



Carolina Power & Light Company

JUN 21 1989

SERIAL: NLS-89-177

United States Nuclear Regulatory Commission  
ATTENTION: Document Control Desk  
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2  
DOCKET NOS. 50-325 & 50-324/LICENSE NOS. DPR-71 & DPR-62  
REQUEST FOR CORRECTED PAGE  
(NRC TAC NOS. 72061 AND 72062)

Gentlemen:

On May 22, 1989, the NRC issued Amendments 130 and 160 to the Technical Specifications for the Brunswick Steam Electric Plant (BSEP), Units 1 and 2. The amendments were issued as requested by Carolina Power & Light Company (CP&L) in a letter dated February 1, 1989.

During subsequent review of these amendments, the Company determined that an error was introduced in Table 4.3.6.1-1 (Page 3/4 3-92) of the BSEP-2 Technical Specifications. This error was introduced in the February 1, 1989 license amendment request referenced above and was not discovered until after Amendment 160 had been issued. Amendments 130 and 160 were intended to remove only the instrument tag numbers from Table 4.3.6.1-1 and administratively revise the table. No changes to surveillance requirements were intended.

Technical Specification Table 4.3.6.1-1, Item 2 provides the surveillance requirements for the reactor vessel pressure - high instrumentation relative to the anticipated transient without scram (ATWS) recirculation pump trip system. A channel check is not necessary for the transmitter portion of the logic, and a channel calibration is required only at refueling intervals. There should be a reference to Footnote (a) in the channel check column associated with the transmitter to indicate that a channel check is not required. Also, there should be a reference to Footnote (b) in the channel calibration column to state that transmitters are exempt from monthly channel calibrations. These references do not exist on the BSEP-2 Technical Specification page issued by Amendment 160. The correct references are shown on the corresponding Technical Specification Table for BSEP-1. Therefore, CP&L requests that the NRC re-issue Technical Specification Page 3/4 3-92 for BSEP-2 to correct this error. A corrected page for BSEP-2 is provided in Enclosure 1.

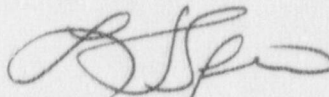
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Please refer any questions regarding this submittal to Mr. William R. Murray at (919) 546-4661.

Yours very truly,



Leonard I. Loflin  
Manager  
Nuclear Licensing Section

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Enclosure

cc: Mr. S. D. Ebnetter  
Mr. W. H. Ruland  
Mr. E. G. Tourigny

ENCLOSURE 1

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2  
NRC DOCKETS 50-325 & 50-324  
OPERATING LICENSES DPR-71 & DPR-62  
REQUEST FOR CORRECTED PAGE  
(NRC TAC NOS. 72061 AND 72062)

CORRECTED TECHNICAL SPECIFICATION PAGE - UNIT 2

TABLE 4.3.6.1-1

ATWS RECIRCULATION PUMP TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor Vessel Water Level - Low, Level 2			
Transmitter:	NA (a)	NA	R (b)
Trip Logic:	D	M	M
2. Reactor Vessel Pressure - High			
Transmitter:	NA (a)	M	R (b)
Trip Logic:	D	M	M

(a) The transmitter channel check is satisfied by the trip unit channel check. A separate transmitter check is not required.

(b) Transmitters are exempted from the monthly channel calibration.