+00	.00E+00	1.70E-02	.00E+00	9.02E-03	6.12E-02			
65 ZN	3.47E-03	9.24E-03	5.85E-03	.00E+00	1.65E-03	.00E+00	5.75E-03	5.90E-03			
134 CS	8.63E-04	1.42E-03	4.38E-04	1.58E-04	7.64E-06	.00E+00	2.99E-04	1.14E-02			
137 CS	8.08E-02	7.74E-02	2.52E-02	9.07E-03	4.84E-04	.00E+00	1.14E-02	1.14E-02			
TOTAL FOR PATHWAY	8.52E-02	9.13E-02	3.15E-02	9.23E-03	1.93E-02	.00E+00	2.66E-02	2.66E-02			
LEAFY VEGETABLES											
54 MN	.00E+00	1.17E-05	3.29E-06	.00E+00	9.84E-06	.00E+00	3.13E-06	3.13E-06			
60 CG	.00E+00	1.57E-04	.00E+00	.00E+00	8.68E-04	.00E+00	4.62E-04	4.62E-04			
65 ZN	2.05E-04	5.46E-04	3.44E-04	.00E+00	9.60E-05	.00E+00	3.40E-04	3.40E-04			
134 CS	4.56E-05	7.48E-05	2.32E-05	8.32E-06	4.03E-07	.00E+00	1.58E-05	1.58E-05			
137 CS	4.04E-03	3.88E-03	1.27E-03	4.55E-04	2.43E-05	.00E+00	5.75E-04	5.75E-04			
TOTAL FOR PATHWAY	4.31E-03	4.67E-03	1.64E-03	4.64E-04	9.99E-04	.00E+00	1.39E-03	1.39E-03			

PATHWAY	CHILD DOSES RECEIVED FROM VARIOUS PATHWAYS (MREM)							FLD SECTOR AT A DISTANCE OF 0 METERS				
	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN				
COW MILK												
54 MN	.00E+00	1.69E-06	4.74E-07	.00E+00	1.42E-06	.00E+00	4.50E-07					
60 CO	.00E+00	9.78E-02	.00E+00	.00E+00	5.42E-04	.00E+00	2.89E-04					
65 ZN	4.49E-03	1.20E-02	7.58E-03	.00E+00	2.10E-03	.00E+00	7.44E-03					
134 CS	3.33E-04	5.47E-04	1.65E-04	6.08E-05	2.95E-06	.00E+00	1.15E-04					
137 CS	3.08E-02	2.95E-02	9.61E-03	3.46E-03	1.85E-04	.00E+00	4.35E-03					
TOTAL FOR PATHWAY	3.56E-02	4.21E-02	1.73E-02	3.52E-03	2.63E-03	.00E+00	1.22E-02					
MEAT												
54 MN	.00E+00	6.46E-07	1.81E-07	.00E+00	5.42E-07	.00E+00	1.22E-07					
60 CO	.00E+00	1.57E-04	.00E+00	.00E+00	8.70E-04	.00E+00	4.83E-04					
65 ZN	4.08E-04	1.09E-03	6.84E-04	.00E+00	1.91E-04	.00E+00	6.76E-04					
134 CS	1.36E-05	2.23E-05	6.90E-06	2.48E-06	1.20E-07	.00E+00	4.20E-06					
137 CS	1.27E-03	1.22E-03	3.97E-04	1.43E-04	7.64E-06	.00E+00	1.80E-04					
TOTAL FOR PATHWAY	1.70E-03	2.49E-03	1.09E-03	1.42E-04	1.07E-03	.00E+00	1.32E-03					
TOTAL ALL PATHS	1.28E-01	1.42E-01	5.20E-02	3.97E-02	2.45E-02	.00E+00	9.94E-02					6.78E-02
TOTAL ALL PATHS INCLUDING WHOLE BODY DOSE FROM GROUND PLANE EXPOSURE	1.86E-01	1.99E-01	1.10E-01	9.73E-02	9.22E-02	5.76E-02	9.94E-02					

INFANT
DOSES RECEIVED FROM
VARIOUS PATHWAYS
(MREM)

FLD SECTOR AT A DISTANCE OF 0 METERS

PATHWAY	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN
GROUND PLANE								
54 MN	.00E+00	1.54E-04	3.02E-07	6.06E-03	4.28E-07	.00E+00	2.13E-04	2.50E-04
60 CO	.00E+00	2.95E-05	.00E+00	1.66E-02	1.18E-04	.00E+00	5.20E-02	6.12E-02
65 ZM	1.28E-06	4.13E-06	2.14E-06	4.27E-03	3.39E-06	.00E+00	1.46E-04	1.68E-04
134 CS	9.86E-06	1.77E-05	4.79E-06	2.00E-06	1.87E-08	.00E+00	2.27E-04	2.65E-04
137 CS	7.54E-04	8.40E-04	2.37E-04	9.79E-05	1.83E-06	.00E+00	5.06E-03	5.90E-03
TOTAL FOR PATHWAY	7.65E-04	8.93E-04	2.44E-04	1.68E-02	1.23E-03	.00E+00	5.76E-02	6.78E-02

PATHWAY	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN
INHALATION								
54 MN	.00E+00	1.54E-04	3.02E-07	6.06E-03	4.28E-07	.00E+00	3.02E-07	
60 CO	.00E+00	2.95E-05	.00E+00	1.66E-02	1.18E-04	.00E+00	4.34E-05	
65 ZM	1.28E-06	4.13E-06	2.14E-06	4.27E-03	3.39E-06	.00E+00	2.05E-06	
134 CS	9.86E-06	1.77E-05	4.79E-06	2.00E-06	1.87E-08	.00E+00	1.87E-06	
137 CS	7.54E-04	8.40E-04	2.37E-04	9.79E-05	1.83E-06	.00E+00	6.25E-05	
TOTAL FOR PATHWAY	7.65E-04	8.93E-04	2.44E-04	1.68E-02	1.23E-03	.00E+00	1.10E-04	

PATHWAY	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN
STORED VEGETABLES								
54 MN	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
60 CO	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
65 ZM	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
134 CS	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
137 CS	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
TOTAL FOR PATHWAY	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00

PATHWAY	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN
LEAFY VEGETABLES								
54 MN	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
60 CO	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
65 ZM	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
134 CS	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
137 CS	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
TOTAL FOR PATHWAY	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00

