

Docket



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 14, 1989

Docket No. 50-482

Mr. Bart D. Withers
President and Chief Executive Officer
P. O. Box 411
Burlington, Kansas 66839

Dear Mr. Withers:

The purpose of this letter is to request your prompt review and reporting of the status of implementation of TMI Action Plan Items at your facility. As an enclosure we are providing a printout of all TMI items annotated where our tracking system indicates that implementation is not complete. For the purpose of this request, implementation should be considered complete when, to the best of your knowledge, all actions necessary to satisfactorily meet the requirements of NUREG-0737 and NUREG-0737 Supplement 1 have been completed.

Where you have not fully completed an item as described above, we request that you mark-up the enclosure to reflect your projected implementation date. Add a short note identifying remaining work (e.g., hardware, procedures, training, technical specifications). More explicit instructions are provided as part of the enclosure. We request that this information reach this office by April 18, 1989.

The information we are requesting will be utilized to validate our existing data base so that we can accurately respond to congressional inquiries concerning the status of implementation of TMI Action Plan Items. In view of the importance of the information we request your personal attention and certification that the information is correct to the best of your knowledge.

This request is covered by Office of Management and Budget Clearance Number 3150-0011, which expires December 31, 1989. The estimated average burden hours is 40 person hours per owner response, including searching data source, gathering and analyzing the data, and preparing the required letter. Send comments regarding this burden estimate or any other aspect of this collection of information, including suggestions for reducing this burden, to the Records and Reports Management Branch, Division of Information Support Services, Office of Information Resources Management, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555; and to the Paperwork Reduction Project (3150-0011), Office of Management and Budget, Washington, D.C. 20503.

DF-01
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April 14, 1989

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Your prompt attention to this matter within the requested time frame is appreciated.

Sincerely,

/s/

Thomas E. Murley, Director
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

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PM:PD4 *PM*
DPickett *for*
4/14/84 *DP*

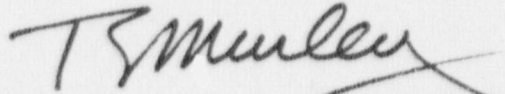
PM:PD4 *PM*
PO'Connor
4/14/84

D:PD4 *MAC*
JCalvo
4/14/84

- 2 -

Your prompt attention to this matter within the requested time frame is appreciated.

Sincerely,

A handwritten signature in dark ink, appearing to read "T. Murley", with a stylized flourish extending from the end of the name.

Thomas E. Murley, ~~Director~~
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

Mr. Bart D. Withers
Wolf Creek Nuclear Operating Corporation

Wolf Creek Generating Station
Unit No. 1

cc:

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Mr. Otto Maynard, Manager Licensing
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GUIDELINES FOR UPDATING TMI STATUS

1. The enclosure provides a complete listing of all TMI requirements from NUREG-0737 and NUREG-0737 Supplement 1. Regulatory Guide 1.97 requirements, although addressed in Supplement 1 to NUREG-0737, are excluded since they did not originate from the TMI Action Plan. This listing consists of a matrix of items applicable to both operating reactors and operating license applicants (NTOLS) as of November, 1980, as shown in NUREG-0737 Enclosure 1 and 2, respectively. All items which are not listed identically in Enclosure 1 and 2 are marked with asterisks to indicate difference between operating reactor and NTOL requirements. Whenever these differences occur, enter data in accordance with the appropriate title, regardless of the TMI Action Plan item number listed. Your NRC Project Manager has indicated which issues are shown as unimplemented in the NRC tracking system. Please review the entire listing for each licensed reactor unit. Where an item was not applicable for your facility, mark N/A in the implementation status column.
2. Where an item was applicable to your facility but no changes were necessary, mark NC in the implementation status column.
3. Where an item was applicable to your facility and changes are complete mark C in the implementation status column. If the date is readily available (month and year) we request you provide it, however, C is sufficient.
4. Where an item was applicable to your facility and is not fully implemented, provide your projected implementation date (month and year) and a short note identifying the outstanding item (e.g. hardware, procedures, training, technical specifications).

