

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Shoreham Nuclear Power Station Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 2 2										PAGE (3) 1 OF 0 3			
TITLE (4) Results of LLRT of Penetration showed leakage that, when combined with all type B & C penetration leakages, exceeded the Tech Spec limit of 0.6 La																							
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)					
MONTH		DAY		YEAR		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER		MONTH		DAY		YEAR		FACILITY NAMES				DOCKET NUMBER(S)	
0 3		2 1		8 9		8 9		0 0 5		0 0		0 4		2 0		8 9						0 5 0 0 0	
																						0 5 0 0 0	
OPERATING MODE (9) 4				THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																			
POWER LEVEL (10) 0 0 0				20.402(b)						20.406(c)						30.73(a)(2)(iv)						73.71(b)	
				20.406(a)(1)(i)						30.36(e)(1)						30.73(a)(2)(v)						73.71(e)	
				20.406(a)(1)(ii)						30.36(e)(2)						30.73(a)(2)(vi)						OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
				20.406(a)(1)(iii)						30.73(a)(2)(ii)						30.73(a)(2)(viii)(A)							
				20.406(a)(1)(iv)						30.73(a)(2)(iii)						30.73(a)(2)(viii)(B)							
20.406(a)(1)(v)						30.73(a)(2)(iv)						30.73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12)																							
NAME R.W. Grunseich, Operational Compliance Engineer																TELEPHONE NUMBER AREA CODE 5 1 6 9 2 9 - 8 3 0 0							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NIPDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NIPDS														
B	B	O I S V	A 3 9 1	No	B	W K I S V	V 0 8 5	No															
B	B	O I S V	V 0 8 5	No	B	B	O I S V	V 0 8 5	No														
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)				MONTH		DAY		YEAR					
<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)														0 7		3 1		8 9					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On March 21, 1989 at 1430, it was determined by the Maintenance section that the results of a Local Leak Rate Test (LLRT) of valve E11*MOV-031A (Core Spray "A" Suction), showed leakage that, when combined with all "B" and "C" penetration leakages exceeded the Tech. Spec limit of 0.6 La. The plant was in Operational Condition 4 (Cold Shutdown) with the mode switch in Shutdown and all rods inserted in the core. Subsequent testing identified three additional valves which had excessive leakage. Maintenance Work Requests (MWRs) were generated to investigate the leakage and to repair the valves as needed. Three valves were repaired and retested with acceptable results. The fourth valve is scheduled to be repaired. A supplemental report will be generated upon completion of the LLRT and will identify all valves which significantly contributed to the combined leakage along with any corrective actions taken. Valves that consistently fail leak rate testing will be introduced into the repetitive maintenance review to determine if additional corrective actions are required.

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PDR ADOCK 05000322
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED O'B NO 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1) Shoreham Nuclear Power Station Unit -1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 9 -	0 0 5 -	0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor

Energy Industry Identification System (EIIS) codes are identified in the text as [xx].

IDENTIFICATION OF THE EVENT

Results of LLRT for check valve showed leakage that, when combined with all Type B and C penetration leakage, exceeded the Tech. Spec limit of 0.6 La.

Event Date: 3/21/89

Report Date: 4/20/89

CONDITIONS PRIOR TO THE EVENT

Operational Condition 4 (Cold Shutdown)

Mode Switch - Shutdown

RPV Pressure= 0 psig RPV Temperature = 105 Degrees F

POWER LEVEL - 0

All rods inserted in the core

DESCRIPTION OF THE EVENT

On March 21, 1989 at 1430, it was determined by the Maintenance section that the results of a Local Leak Rate Test (LLRT) of valve 1E11*MOV-031A (Core Spray "A" suction valve), showed leakage that, when combined with all "B" and "C" valve leakages exceeded the Tech. Spec. limit of 0.6 La. The penetration was tested to satisfy Tech. Spec. requirement 3.6.1.2.b. Upon identification of the excessive leakage, a Maintenance Work Request was generated to investigate the cause and subsequent repair of the valve. Subsequent testing identified three additional valves which had excessive leakage. They are:

1E11*MOV-040B [BO] (RHR Containment Spray Valve)

1G11*MOV-249 [WK] (Drywell Equipment Drains Outboard Isolation Valve)

1E11*MOV-040A [BO] (RHR Containment Spray Valve)

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		8 9	0 0 5	0 0			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

CAUSE OF THE EVENT

The cause of the leakage for the valves was due to normal valve degradation (excessive valve disc to seat leakage).

ANALYSIS OF THE EVENT

This event is reportable per 10CFR50.73(a)(2)(ii) as identified in NUREG 1022, page 12. There is minimal safety significance to this event. All four valves are set up in a double valve configuration and their adjacent valves leak tested satisfactorily, and would have provided a means for containment isolation upon an accident signal. Had a LOCA occurred during 5% power operation with the leakage rate above the allowable rate, the radiation doses would have been well below the limits as required by 10CFR100 and the General Design Criterion 19 of Appendix A of 10CFR50.

CORRECTIVE ACTIONS

1. MWRs were generated for the four valves. Three were repaired and retested with acceptable results. The fourth (1E11*MOV-040A) is scheduled to be repaired. The supplemental report will identify the action taken to repair the valve along with corrective actions taken for any additional valves which significantly contributed to the combined leakage.
2. Valves that consistently fail leak rate testing will be introduced into the repetitive maintenance review to determine if additional corrective actions are required.

ADDITIONAL INFORMATIONa. Manufacturer and model number of failed component (s)

<u>Valve No.</u>	<u>Manufacturer</u>	<u>Model No.</u>
1E11*MOV-031A	Anchor Darling Co.	E5836-7-1
1E11*MOV-040A	Velan Valve Co.	B20-01064B-02WN
1E11*MOV-040B	Velan Valve Co.	B20-01064B-0210N
1G11*MOV-249	Velan Valve Co.	10-0054B-02WN

b. LER numbers of previous similar events

85-049
87-029



LONG ISLAND LIGHTING COMPANY

SHOREHAM NUCLEAR POWER STATION

P.O. BOX 618, NORTH COUNTRY ROAD • WADING RIVER, N.Y. 11792

Direct Dial Number:

April 20, 1989

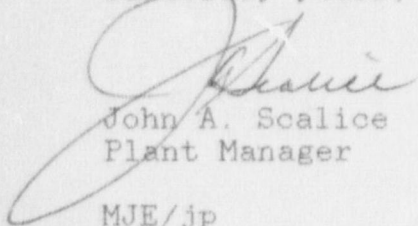
PM-89-076

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

In accordance with 10CFR50.73, enclosed is Shoreham Nuclear Power Station's Licensee Event Report (LER 89-005).

Sincerely yours,



John A. Scalice
Plant Manager

MJE/jp

Enclosure

cc: William T. Russell, Regional Administrator
Frank Crescenzo, Senior Resident Inspector
Institute of Nuclear Power Operations, Records Center
American Nuclear Insurers

SR.A21.0200

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