PATHWAY	INFANT DOSES RECEIVED FROM VARIOUS PATHWAYS (MREM)							FLD SECTOR AT A DISTANCE OF 0 METERS		
	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN		
COW MILK										
54 MN	.00E+00	3.14E-06	6.97E-07	.00E+00	1.14E-06	.00E+00	7.13E-07			
60 CD	.00E+00	2.00E-04	.00E+00	.00E+00	4.75E-04	.00E+00	4.75E-04			
45 ZN	6.03E-03	2.07E-02	1.00E-02	.00E+00	1.75E-02	.00E+00	9.54E-03			
134 CS	5.37E-04	1.00E-03	2.58E-04	1.04E-04	2.75E-06	.00E+00	1.01E-04			
137 CS	4.92E-02	5.74E-02	1.54E-02	6.23E-03	1.89E-04	.00E+00	4.08E-03			
TOTAL FOR PATHWAY	5.57E-02	7.94E-02	2.57E-02	6.34E-03	1.81E-02	.00E+00	1.47E-02			
MEAT										
54 MN	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00			
60 CD	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00			
45 ZN	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00			
134 CS	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00			
137 CS	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00			
TOTAL FOR PATHWAY	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00			
TOTAL ALL PATHS	5.65E-02	8.03E-02	2.60E-02	2.32E-02	1.83E-02	.00E+00	7.19E-02			6.78E-02
TOTAL ALL PATHS INCLUDING WHOLE BODY DOSE FROM GROUND PLANE EXPOSURE	1.14E-01	1.38E-01	8.34E-02	8.08E-02	7.59E-02	5.74E-02	7.19E-02			

B.4 ATMODOS - Radiological Impact After Termination of VY

Control of the Disposal Site (All Nuclides)

Presented below is a partial listing of the ATMADOS Tape 5 input used in this portion of the calculation. It corresponds to the output which follows. With the exception of the nuclide data library, a large portion of which was deleted due to space limitations, the listing is complete.

1 1VYPI VY STANDARD PROGRAM INFORMATION FILE
VY - SEWAGE CONTAM - SOLIDS 1% BY WT - 2 ACRES - SHIELD F = 0.242 - CONT. OCCUP
1. .200 44.56FRESH .500YESNO NO 0..242 1.NO NO NO 0.1.00

--EOR--

--EOF--

0 2VYGASRLGAS STK VY 87 1 1 0 87063023:

MN54 4.132E-6 25 MN 54

CO60 2.511E-4 27 CO 60

ZN65 4.502E-6 30 ZN 65

CS134 1.715E-6 55 CS 134

CS137 9.366E-5 55 CS 137

--EOR--

--EOF--

1 5QUERY PATHWAYS: SHORE(1,2,3),BND,ROAD,RES1,RAD,MEAT,GOAT,COW

SHORE1 YESYES

SHORE2 YESYES

SHORE3 YESYES

BOUND YESYES

ROAD YESYES

RFS1 YESYESYESYES

RADIUS YESYES

MEAT YESYESYESYES YESYESYES

GOAT YESYESYESYES YES YESYES

COW YESYESYESYESYES YESYES

--EOR--

--EOF--

1 6VYUFMAXVY MAXIMUM INDIVIDUAL USAGE FACTORS FOR STANDARD PROBLEMS

520.00	64.00	310.00	110.00	21.00	.00	.00	12.00	.00	8000.00
--------	-------	--------	--------	-------	-----	-----	-------	-----	---------

630.00	42.00	400.00	65.00	16.00	.00	.00	67.00	.00	8000.00
--------	-------	--------	-------	-------	-----	-----	-------	-----	---------

520.00	26.00	330.00	41.00	6.90	.00	.00	14.00	.00	3700.00
--------	-------	--------	-------	------	-----	-----	-------	-----	---------

.00	.00	330.00	.00	.00	.00	.00	.00	.00	1400.00
-----	-----	--------	-----	-----	-----	-----	-----	-----	---------

--EOR--

--EOF--

1 7VYGS D VY MAX INDIVIDUAL GAS SITE DATA FILE FOR STANDARD

						PROBLEMS	
2.00	2.00	.70	2.00	.70	2.00	.70	2.00
240.00	240.00	240.00	240.00	240.00	240.00	240.00	240.00
.00	.00	48.00	48.00	48.00	48.00	480.00	480.00
8766.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00
0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
1440.00	24.00	.00	2160.00	.00	2160.00	.00	2160.00
.00	.00	50.00	50.00	6.00	6.00	50.00	50.00
.00	.00	.50	.00	.50	.00	.50	.00
.00	.00	1.00	.00	1.00	.00	1.00	.00
1.0	1.00	.00	.00	.00	.00	.00	.00
5.60	.00	.00	.00	.00	.00	.00	.00

--EOR--

--EOF--

1 81STPNBLNUCLIDE LIBRARY FOR ALL DOSE PROGRAMS

89

1	3	1.78E-090.00E+000.00E+00						SOLUBLE	
H	9.0E-01	9.0E-01	9.0E-01	9.0E-01	9.3E-01	9.3E-01	4.8E-00	1.0E-02	1.2E-02
0.	1.05E-071.05E-071.05E-071.05E-071.05E-071.05E-07								
0.	1.58E-071.58E-071.58E-071.58E-071.58E-071.58E-07								
0.	1.06E-071.06E-071.06E-071.06E-071.06E-071.06E-07								
0.	1.59E-071.59E-071.59E-071.59E-071.59E-071.59E-07								
0.	2.03E-072.03E-072.03E-072.03E-072.03E-072.03E-07								
0.	3.04E-073.04E-073.04E-073.04E-073.04E-073.04E-07								
0.	3.08E-073.08E-073.08E-073.08E-073.08E-073.08E-07								
0.	4.62E-074.62E-074.62E-074.62E-074.62E-074.62E-07								
6	14	3.83E-12						SOLUBLE	
C	4.6E+03	9.1E+03	4.6E+03	1.8E+03	1.4E+03	1.8E+03	5.5E+00	1.2E-02	3.1E-02
2.84E-065.68E-075.68E-075.68E-075.68E-075.68E-075.68E-07									
2.27E-064.26E-074.26E-074.26E-074.26E-074.26E-074.26E-07									
4.06E-068.12E-078.12E-078.12E-078.12E-078.12E-078.12E-07									
3.25E-066.09E-076.09E-076.09E-076.09E-076.09E-076.09E-07									
1.21E-052.42E-062.42E-062.42E-062.42E-062.42E-062.42E-06									
9.70E-061.82E-061.82E-061.82E-061.82E-061.82E-061.82E-06									
2.37E-055.06E-065.06E-065.06E-065.06E-065.06E-065.06E-06									
1.89E-053.79E-063.79E-063.79E-063.79E-063.79E-063.79E-06									
11	24	1.28E-052.50E-08	2.90E-08						SOLUBLE
.									
.									
.									
.									
ETC (FOR A TOTAL CF 89 NUCLIDES)									

--EOR--

--EOF--

1 3VYXQF VY X/QFILE - SPECIAL VALUES - SEWAGE CONTAM. PROBLEM - 2 ACRE PLOTS
 FLD 0 COW MEAT 3.896E-02 3.896E-02 1.236E-04 3.896E-02

--EOR--

--EOF--

REGULATORY GUIDE 1.109, APPENDIX C
MODELS FOR CALCULATING DOSE VIA
ADDITIONAL PATHWAYS FROM RADIOIODINES
AND OTHER RADIONUCLIDES DISCHARGED
TO THE ATMOSPHERE NOV. 1977 LIBRARY

SECTOR - FLD 0 (HECTARS)
DISTANCE - 3.90E-02 (SEC/M-3)
X/D DEPLETED - 3.90E-02 (SEC/M-3)
DELTA - 1.24E-04 (1/M-2)