LICENSEE IMPLEMENTATION STATUS

ISSUE NUMBER	MULTI-PLANT ACTION NO.	ISSUE TITLE
I.D.2.2	F009	*PLANT-SAFETY PARAMETER DISPLAY CONSOLE - INSTALLED.
I.D.2.3	F009	PLANT-SAFETY PARAMETER DISPLAY CONSOLE - FULLY IMPLEMENTED.
*I.G.1.1		TRAINING DURING LOW-POWER TESTING - PROPOSE TESTS.
*I.G.1.2		TRAINING DURING LOW-POWER TESTING - SUBMIT ANAL. & PROCS.
*I.G.1.3		TRAINING DURING LOW-POWER TESTING - TRAINING & RESULTS.
II.B.1.1		REACTOR-COOLANT SYSTEM VENTS - DESIGN VENTS.
II.B.1.2	F010	REACTOR-COOLANT SYSTEM VENTS - INSTALL VENTS (LL CAT B).
II.B.1.3	F010	REACTOR-COOLANT SYSTEM VENTS - PROCEDURES.
II.B.2.1		PLANT SHIELDING - REVIEW DESIGNS.
*II.B.2.2		PLANT SHIELDING - CORRECTIVE ACTIONS TO ASSURE ACCESS.
*II.B.2.3		PLANT SHIELDING - PLANT MODIFICATIONS (LL CAT B).
*II.B.2.4	F011	PLANT SHIELDING - EQUIPMENT QUALIFICATION- NOT TRACKED AS A TMI ACTION ITEM.
*II.B.3.1		POSTACCIDENT SAMPLING - INTERIM SYSTEM.
*II.B.3.2		POSTACCIDENT SAMPLING - CORRECTIVE ACTIONS.
*II.B.3.3		POSTACCIDENT SAMPLING - PROCEDURES.
*II.B.3.4	F012	POSTACCIDENT SAMPLING - PLANT MODIFICATIONS (LL CAT B).
II.B.4.1	F013	TRAINING FOR MITIGATING CORE DAMAGE - DEVELOP TRAINING PROGRAM.
*II.B.4.2.A	F013	TRAINING FOR MITIGATING CORE DAMAGE - INITIAL.
*II.B.4.2.B	F013	TRAINING FOR MITIGATING CORE DAMAGE - COMPLETE.
II.D.1.1		RELIEF & SAFETY VALVE TEST REQUIREMENTS - SUBMIT PROGRAM.
*II.D.1.2.A		RELIEF & SAFETY VALVE TEST REQUIREMENTS - COMPLETE TESTING.
*II.D.1.2.B	F014	RELIEF & SAFETY VALVE TEST REQUIREMENTS - PLANT SPECIFIC REPORT.
II.D.1.3		RELIEF & SAFETY VALVE TEST REQUIREMENTS - BLOCK-VALVE TESTING.
*II.D.3.1		VALVE POSITION INDICATION - INSTALL DIRECT INDICATIONS OF VALVE POS.
*II.D.3.2		VALVE POSITION INDICATION - TECH SPECS.
*II.E.1.1.1	F015	AFS EVALUATION-ANALYSIS.
(SEE NOTE 1)		
*II.E.1.1.2	F015	AFS EVALUATION-SHORT TERM MODS.
(SEE NOTE 2)		
II.E.1.1.3	F015	AFS -LONG TERM MODS.
(SEE NOTE 3)		

NOTE 1 - THE ITEM LISTED IS FROM NUREG-0737, ENCLOSURE 2 AND IS APPLICABLE TO NTOLS'S ONLY

NOTE 2 - THE ITEM LISTED IS FOR ALL PLANTS (OPERATING REACTORS AND NTOL'S)

NOTE 3 - THE ITEM LISTED IS FOR ALL PLANTS (OPERATING REACTORS AND NTOL'S)

ISSUE NUMBER
MULTI-PLANT
ACTION NO.

