



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

April 28, 1989

Docket No. 50-219

Mr. E. E. Fitzpatrick  
Vice President and Director  
Oyster Creek Nuclear Generating Station  
Post Office Box 388  
Forked River, New Jersey 08731

Dear Mr. Fitzpatrick:

SUBJECT: DRYWELL CONTAINMENT - OYSTER CREEK NUCLEAR GENERATING STATION  
(TAC NO. 72C29)

In a letter dated September 12, 1988, GPU Nuclear Corporation (GPUN/the licensee) committed to provide an assessment of the drywell corrosion to date (12 Refueling outage) and the projected corrosion rate for the following operating cycle. In a letter dated February 9, 1989, GPUN provided the staff with this information. The pertinent information as given by GPUN is summarized in Table 1 (enclosed).

On the basis of the corrosion rate listed in Table 1, the licensee concluded that the most limiting condition is in the sand bed region of the drywell shell and the drywell shell thickness is projected to be acceptable until June 1992. In an attempt to reduce the corrosion rate, the licensee has (1) installed cathodic protection in selected sand bed locations, (2) taken steps to eliminate water leakage from reactor building equipment and refueling cavity, and (3) drained water from sand bed region. In order to assure the structural integrity of the drywell, the licensee has committed periodic UT thickness measurements of the drywell shell at all outages of opportunity. The licensee emphasized that the projection to June 1992 was based on conservative approaches.

Based on our review of the information provided by GPUN, we concur with the licensee that with the actions taken and to be taken by the licensee to ensure drywell integrity, and that plant operation can continue to the 13R refueling outage. In the event that efforts to arrest corrosion are not successful the licensee has argued that existing conservatism would still allow operation. However, the staff has reservations due to the fact that such conservatisms are not easily quantifiable and are required in assuring drywell adequacy for the protection of public health and safety. The licensee is required to perform

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thickness measurements and reconfirm the adequacy of the containment integrity at future outages of opportunity, including forced outages requiring drywell entry during the next cycle, but no later than prior to the resumption of power operation following the 13R refueling outage.

Sincerely,

/s/

Alexander W. Dromerick, Project Manager  
Project Directorate I-4  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

Enclosure:

Table 1

cc w/enclosure:

See next page

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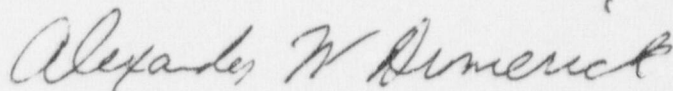
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Mr. E. E. Fitzpatrick

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Sincerely,



Alexander W. Dromerick, Project Manager  
Project Directorate I-4  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

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Table 1

cc w/enclosure:  
See next page



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Oyster Creek Nuclear  
Generating Station

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TABLE 1

Location (Elevation)	Thickness (Inch)			Corrosion Rate (MPY)
	Nominal	Code Req'd	UT Measured	
8'-11 3/4" to 12'-3" (sand bed region)	1.15	0.700	0.838	-27.6 ± 6.1
50' - 2"	0.77	.725	0.750	-4.3 ± .03
87' - 5"	0.64	.639	0.620*	0

\*Accepted on the basis of data from certified material test reports (CMTRs) and no corrosion after plant operation (corrosion occurred during erection).