VV - SEWAGE CONTAM. - SOLIDS 12 BY WT -
Z ACKES - SHIELD F - 0.242 - CONT. OCCUP

THE FOLLOWING 5 NUCLIDES WERE USED IN THIS CALCULATION

NUCLIDE	RELEASE CURIES
25 MN 54	4.13E-06
27 CO 60	2.51E-04
30 ZN 65	4.50E-06
55 CS 134	1.72E-06
55 CS 137	9.57E-05

6 PATHWAYS CONSIDERED

GROUND FLARE YES
INHALATION YES
STORED VEGETABLES YES
LEAFY VEGETABLES YES
COW MILK YES
GOAT MILK NO
MEAT YES

VARIABLE	VEGETABLES STORED	LEAFY	CGW MILK PASTURE	STORED	FASUTURE	GOAT MILK STORED	FASUTURE	MEAT STORED
YV AGRICULTURAL PRODUCTIVITY (KG/H-2)	2.00	2.00	.70	2.00	.70	2.00	.70	2.00
F SOIL SURFACE DENSITY (KG/H-2)	240.00	240.00	240.00	240.00	240.00	240.00	240.00	240.00
T TRANSFOT TIME TO USER (HRS)	48.00	48.00	48.00	48.00	48.00	48.00	48.00	48.00
TR SOIL EXPOSURE TIME TO PLUME (HRS)	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00
TE CROP EXPOSURE TIME TO PLUME (HRS)	.00	.00	.00	.00	.00	.00	.00	.00
TH HOLDUP AFTER HARVEST (HRS)	1440.00	24.00	.00	2160.00	.00	2160.00	.00	2160.00
9F ANIMALS DAILY FEED (KG/DAY)			50.00	50.00	50.00	50.00	50.00	50.00
FF FRACTION OF YEAR ON PASTURE			1.00	.50	.50	1.00	.50	1.00
FS FRACTION PASTURE WHEN ON PASTURE	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
FG FRACTION OF STORED VEG GROWN IN GARDEN								
FL FRACTION OF LEAFY VEG GROWN IN GARDEN								
FI FRACTION ELEMENTAL IODINE = .500								
H ABSOLUTE HUMIDITY = 5.60 (GR/H-3)								
FC FRACTIONAL EQUILIBRIUM RATIO FOR C-14 = 1.000								

AGE	VEG (KG/YR)	LEAFY VEG (KG/YR)	MILK (LI/YR)	MEAT (KG/YR)	INHALATION (M-3/YR)	VEG (KG/YR)	LIVER	KIDNEY	LUNGS (MEEK)	GI-LLI	THYROID	WHOLE BODY	SKIN
ADULT	520.00	64.00	3.0.00	110.00	8000.00	1.25E+00	1.21E+00	1.21E+00	3.04E+00	1.29E+00	1.16E+00	1.25E+00	1.37E+00
TEEN	430.00	42.00	400.00	45.00	8000.00	1.70E+00	1.23E+00	1.23E+00	3.71E+00	1.25E+00	1.16E+00	1.24E+00	1.37E+00
CHILD	520.00	26.00	330.00	41.00	3700.00	1.40E+00	1.23E+00	1.23E+00	3.39E+00	1.25E+00	1.16E+00	1.23E+00	1.37E+00
INFANT	.00	.00	530.00	.00	1400.00	1.25E+00	1.21E+00	1.21E+00	2.08E+00	1.19E+00	1.16E+00	1.15E+00	1.37E+00

USAGE FACTORS

NOSE DELIVERED TO EACH ORGAN FROM ALL PATHWAYS COMBINED

ADULT
 ROSES RECEIVED FROM
 VARIOUS PATHWAYS
 (MREM)

FLD SECTOR AT A DISTANCE OF 0 METERS

PATHWAY	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN
GROUND PLANE								
54 MN	.00E+00	2.02E-04	5.02E-05	7.14E-03	3.92E-04	.00E+00	4.30E-03	5.02E-03
60 CD	.00E+00	3.57E-03	.00E+00	1.63E+00	8.83E-02	.00E+00	1.02E+00	1.23E+00
65 ZN	1.80E-04	5.74E-04	3.83E-04	4.80E-03	2.97E-04	.00E+00	2.74E-03	3.38E-03
134 CS	7.90E-04	1.80E-03	6.08E-04	2.07E-04	2.70E-05	.00E+00	4.58E-03	5.34E-03
137 CS	5.53E-02	7.18E-02	2.57E-02	8.70E-03	9.72E-04	.00E+00	1.02E-01	1.19E-01
TOTAL FOR PATHWAY	5.63E-02	7.80E-02	2.68E-02	1.87E+00	9.00E-02	.00E+00	1.16E+00	1.37E+00

INHALATION

54 MN	.00E+00	8.80E-05	2.62E-05	.00E+00	2.70E-04	.00E+00	1.68E-05	
60 CD	.00E+00	1.24E-03	.00E+00	.00E+00	2.33E-02	.00E+00	2.74E-03	
65 ZN	1.23E-03	3.90E-03	2.61E-03	.00E+00	2.46E-03	.00E+00	1.76E-03	
134 CS	2.30E-04	5.46E-04	1.77E-04	5.87E-05	9.58E-06	.00E+00	4.87E-04	
137 CS	1.97E-02	2.69E-02	9.15E-03	3.04E-03	5.22E-04	.00E+00	1.76E-02	
TOTAL FOR PATHWAY	2.12E-02	3.27E-02	1.20E-02	3.10E-03	2.66E-02	.00E+00	2.58E-02	

STORED VEGETABLES

54 MN	.00E+00	8.80E-05	2.62E-05	.00E+00	2.70E-04	.00E+00	1.68E-05	
60 CD	.00E+00	1.24E-03	.00E+00	.00E+00	2.33E-02	.00E+00	2.74E-03	
65 ZN	1.23E-03	3.90E-03	2.61E-03	.00E+00	2.46E-03	.00E+00	1.76E-03	
134 CS	2.30E-04	5.46E-04	1.77E-04	5.87E-05	9.58E-06	.00E+00	4.87E-04	
137 CS	1.97E-02	2.69E-02	9.15E-03	3.04E-03	5.22E-04	.00E+00	1.76E-02	
TOTAL FOR PATHWAY	2.12E-02	3.27E-02	1.20E-02	3.10E-03	2.66E-02	.00E+00	2.58E-02	

LEAFY VEGETABLES

54 MN	.00E+00	1.23E-05	5.67E-06	.00E+00	3.78E-05	.00E+00	2.56E-06	
60 CD	.00E+00	1.56E-04	.00E+00	.00E+00	2.93E-03	.00E+00	3.44E-04	
65 ZN	1.78E-04	5.67E-04	3.80E-04	.00E+00	3.57E-04	.00E+00	2.56E-04	
134 CS	2.98E-05	7.10E-05	2.30E-05	7.63E-06	1.24E-06	.00E+00	5.80E-05	
137 CS	2.43E-03	3.33E-03	1.13E-03	3.76E-04	6.44E-05	.00E+00	2.16E-03	
TOTAL FOR PATHWAY	2.64E-03	4.14E-03	1.54E-03	5.83E-04	5.39E-03	.00E+00	2.84E-03	