ISSUE TITLE

LICENSEE IMPLEMENTATION STATUS

II.E.1.2.1.A	AFS INITIATION & FLOW - CONTROL GRADE.....	
II.E.1.2.1.B	AFS INITIATION & FLOW - SAFETY GRADE.....	
II.E.1.2.2.A	AFS INITIATION & FLOW - FLOW INDICATION CONTROL GRADE.....	
II.E.1.2.2.B	AFS INITIATION & FLOW - LL CAT A TECH SPECS.....	
II.E.1.2.2.C	AFS INITIATION & FLOW - SAFETY GRADE.....	
II.E.3.1.1	EMERGENCY POWER FOR PRESSURIZER HEATERS - UPGRADE POWER SUPPLY.....	
II.E.3.1.2	EMERGENCY POWER FOR PRESSURIZER HEATERS - TECH SPECS.....	
II.E.4.1.1	DEDICATED HYDROGEN PENETRATIONS - DESIGN.....	
II.E.4.1.2	DEDICATED HYDROGEN PENETRATIONS - REVIEW & REVISE H2 CONTROL PROC.....	
II.E.4.1.3	DEDICATED HYDROGEN PENETRATION - INSTALL.....	
II.E.4.2.1-4	CONTAINMENT ISOLATION DEPENDABILITY - IMP. DIVERSE ISOLATION.....	
II.E.4.2.5.A	CONTAINMENT ISOLAT. DEPENDABILITY - CNMT PRESS. SETPT. SPECIFY PRESS.....	
II.E.4.2.5.B	CONTAINMENT ISOLATION DEPENDABILITY - CNMT PRESSURE SETPT. MODS.....	
II.E.4.2.6	CONTAINMENT ISOLATION DEPENDABILITY - CNMT PURGE VALVES.....	
II.E.4.2.7	CONTAINMENT ISOLATION DEPENDABILITY - RADIATION SIGNAL ON PURGE VALVES.....	
II.E.4.2.8	CONTAINMENT ISOLATION DEPENDABILITY - TECH SPECS.....	
II.F.1.1	ACCIDENT - MONITORING - PROCEDURES.....	
II.F.1.2.A	ACCIDENT - MONITORING - NOBLE GAS MONITOR.....	
II.F.1.2.B	ACCIDENT - MONITORING - IODINE/PARTICULATE SAMPLING.....	
II.F.1.2.C	ACCIDENT - MONITORING - CONTAINMENT HIGH-RANGE MONITOR.....	
II.F.1.2.D	ACCIDENT - MONITORING - CONTAINMENT PRESSURE.....	
II.F.1.2.E	ACCIDENT - MONITORING - CONTAINMENT WATER LEVEL.....	
II.F.1.2.F	ACCIDENT - MONITORING - CONTAINMENT HYDROGEN.....	
II.F.2.1	INSTRUMENTATION FOR DETECT. OF INADEQUATE CORE COOLING - PROCEDURES.....	
II.F.2.2	INSTRUMENTATION FOR DETECT. OF INADEQUATE CORE COOLING - SUBCOOL METER.....	
II.F.2.3	INSTRUMENTATION FOR DETECT. OF INADEQUATE CORE COOLING - DESC. OTHER.....	
II.F.2.4	INSTRUMENTATION FOR DETECT. OF INADEQUATE CORE CLING INSTILL ADD'L INSTRUMENTATION.....	
II.G.1.1	POWER SUPP. FOR PRESSURIZER RELIEF, BLOCK VALVES & LEVEL IND. - UPGRADE.....	
II.G.1.2	POWER SUPP. FOR PRESSURIZER RELIEF, BLOCK VALVES & LEVEL IND. - TECH SP.....	
II.K.1 (Oper. Reactors Only)	IE BULLETINS - 79-05, 79-06, 79-08.....	

ISSUE NUMBER
MULTI-PLANT
ACTION NO.

