



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CONSUMERS POWER COMPANY

BIG ROCK POINT PLANT

DOCKET NO. 50-155

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 99  
License No. DPR-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Consumers Power Company (the licensee) dated May 25, 1989, and supplemented by letter of June 30, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. DPR-6 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 99, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*William C. Long*

for Lawrence Yandell, Acting Director  
Project Directorate III-1  
Division of Reactor Projects - III, IV, V  
& Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: July 31, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 99

FACILITY OPERATING LICENSE NO. DPR-6

DOCKET NO. 50-155

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3-7

3-8

INSERT

3-7

3-8



After cutting into the sphere or its components, or any disassembly of components that would affect sphere integrity, an individual component leakage rate or an integrated leakage rate test, whichever is deemed more appropriate by the operator, shall be performed, with air at a pressure not less than 11.5 psig. It shall be permissible to employ a leak detection test in lieu of the above for insuring containment integrity following disassembly of the emergency condenser or the gasketed, bolted closure of the coaxial cable electrical penetrations.

The individual component leakage rate determined from the above tests when combined with the previously measured integrated leakage rate shall yield an overall leakage rate not greater than 0.5%/day at 27 psig.

- (d) A containment sphere integrated leakage rate test shall be performed at a pressure not less than 11.5 psig in conformance with the criteria specified in Appendix J of 10 CFR Part 50. Three tests shall be conducted at  $40 \pm 10$  month intervals during each 10-year service period.
- (e) If any containment sphere integrated leakage rate test (ILRT) fails to meet the specifications of 0.5%/day of contained atmosphere (weight basis) at the design pressure (27 psig) or extrapolated to the design pressure, the test schedule for subsequent ILRTs shall be reviewed and approved by the Commission. If two consecutive ILRTs fail to meet these specifications, then an ILRT shall be performed at least every 18 months until two consecutive ILRTs meet the acceptance criteria, at which time the above test schedule may be resumed.
- (f) The accuracy of each ILRT shall be verified by a supplemental test in accordance with Appendix J of 10 CFR Part 50 which confirms the accuracy of the test by (1) verifying the difference between the supplemental test data and the ILRT is within  $0.25 L_g$  or  $0.25 L_t$ , and (2) continuing the test for a duration sufficient to establish accurately the change in leakage rate between the ILRT and supplemental test.

- (g) All leakage rates determined by a test pressure less than the applicable design pressure (containment design or design basis accident) shall be corrected using the following formula:

$$L_t = L_e (P_t/P_e)^{\frac{1}{2}}$$

$L_t$  = % maximum allowable leakage rate, at test pressure.

$L_e$  = % leakage rate, at extrapolated pressure.

$P_t$  = Test pressure (PSIG).

$P_e$  = Extrapolated pressure (PSIG).

Acceptance criteria on allowable leakage for the ILRT is

$$.75L_t.$$

- (h) The RDS containment penetration assemblies seal pressure shall be examined at six-month intervals.