PATHWAY	ADULT DOSES RECEIVED FROM VARIOUS PATHWAYS (MREM)							FLD SECTOR AT A DISTANCE OF 0 METERS				
	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN				
COW MILK												
54 MN	.00E+00	6.78E-07	2.02E-07	.00E+00	2.08E-06	.00E+00	1.29E-07					
60 CD	.00E+00	3.72E-05	.00E+00	.00E+00	6.99E-04	.00E+00	8.50E-05					
65 ZN	1.49E-03	4.74E-03	3.17E-03	.00E+00	2.97E-03	.00E+00	2.14E-03					
134 CS	8.32E-03	1.98E-04	6.49E-05	2.13E-05	3.46E-06	.00E+00	1.62E-04					
137 CS	7.05E-03	9.65E-03	3.27E-03	1.09E-03	1.87E-04	.00E+00	6.72E-03					
TOTAL FOR PATHWAY	8.63E-03	1.46E-02	6.51E-03	1.11E-03	3.88E-03	.00E+00	8.70E-03					
MEAT												
54 MN	.00E+00	7.40E-07	2.20E-07	.00E+00	2.27E-06	.00E+00	1.41E-07					
60 CD	.00E+00	1.70E-04	.00E+00	.00E+00	3.20E-03	.00E+00	3.76E-04					
65 ZN	3.86E-04	1.23E-03	8.22E-04	.00E+00	7.72E-04	.00E+00	5.56E-04					
134 CS	9.67E-04	2.30E-05	7.45E-06	2.47E-06	4.03E-07	.00E+00	1.88E-05					
137 CS	8.33E-04	1.14E-03	3.87E-04	1.29E-04	2.21E-05	.00E+00	7.46E-04					
TOTAL FOR PATHWAY	1.23E-03	2.56E-03	1.22E-03	1.31E-04	4.00E-03	.00E+00	1.70E-03					
TOTAL ALL PATHS	9.00E-02	1.32E-01	4.60E-02	1.88E+00	1.28E-01	.00E+00	1.22E+00					1.37E+00
TOTAL ALL PATHS INCLUDING WHOLE BODY DOSE FROM GROUND PLANE EXPOSURE	1.25E+00	1.29E+00	1.21E+00	3.04E+00	1.29E+00	1.16E+00	1.22E+00					

TEEN
 DOS'S RECEIVED FROM
 VARIOUS PATHWAYS
 (MREM)

FLD SECTOR AT A DISTANCE OF 0 METERS

PATHWAY	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN
GROUND PLANE								
54 MN	.00E+00	2.61E-04	6.49E-05	1.01E-02	3.41E-04	.00E+00	4.30E-03	5.05E-03
60 CD	.00E+00	4.69E-03	.00E+00	2.70E+00	8.04E-02	.00E+00	1.05E+00	1.23E+00
65 ZN	2.14E-04	7.43E-04	4.80E-04	6.89E-03	2.09E-04	.90E+00	2.94E-03	3.88E-03
134 CS	1.86E-03	2.39E-03	7.95E-04	7.10E-04	2.07E-03	.00E+00	4.58E-03	5.34E-03
137 CS	7.75E-02	5.81E-02	3.55E-02	1.40E-02	9.61E-04	.00E+00	1.05E-01	1.19E-01
TOTAL FOR PATHWAY	7.88E-02	1.06E-01	3.65E-02	2.74E+00	8.20E-02	.00E+00	1.16E+00	1.37E+00

INHALATION

54 MN	.00E+00	1.38E-04	4.11E-05	.00E+00	2.82E-04	.00E+00	2.73E-05	
60 CD	.00E+00	1.97E-03	.00E+00	.00E+00	2.57E-02	.00E+00	4.43E-03	
65 ZN	1.77E-03	6.13E-03	3.93E-03	.00E+00	2.60E-03	.00E+00	2.86E-03	
134 CS	3.74E-04	8.81E-04	2.86E-04	1.07E-04	3.10E-03	.00E+00	4.09E-04	
137 CS	3.35E-02	4.46E-02	1.55E-02	5.99E-03	6.55E-04	.00E+00	1.55E-02	
TOTAL FOR PATHWAY	3.57E-02	5.37E-02	1.94E-02	6.01E-03	2.52E-02	.00E+00	2.33E-02	

STORED VEGETABLES

54 MN	.00E+00	1.05E-05	3.12E-06	.00E+00	2.15E-05	.00E+00	2.07E-06	
60 CD	.00E+00	1.34E-04	.00E+00	.00E+00	1.75E-03	.00E+00	3.03E-04	
65 ZN	1.39E-04	4.64E-04	3.10E-04	.00E+00	2.05E-04	.00E+00	2.66E-04	
134 CS	2.83E-03	6.20E-03	1.97E-03	7.52E-06	2.71E-07	.00E+00	2.88E-05	
137 CS	2.24E-03	2.99E-03	1.02E-03	3.95E-04	4.25E-05	.00E+00	1.04E-03	
TOTAL FOR PATHWAY	2.41E-03	3.68E-03	1.35E-03	4.02E-04	2.05E-03	.00E+00	1.60E-03	

LEAFY VEGETABLES

54 MN	.00E+00	1.05E-05	3.12E-06	.00E+00	2.15E-05	.00E+00	2.07E-06	
60 CD	.00E+00	1.34E-04	.00E+00	.00E+00	1.75E-03	.00E+00	3.03E-04	
65 ZN	1.39E-04	4.64E-04	3.10E-04	.00E+00	2.05E-04	.00E+00	2.66E-04	
134 CS	2.83E-03	6.20E-03	1.97E-03	7.52E-06	2.71E-07	.00E+00	2.88E-05	
137 CS	2.24E-03	2.99E-03	1.02E-03	3.95E-04	4.25E-05	.00E+00	1.04E-03	
TOTAL FOR PATHWAY	2.41E-03	3.68E-03	1.35E-03	4.02E-04	2.05E-03	.00E+00	1.60E-03	

TEEN
DOSES RECEIVED FROM
VARIOUS PATHWAYS
(MREM)

FLD SECTOR AT A DISTANCE OF 0 METERS

PATHWAY BONE LIVER KIDNEY LUNG GI-LLI THYROID WHOLE BODY SKIN

COW MILK
 54 MN .00E+00
 60 CO .00E+00
 65 ZN 2.59E-03
 134 CS 1.44E-04
 137 CS 1.28E-02
 TOTAL FOR PATHWAY 1.52E-02

MEAT
 54 MN .00E+00
 60 CO .00E+00
 65 ZN 2.72E-04
 134 CS 7.69E-06
 137 CS 6.92E-04
 TOTAL FOR PATHWAY 9.71E-04

TOTAL ALL PATHS INCLUDING WHOLE BODY DOSE FROM GROUND FLARE EXPOSURE

1.33E-01
 1.30E+00
 1.91E-01
 6.92E-02
 1.23E+00
 3.91E+00
 1.20E-01
 .00E+00
 1.24E+00
 1.37E+00

CHILD
DOSES RECEIVED FROM
VARIOUS PATHWAYS
(MREM)