ISSUE TITLE

LICENSEE IMPLEMENTATION STATUS

*11.K.1.5	IE BULLETINS - REVIEW ESF VALVES.....	
*11.K.1.10	IE BULLETINS - OPERABILITY STATUS.....	
*11.K.1.20	IE BULLETINS - PROMPT MANUAL REACTOR TRIP.....	
*11.K.1.21	IE BULLETINS - AUTO SG ANTICIPATORY REACTOR TRIP.....	
*11.K.1.22	IE BULLETINS - AUX. HEAT REM SYSTEM, PROC.....	
*11.K.1.23	IE BULLETINS - RV LEVEL, PROCEDURES.....	
*11.K.2.2	ORDERS ON B&W PLANTS - PROCEDURES TO CONTROL AFW IND OF ICS.....	
*11.K.2.8	ORDERS ON B&W PLANTS - UPGRADE AFW SYSTEM.....	
11.K.2.9	ORDERS ON B&W PLANTS - FEMA ON ICS.....	F027
11.K.2.10	ORDERS ON B&W PLANTS - SAFETY-GRADE TRIP.....	F028
*11.K.2.11	ORDERS ON B&W PLANTS - OPERATOR TRAINING.....	F029
11.K.2.13	ORDERS ON B&W PLANTS - THERMAL MECHANICAL REPORT (CE & W PLANTS ALSO).....	F030
11.K.2.14	ORDERS ON B&W PLANTS - LIFT FREQUENCY OF PORV'S & SV'S.....	F031
11.K.2.15	ORDERS ON B&W PLANTS - EFFECTS OF SLUG FLOW.....	
11.K.2.16	ORDERS ON B&W PLANTS - RCP SEAL DAWFGE.....	F032
11.K.2.17	ORDERS ON B&W PLANTS - VOIDING IN RCS (CE & W PLANTS ALSO).....	F033
11.K.2.19	BENCHMARK ANALYSIS OF SEQUENTIAL AFW FLOW TO ONCE THROUGH STM GENERATOR.....	
*11.K.2.20	ORDERS ON B&W PLANTS - SYSTEM RESPONSE TO SB LOCA.....	F035
*11.K.3.1.A	B&O TASK FORCE - AUTOMATIC PORV ISOLATION DESIGN.....	F036
*11.K.3.1.B	FINAL RECOMMENDATIONS, B&O TASK FORCE - AUTO PORV ISO TEST/INSTALL.....	
11.K.3.2	B&O TASK FORCE - REPORT ON PORV FAILURES.....	F037
11.K.3.3	B&O TASK FORCE - REPORTING SV & RV FAILURES AND CHALLENGES.....	F038
11.K.3.5.A	B&O TASK FORCE - AUTO TRIP OF RCP'S PROPOSED MODIFICATIONS.....	F039
11.K.3.5.B	B&O TASK FORCE - AUTO TRIP OF RCP'S MODIFICATIONS.....	F039
11.K.3.7	B&O TASK FORCE - EVALUATION OF PORV OPENING PROBABILITIES.....	
11.K.3.9	B&O TASK FORCE - PID CONTROLLER MODIFICATION.....	F040
11.K.3.10	B&O TASK FORCE - PROPOSED ANTICIPATORY TRIP MODIFICATIONS.....	F041
11.K.3.11	B&O TASK FORCE - JUSTIFY USE OF CERTAIN PORV.....	
11.K.3.12.A	B&O TASK FORCE - ANTICIPATORY TRIP ON TURBINE TRIP PROPOSED MODS.....	F042
11.K.3.12.B	B&O TASK FORCE - ANTICIPATORY TRIP ON TURBINE TRIP INSTALL MODS.....	
11.K.3.13.A	B&O TASK FORCE - HPCI & RCIC SYSTEM INITIATION LEVELS ANALYSIS.....	F043
11.K.3.13.B	B&O TASK FORCE - HPCI & RCIC INITIATION LEVELS MODIFICATION.....	F043

ISSUE NUMBER MULTI-PLANT
ACTION NO.

ISSUE TITLE

LICENSEE IMPLEMENTATION STATUS

*11 K.3.14	F044	B&O TASK FORCE	- ISO CONDENSER ISOLATION ON HIGH RAD	
11 K.3.15	F045	B&O TASK FORCE	- MODIFY HPCI & RCIC BRK DETECTION CIRCUITRY	
11 K.3.16A	F046	B&O TASK FORCE	- CHALLENGE & FAILURE OF RELIEF VALVES STUDY	
11 K.3.16.B	F046	B&O TASK FORCE	- CHALLENGE & FAILURE OF RELIEF VALVES MODIFICATIONS	
11 K.3.17	F047	B&O TASK FORCE	- ECC SYSTEM OUTAGES	
11 K.3.18.A	F048	B&O TASK FORCE	- ADS ACTUATION STUDY	
11 K.3.18.B	F048	B&O TASK FORCE	- ADS ACTUATION PROPOSED MODIFICATIONS	
11 K.3.18.C	F048	B&O TASK FORCE	- ADS ACTUATION MODIFICATIONS	
*11 K.3.19	F049	B&O TASK FORCE	- INTERLOCK RECIRCULATORY PUMP MODIFICATIONS	
*11 K.3.20	F050	B&O TASK FORCE	- LOSS OF SVC WATER AT BRP	
11 K.3.21.A	F050	B&O TASK FORCE	- RESTART OF CSS & LPCI LOGIC DESIGN	
11 K.3.21.B	F050	B&O TASK FORCE	- RESTART OF CSS & LPCI LOGIC DESIGN MODIFICATIONS	
11 K.3.22.A	F051	B&O TASK FORCE	- RCIC SUCTION VERIFICATION PROCEDURE	
11 K.3.22.B	F051	B&O TASK FORCE	- RCIC SUCTION MODIFICATION	
11 K.3.24	F052	B&O TASK FORCE	- SPACE COOLING FOR HPCI/RCI LOSS OF 1C POWER	
11 K.3.25.A	F053	B&O TASK FORCE	- POWER ON PUMP SEALS PROPOSED MODIFICATIONS	
11 K.3.25.B	F054	B&O TASK FORCE	- POWER ON PUMP SEALS MODIFICATIONS	
11 K.3.27	F055	B&O TASK FORCE	- COMMON REFERENCE LEVEL FOR Pumps	
11 K.3.28	F055	B&O TASK FORCE	- QUALIFICATION OF ADS ACCUMULATORS	
*11 K.3.29	F056	B&O TASK FORCE	- PERFORMANCE OF ISOLATION CONDENSERS	
11 K.3.30.A	F057	B&O TASK FORCE	- SCHEDULE FOR OUTLINE OF SB LOCA MODEL	
11 K.3.30.B	F057	B&O TASK FORCE	- SB LOCA MODEL, JUSTIFICATION	
11 K.3.30.C	F058	B&O TASK FORCE	- SB LOCA METHODS NEW ANALYSES	
11 K.3.31	F058	B&O TASK FORCE	- COMPLIANCE WITH LFR 50.46	
*11 K.3.40	F059	B&O TASK FORCE	- RCP SEAL DAMAGE - COVERED BY 11 K.2.16 AND 11 K.3.25	
*11 K.3.43	F060	B&O TASK FORCE	- EFFECTS OF SLUG FLOW - COVERED BY 11 K.2.15	
11 K.3.44	F061	B&O TASK FORCE	- EVALUATE TRANSIENT WITH SINGLE FAILURE	
11 K.3.45	F061	B&O TASK FORCE	- ANALYSES TO SUPPORT	
11 K.3.46	F062	B&O TASK FORCE	- RESPONSE TO LIST OF CONCERNS FROM ACRS CONSULTANT	
*11 K.3.57	F062	B&O TASK FORCE	- IDENTIFY WATER SOURCES PRIOR TO MANUAL ACTIVATION OF ADS	
11 A.1.1			- EMERGENCY PREPAREDNESS, SHORT TERM	