FLD SECTOR AT A DISTANCE OF 0 METERS

PATHWAY	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN
GROUND PLANE								
54 MN	.00E+00	2.19E-04	5.12E-03	6.04E-03	1.17E-04	.00E+00	4.30E-03	5.05E-03
60 CD	.00E+00	4.07E-03	.00E+00	3.19E+00	7.98E-02	.00E+00	1.05E+00	1.23E+00
65 ZM	2.37E-04	8.50E-04	3.97E-04	5.53E-03	9.07E-05	.00E+00	2.94E-03	3.38E-03
134 CS	1.38E-03	2.15E-03	7.00E-04	2.56E-04	8.15E-06	.00E+00	4.58E-03	5.34E-03
137 CS	1.05E-01	9.54E-02	3.27E-02	1.20E-02	4.19E-04	.00E+00	1.02E-01	1.19E-01
TOTAL FOR FAT "UP"	1.06E-01	1.03E-01	3.58E-02	2.22E+00	3.05E-02	.00E+00	1.16E+00	1.37E+00
INHALATION								
54 MN	.00E+00	2.06E-04	5.78E-03	.00E+00	1.77E-04	.00E+00	4.85E-03	
60 CD	.00E+00	3.07E-03	.00E+00	.00E+00	1.77E-02	.00E+00	7.02E-03	
65 ZM	3.47E-03	9.24E-03	5.82E-03	.00E+00	1.64E-03	.00E+00	3.91E-04	
134 CS	8.43E-04	1.42E-03	4.39E-04	1.58E-04	7.67E-06	.00E+00	4.76E-04	
137 CS	8.08E-02	7.74E-02	2.52E-02	9.07E-03	4.84E-04	.00E+00	1.49E-02	
TOTAL FOR PATHWAY	8.52E-02	9.13E-02	3.15E-02	9.23E-03	1.93E-02	.00E+00	2.28E-02	
STORED VEGETABLES								
54 MN	.00E+00	2.06E-04	5.78E-03	.00E+00	1.77E-04	.00E+00	5.49E-03	
60 CD	.00E+00	3.07E-03	.00E+00	.00E+00	1.77E-02	.00E+00	9.02E-03	
65 ZM	3.47E-03	9.24E-03	5.82E-03	.00E+00	1.64E-03	.00E+00	5.75E-03	
134 CS	8.43E-04	1.42E-03	4.39E-04	1.58E-04	7.67E-06	.00E+00	2.99E-04	
137 CS	8.08E-02	7.74E-02	2.52E-02	9.07E-03	4.84E-04	.00E+00	1.14E-02	
TOTAL FOR PATHWAY	8.52E-02	9.13E-02	3.15E-02	9.23E-03	1.93E-02	.00E+00	2.66E-02	
LEAFY VEGETABLES								
54 MN	.00E+00	1.17E-02	3.29E-06	.00E+00	9.86E-06	.00E+00	3.13E-06	
60 CD	.00E+00	1.57E-04	.00E+00	.00E+00	8.88E-04	.00E+00	4.62E-04	
65 ZM	2.05E-04	5.46E-04	3.44E-04	.00E+00	9.60E-05	.00E+00	3.40E-04	
134 CS	4.56E-05	7.48E-05	2.52E-05	8.52E-06	4.03E-07	.00E+00	1.58E-05	
137 CS	4.06E-03	3.88E-03	1.27E-03	4.55E-04	2.43E-05	.00E+00	5.73E-04	
TOTAL FOR PATHWAY	4.31E-03	4.67E-03	1.64E-03	4.64E-04	9.99E-04	.00E+00	1.39E-03	

PATHWAY	CHILD DOSES RECEIVED FROM VARIOUS PATHWAYS (MREM)							FLD SECTOR AT A DISTANCE OF 0 METERS				
	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN				
COM MILK												
54 MN	.00E+00	1.69E-06	4.74E-07	.00E+00	1.42E-06	.00E+00	4.50E-07					
60 CD	.00E+00	9.78E-05	.00E+00	.00E+00	5.42E-04	.00E+00	2.89E-04					
65 ZN	4.89E-03	1.20E-02	7.54E-03	.00E+00	2.10E-03	.00E+00	7.44E-03					
134 CS	3.23E-04	5.47E-04	1.69E-04	6.08E-05	2.92E-06	.00E+00	1.12E-04					
137 CS	3.08E-02	2.92E-02	9.61E-03	3.44E-03	1.82E-04	.00E+00	4.32E-03					
TOTAL FOR PATHWAY	3.56E-02	4.21E-02	1.72E-02	3.32E-03	2.83E-03	.00E+00	1.22E-02					
MEAT												
54 MN	.00E+00	6.46E-07	1.81E-07	.00E+00	5.42E-07	.00E+00	1.72E-07					
60 CD	.00E+00	1.57E-04	.00E+00	.00E+00	8.70E-04	.00E+00	4.63E-04					
65 ZN	4.08E-04	1.09E-03	6.84E-04	.00E+00	1.91E-04	.00E+00	6.76E-04					
134 CS	1.36E-05	2.23E-05	6.90E-06	2.48E-06	1.20E-07	.00E+00	4.70E-06					
137 CS	1.27E-03	1.22E-03	3.97E-04	1.43E-04	7.64E-06	.00E+00	1.80E-04					
TOTAL FOR PATHWAY	1.70E-03	2.49E-03	1.09E-03	1.43E-04	1.07E-03	.00E+00	1.32E-03					
TOTAL ALL PATHS	2.33E-01	2.43E-01	8.54E-02	2.22E+00	5.46E-02	.00E+00	1.23E+00					1.37E+00
TOTAL ALL PATHS INCLUDING WHOLE BODY DOSE FROM GROUND PLANE EXPOSURE	1.40E+00	1.41E+00	1.22E+00	3.39E+00	1.22E+00	1.16E+00	1.23E+00					

INFANT
DOSES RECEIVED FROM
VARIOUS PATHWAYS
(MREM)

FLD SECTOR AT A DISTANCE OF 0 METERS

PATHWAY	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN
GROUND FLARE								
54 MN	.00E+00	1.29E-04	2.54E-05	5.10E-03	3.60E-02	.00E+00	4.30E-03	2.05E-03
60 CO	.00E+00	2.49E-03	.00E+00	1.40E+00	9.90E-03	.00E+00	1.05E+00	1.53E+00
65 ZN	1.07E-04	3.48E-04	1.81E-04	3.60E-03	2.86E-04	.00E+00	2.94E-03	3.38E-03
134 CS	8.39E-04	1.49E-03	4.03E-04	1.69E-04	2.83E-04	.00E+00	4.58E-03	5.34E-03
137 CS	6.35E-02	7.08E-02	1.99E-02	8.24E-03	1.54E-04	.00E+00	1.02E-01	1.19E-01
TOTAL FOR PATHWAY	6.44E-02	7.52E-02	2.05E-02	1.42E+00	1.04E-02	.00E+00	1.18E+00	1.37E+00
INHALATION								
54 MN	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
60 CO	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
65 ZN	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
134 CS	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
137 CS	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
TOTAL FOR PATHWAY	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
STORED VEGETABLES								
54 MN	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
60 CO	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
65 ZN	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
134 CS	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
137 CS	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
TOTAL FOR PATHWAY	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
LEAFY VEGETABLES								
54 MN	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
60 CO	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
65 ZN	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
134 CS	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
137 CS	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00
TOTAL FOR PATHWAY	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00

PATHWAY	INFANT DOSES RECEIVED FROM VARIOUS PATHWAYS (MREM)							FLD SECTOR AT A DISTANCE OF 0 METERS				
	BONE	LIVER	KIDNEY	LUNG	GI-LLI	THYROID	WHOLE BODY	SKIN				
COM MILK												
54 MN	.00E+00	3.14E-06	6.97E-07	.00E+00	1.16E-06	.00E+00	7.13E-07					
60 CD	.00E+00	2.00E-04	.00E+00	.00E+00	4.72E-04	.00E+00	4.72E-04					
65 ZN	6.03E-03	2.07E-02	1.00E-02	.00E+00	1.72E-02	.00E+00	9.54E-03					
134 CS	5.37E-04	1.00E-03	2.58E-04	1.06E-04	2.72E-06	.00E+00	1.01E-04					
137 CS	4.92E-02	5.76E-02	1.54E-02	6.22E-03	1.80E-04	.00E+00	4.08E-03					
TOTAL FOR PATHWAY	5.57E-02	7.94E-02	2.57E-02	6.36E-03	1.81E-02	.00E+00	1.42E-02					
MEAT												
54 MN	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00					
60 CD	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00					
65 ZN	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00					
134 CS	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00					
137 CS	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00					
TOTAL FOR PATHWAY	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00	.00E+00					
TOTAL ALL PATHS	1.20E-01	1.55E-01	4.63E-02	1.42E+00	2.85E-02	.00E+00	1.19E+00					1.37E+00
TOTAL ALL PATHS INCLUDING WHOLE BODY DOSE FROM GROUND PLANE EXPOSURE	1.28E+00	1.32E+00	1.21E+00	2.58E+00	1.19E+00	1.16E+00	1.19E+00					

B.5 ATMODOS - Unplowed-Land Dose Conversion Factors for
Radiological Impact Assessment

The Tape 5 inputs to ATMODOS for these cases are identical to the one shown in Attachment B.3, the only exception being the isotopic intensities in File 2. In the current computer runs, each isotope was assumed to have an annual release rate which would yield an accumulated intensity of 1 μCi at the end of one year (uniformly spread over 2 acres of unplowed land).

B.5.1 Impact due to 1 uCi of Mn 54

PROGRAM ATHODOS
YANKEE ATOMIC ELECTRIC COMPANY
DEC. 1983 REV. 7

REGULATORY GUIDE 1.109, APPENDIX C
MODELS FOR CALCULATING DOSE VIA
ADDITIONAL PATHWAYS FROM RADIOIODINES
AND OTHER RADIONUCLIDES DISCHARGED
TO THE ATMOSPHERE NOV. 1977 LIBRARY

FLD 0 (METERS)
DISTANCE * 4.63E-04 (SEC/M-3)
X/O * 4.63E-04 (SEC/M-3)
X/O DEPLETED * 4.63E-04 (SEC/M-3)
DELTA * 1.24E-04 (1/M-2)

YV - SEWAGE CONTAM - ASSUMED SRC - 2 ACR
ES - SHIELD F * 0.012 - 104 HR OCCUP.

THE FOLLOWING 1 NUCLIDES WERE USED IN THIS CALCULATION

6 PATHWAYS CONSIDERED
GROUND PLANK YES
INHALATION YES
STORED VEGETABLES YES
LEAFY VEGETABLES YES
COW MILK YES
GOAT MILK NO
MEAT YES

NUCLIDE RELEASE CURIES
25 MN 54 1.46E-06

VARIABLE	VEGETABLES		COW MILK		GOAT MILK		MEAT	
	STORED	LEAFY	PASTURE	STORED	PASTURE	STORED	PASTURE	STORED
YV AGRICULTURAL PRODUCTIVITY (KG/M-2)	2.00	2.00	.70	2.00	.70	2.00	.70	2.00
P SOIL SURFACE DENSITY (KG/M-2)	240.00	240.00	240.00	240.00	240.00	240.00	240.00	240.00
T TRANSPORT TIME TO USER (HRS)	8766.00	8766.00	48.00	48.00	48.00	48.00	48.00	48.00
TR SOIL EXPOSURE TIME (HRS)	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00
TE CROP EXPOSURE TIME TO PLUME (HRS)	.00	.00	.00	.00	.00	.00	.00	.00
TH HOLDUP AFTER HARVEST (HRS)	1440.00	24.00	.00	2160.00	.00	2160.00	.00	2160.00
OF ANIMALS DAILY FEED (KG/DAY)			50.00	50.00	.00	50.00	.00	50.00
FP FRACTION OF YEAR ON PASTURE			.50		.50		.50	
FB FRACTION OF PASTURE WHEN ON PASTURE			1.00		1.00		1.00	
FL FRACTION OF STORED VEG GROWN IN GARDEN	1.00							
FI FRACTION OF LEAFY VEG GROWN IN GARDEN		1.00						
HI FRACTION ELEMENTAL IODINE = .500								
H ABSOLUTE HUMIDITY = 3.60 (GM/M-3)								
PC FRACTIONAL EQUILIBRIUM RATIO FOR C-14 = 1.000								

AGE	DOSE DELIVERED TO EACH ORGAN FROM ALL PATHWAYS COMBINED												
	VEG	LEAFY VEG	MILK	MEAT	INHALATION	BONE	LIVER	KIDNEY	LUNG (MBER)	GI-LLI	THYROID	WHOLE BODY	SKIN
ADULT	520.00	64.00	310.00	110.00	8000.00	7.54E-05	1.12E-04	8.63E-05	1.03E-04	1.87E-04	7.54E-05	8.24E-05	8.84E-05
TEEN	630.00	42.00	400.00	85.00	8000.00	7.54E-05	1.39E-04	9.15E-05	1.18E-04	1.82E-04	7.54E-05	8.61E-05	8.84E-05
CHILD	520.00	26.00	330.00	41.00	3700.00	7.54E-05	1.54E-04	9.74E-05	1.09E-04	1.81E-04	7.54E-05	9.43E-05	8.84E-05
INFANT	.00	.00	330.00	.00	1400.00	7.54E-05	2.77E-05	7.58E-05	9.88E-05	7.60E-05	7.54E-05	7.58E-05	8.84E-05

B.5.2 Impact due to 1 uCi of Co 60

PROGRAM ATHORDS
 YANKEE ATOMIC ELECTRIC COMPANY
 DEC, 1985 REV. 7

REGULATORY GUIDE 1.109, APPENDIX C
 MODELS FOR CALCULATING DOSE VIA
 ADDITIONAL PATHWAYS FROM RADIOIODINES
 AND OTHER RADIONUCLIDES DISCHARGED
 TO THE ATMOSPHERE NOV, 1977 LIBRARY