ISSUE NUMBER	MULTI-PLANT ACTION NO.	ISSUE TITLE	LICENSEE IMPLEMENTATION STATUS
III.A.1.2.1		UPGRADE EMERGENCY SUPPORT FACILITIES - INTERIM TSC OSC & EOF	
III.A.1.2.2		UPGRADE EMERGENCY SUPPORT FACILITIES-DESIGN-INCORP. INTO F063/F064/F065	
(SEE F063, F064, F065)			
III.A.1.2.3		UPGRADE EMER SUPPORT FACILITIES - MODS INCORPOR. INTO F063, F064 & F065	
(SEE F063, F064, F065)			
III.A.2.1	F067	UPGRADE PREPAREDNESS - UPGRADE EMERGENCY PLANS TO APP. E. 10 CFR 50.	
III.A.2.2	F068	UPGRADE PREPAREDNESS - METEOROLOGICAL DATA	
*III.D.1.1.1		PRIMARY COOLANT OUTSIDE CONTAINMENT - LEAK REDUCTION	
*III.D.1.1.2		PRIMARY COOLANT OUTSIDE CONTAINMENT - TECH SPECS	
III.D.3.3.1		INPLANT RAD. MONIT. - PROVIDE MEANS TO DETER. PRESENCE OF RADIOIODINE	
III.D.3.3.2	F069	INPLANT RADIATION MONIT. - MODIFICATIONS TO ACCURATELY MEAS. IODINE	
III.D.3.4.1	F070	CONTROL ROOM HABITABILITY - REVIEW	
*III.D.3.4.2	F070	CONTROL ROOM HABITABILITY - SCHEDULE MODIFICATIONS	
*III.D.3.4.3		CONTROL ROOM HABITABILITY - IMPLEMENT MODIFICATIONS	
MPA-F008		I.D.1.1 DETAILED CONTROL ROOM DESIGN REVIEW PROGRAM PLAN	
MPA-F063		III.A.1.2 TECHNICAL SUPPORT CENTER	
MPA-F064		III.A.1.2 OPERATIONAL SUPPORT CENTER	
MPA-F065		III.A.1.2 EMERGENCY OPERATIONS FACILITY	
MPA-F071		I.D.1.2 DETAILED CONTROL ROOM REVIEW (FOLLOWUP TO F-8)	
MPA-B072	B072	NUREG-0737 TECH SPECS (GENERIC LETTERS 82-16 & 83-02)	
MPA-B083	B083	TECH SPEC COVERED BY GENERIC LETTERS 83-26 & 83-37 FOR NUREG-0737	
67.4.1	G001	REACTOR COOLANT PUMP TRIP (GENERIC LETTER 85-12)	

April 14, 1989

- 2 -

Your prompt attention to this matter within the requested time frame is appreciated.

Sincerely,

/s/

Thomas E. Murley, Director
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

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