SECTOR = FLD
 DISTANCE = 0 (METERS)
 X/D = 4.63E-04 (SEC/N-3)
 X/B DEFLECTED = 4.63E-04 (SEC/N-3)
 DELTA = 1.24E-04 (1/N-2)
 VY - SEWAGE TANK - ASSUMED SOURCE - 2
 ACRES - SHIELD F = 0.012 - 104 HR OCCUP

THE FOLLOWING 1 NUCLIDES WERE USED IN THIS CALCULATION

6 PATHWAYS CONSIDERED
 GROUND PLANE YES
 INHALATION YES
 STORED VEGETABLES YES
 LEAFY VEGETABLES YES
 COW MILK YES
 GOAT MILK NO
 MEAT YES

NUCLIDE RELEASE
 CURIES
 27 CO 60 1.07E-06

VARIABLE	VEGETABLES	COW MILK	GOAT MILK	PASTURE	STORED	MEAT
YV AGRICULTURAL PRODUCTIVITY (KG/H-2)	2.00	.70	.70	2.00	2.00	2.00
P SOIL SURFACE DENSITY (KG/M-2)	240.00	240.00	240.00	240.00	240.00	240.00
T TRANSPORT TIME TO USER (HRS)	240.00	48.00	48.00	48.00	48.00	48.00
TR SOIL EXPOSURE TIME (HRS)	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00
TE CROP EXPOSURE TIME TO PLUME (HRS)	.00	.00	.00	.00	.00	.00
TN HOLDUP AFTER HARVEST (HRS)	1440.00	24.00	24.00	24.00	24.00	24.00
OF ANIMALS DAILY FEED (KG/DAY)		50.00	50.00	50.00	50.00	50.00
FP FRACTION OF YEAR ON PASTURE		1.00	1.00	1.00	1.00	1.00
FS FRACTION OF STORED VEG GROWN IN GARDEN	1.00					
FL FRACTION OF LEAFY VEG GROWN IN GARDEN	1.00					
FI FRACTION ELEMENTAL IODINE = .500						
H ABSOLUTE HUMIDITY = 5.60 (GM/M-3)						
FC FRACTIONAL EQUILIBRIUM RATIO FOR C-14 = 1.000						

AGE	VEG (KG/YR)	MILK (L/YR)	MEAT (KG/YR)	INHALATION (H-3/YR)	LEAFY VEG (KG/YR)	USAGE FACTORS
ADULT	520.00	64.00	310.00	110.00	8000.00	2.21E-04
TEEN	630.00	42.00	400.00	65.00	3700.00	2.21E-04
CHILD	520.00	26.00	330.00	41.00	1400.00	2.21E-04
INFANT	.00	.00	330.00	.00	1400.00	2.21E-04

DOSE DELIVERED TO EACH ORGAN FROM ALL PATHWAYS COMBINED	WHOLE BODY	SKIN	THYROID	LUNG (HRS)	KIDNEY	LIVER	BONE
ADULT	2.34E-04	2.60E-04	2.21E-04	3.14E-04	2.21E-04	2.28E-04	2.21E-04
TEEN	2.43E-04	2.60E-04	2.21E-04	3.57E-04	2.21E-04	2.31E-04	2.21E-04
CHILD	2.65E-04	2.60E-04	2.21E-04	3.32E-04	2.21E-04	2.36E-04	2.21E-04
INFANT	2.23E-04	2.60E-04	2.21E-04	2.92E-04	2.21E-04	2.22E-04	2.21E-04

B.5.3 Impact due to 1 uCi of Zn 65

PROGRAM ATHODOS
YANKEE ATOMIC ELECTRIC COMPANY
DEC. 1985 REV. 7

REGULATORY GUIDE 1.109, APPENDIX C
MODELS FOR CALCULATING DOSE VIA
ADDITIONAL PATHWAYS FROM RADIOIODINES
AND OTHER RADIONUCLIDES DISCHARGED
TO THE ATMOSPHERE NOV. 1977 LIBRARY

SECTOR FLD 0 (METERS)
DISTANCE 1 4.63E-04 (SEC/M-3)
X/R DEPLETED 4.63E-04 (SEC/M-3)
DELTA 1.24E-04 (1/M-2)
VY - SEWAGE CONTAM - ASSUMED SOURCE - 2
ACRES - SHIELD F - 0.012 - 104 HR OCCUP

THE FOLLOWING 1 NUCLIDES WERE USED IN THIS CALCULATION

NUCLIDE RELEASE
30 ZN 65 1.61E-06
CURIED

6 PATHWAYS CONSIDERED
GROUND PLANE YES
INHALATION YES
STORED VEGETABLES YES
LEAFY VEGETABLES YES
COW MILK YES
GOAT MILK NO
MEAT YES

VARIABLE	VEGETABLES STORED	VEGETABLES LEAFY	COW MILK PASTURE	COW MILK STORED	GOAT MILK PASTURE	GOAT MILK STORED	PASTURE STORED	MEAT STORED
YV AGRICULTURAL PRODUCTIVITY (KG/M-2)	2.00	2.00	.70	2.00	.70	2.00	.70	2.00
P SOIL SURFACE DENSITY (KG/M-2)	240.00	240.00	240.00	240.00	240.00	240.00	240.00	240.00
T TRANSPORT TIME TO USER (HRS)	8766.00	8766.00	48.00	48.00	48.00	48.00	480.00	480.00
TR SOIL EXPOSURE TIME (HRS)	.00	.00	8766.00	8766.00	8766.00	8766.00	8766.00	8766.00
TE CROP EXPOSURE TIME TO FLUME (HRS)	1440.00	24.00	.00	.00	.00	.00	.00	.00
TH HOLDUP AFTER HARVEST (HRS)	.00	.00	.00	.00	.00	.00	.00	.00
GF ANIMALS DAILY FEED (KG/DAY)	.00	.00	50.00	50.00	6.00	2160.00	50.00	2160.00
FF FRACTION OF YEAR ON PASTURE	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
FS FRACTION PASTURE WHEN ON PASTURE	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
FG FRACTION OF STORED VEG GROWN IN GARDEN	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
FL FRACTION OF LEAFY VEG GROWN IN GARDEN	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
FI FRACTION ELEMENTAL IODINE = .500								
H ABSOLUTE HUMIDITY = 5.60 (GM/M-3)								
PC FRACTIONAL EQUILIBRIUM RATIO FOR C-14 = 1.000								

AGE	VEG (KG/YR)	LEAFY VEG (KG/YR)	MILK (L/YR)	MEAT (KG/YR)	INHALATION (M-3/YR)	DOSE DELIVERED TO EACH ORGAN FROM ALL PATHWAYS COMBINED
ADULT	520.00	64.00	310.00	110.00	8000.00	5.20E-03
TEEN	630.00	42.00	400.00	65.00	8000.00	5.20E-03
CHILD	520.00	26.00	330.00	41.00	3700.00	5.20E-03
INFANT	.00	.00	330.00	.00	1400.00	5.20E-03

B.5.4 Impact due to 1 uCi of Cs 134

PROGRAM ATHRODS
YANKEE ATOMIC ELECTRIC COMPANY
DEC. 1982 REV. 7

REGULATORY GUIDE 1.107, APPENDIX C
MODELS FOR CALCULATING DOSE VIA
ADDITIONAL PATHWAYS FROM RADIOIODINES
AND OTHER RADIONUCLIDES DISCHARGED
TO THE ATMOSPHERE NOV. 1977 LIBRARY

SECTOR FLD 0 (METERS)
DISTANCE - 4.03E-04 (SEC/M-3)
X/D - 4.03E-04 (SEC/M-3)
R/D DEPLETED - 4.03E-04 (SEC/M-3)
DELTA - 1.24E-04 (1/M-2)
VY - DEWAGE CONTAM - ASSUMED SOURCE - 2
ACRES - SHIELD F - 0.012 - 104 HR DCCUP
6 PATHWAYS CONSIDERED

THE FOLLOWING 1 NUCLIDES WERE USED IN THIS CALCULATION

NUCLIDE RELEASE CURIES
55 CS 134 1.18E-06

GROUND FLAME YES
INHALATION YES
STORED VEGETABLES YES
LEAFY VEGETABLES YES
COW MILK YES
GOAT MILK NO
MEAT YES

VARIABLE	VEGETABLES STORED	LEAFY	FASTURE	STORED	GOAT MILK	FASTURE	STORED	MEAT	STORED
YV AGRICULTURAL PRODUCTIVITY (KG/M-2)	2.00	2.00	.70	2.00	.70	2.00	2.00	.70	2.00
F SOIL SURFACE DENSITY (KG/M-2)	240.00	240.00	240.00	240.00	240.00	240.00	240.00	240.00	240.00
T TRANSPORT TIME TO USER (HRS)	8766.00	8766.00	48.00	48.00	8766.00	48.00	48.00	8766.00	48.00
TR SOIL EXPOSURE TIME TO FLUME (HRS)	.00	.00	.00	.00	.00	.00	.00	.00	.00
TE CROP EXPOSURE TIME TO FLUME (HRS)	1440.00	24.00	50.00	50.00	.00	2160.00	6.00	2160.00	50.00
TH HOLDUP AFTER HARVEST (HRS)			.50	.50					
GF ANIMALS DAILY FEED (KG/DAY)			1.00	1.00					
FS FRACTION OF YEAR ON PASTURE									
FB FRACTION OF STORED VEG GROWN IN GARDEN	1.00								
FL FRACTION OF LEAFY VEG GROWN IN GARDEN									
FI FRACTION ELEMENTAL IODINE = .500									
H ABSOLUTE HUMIDITY = 5.60 (GM/M-3)									
FC FRACTIONAL EQUILIBRIUM RATIO FOR C-14 = 1.000									

AGE	VEG (KG/YR)	MILK (LI/YR)	MEAT (KG/YR)	INHALATION (M-3/YR)	USAGE FACTORS	DOSE DELIVERED TO EACH ORGAN FROM ALL PATHWAYS COMBINED
ADULT	520.00	84.00	310.00	8000.00		1.56E-04
TEEN	630.00	42.00	400.00	8000.00		1.56E-04
CHILD	570.00	26.00	330.00	3700.00		1.56E-04
INFANT	.00	.00	330.00	1400.00		1.56E-04

B.5.5 Impact due to 1 uCi of Cs 137

PROGRAM METHODS
YANKEE ATOMIC ELECTRIC COMPANY
DEC, 1985 REV. 7

REGULATORY GUIDE 1.109, APPENDIX C
MODELS FOR CALCULATING DOSE VIA
ADDITIONAL PATHWAYS FROM RADIOIODINES
AND OTHER RADIONUCLIDES DISCHARGED
TO THE ATMOSPHERE NOV.1977 LIBRARY

SECTOR = FLD
DISTANCE = 0 (METERS)
X/O = 4.63E-04 (SEC/M-3)
X/O DEPLETED = 4.63E-04 (SEC/M-3)
DELTA = 1.24E-04 (1/M-2)
VY - SEWAGE CONTAM - ASSUMED SOURCE - 2
ACRES - SHIELD F = 0.012 - 104 HR OCCUP

THE FOLLOWING 1 NUCLIDES WERE USED IN THIS CALCULATION

NUCLIDE RELEASE
55 CS 137 1.01E-06

6 PATHWAYS CONSIDERED
GROUND FLAME YES
INHALATION YES
STORED VEGETABLES YES
LEAFY VEGETABLES YES
COW MILK YES
GOAT MILK NO
MEAT YES

VARIABLE	VEGETABLES STORED	LEAFY	COW MILK PASTURE	GOAT MILK PASTURE	MEAT STORED
YV AGRICULTURAL PRODUCTIVITY (KG/M-2)	2.00	2.00	.70	.70	2.00
P SOIL SURFACE DENSITY (KG/M-2)	240.00	240.00	240.00	240.00	240.00
T TRANSPORT TIME TO USER (HRS)	8766.00	8766.00	48.00	48.00	480.00
TR SOIL EXPOSURE TIME (HRS)	.00	.00	8766.00	8766.00	8766.00
TE CROP EXPOSURE TIME TO PLUME (HRS)	1440.00	24.00	.00	.00	.00
TH HOLDUP AFTER HARVEST (HRS)			.00	.00	.00
OF ANIMALS DAILY FEED (KG/DAY)			50.00	50.00	2160.00
FF FRACTION OF YEAR ON PASTURE			.50	.50	50.00
FS FRACTION OF PASTURE WHEN ON PASTURE			1.00	1.00	1.00
FG FRACTION OF STORED VEG GROWN IN GARDEN	1.00				
FL FRACTION OF LEAFY VEG GROWN IN GARDEN		1.00			
FI FRACTION OF ELEMENTAL IODINE = .500					
H ABSOLUTE HUMIDITY = 5.60 (GM/M-3)					
FC FRACTIONAL EQUILIBRIUM RATIO FOR C-14 = 1.000					

AGE	VEG (KG/YR)	LEAFY VEG (KG/YR)	MILK (L/YR)	MEAT (KG/YR)	INHALATION (M-3/YR)	USAGE FACTORS
ADULT	520.00	84.00	310.00	110.00	8000.00	3.86E-04 5.07E-04 2.09E-04 1.06E-04 6.33E-05 5.46E-05 3.52E-04 6.37E-05
TEEN	630.00	42.00	400.00	63.00	8000.00	5.97E-04 7.75E-04 3.00E-04 1.50E-04 6.48E-05 5.46E-05 3.04E-04 6.37E-05
CHILD	520.00	26.00	336.00	41.00	3700.00	1.23E-03 1.28E-03 4.53E-04 1.98E-04 6.33E-05 5.46E-05 2.35E-04 6.37E-05
INFANT	.00	.00	330.00	.00	1400.00	5.94E-04 6.86E-04 2.24E-04 1.23E-04 5.66E-05 5.46E-05 9.94E-05 6.37E-05