NRC Form (9-83)	366				ten date van vennamet Afree	L	CEN	ISEI	E EVE	NT RE	PORT	(LER)	U.S. NI	APP	AR REGULATO PROVED OMB TRES: 8/31/88	ORY COM NO 3150-0	MISSION
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VALV	E CI	OSU	RE L	EADS	S TO	NON-C	OMP	LIA	ANCE	WITH	TECH	HNICAL S	PECIFIC	AT	IONS		
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NAME													AREA CODE	TEL	LEPHONE NUM	BER	
R.	M. (NODC	1. NU	CLE	AR SA	FETY	AND	CC	OMPL	ANCE			4,0,4	8	3, 2, 6, .	3, 3	2, 0,
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CAUSE	SYSTEM	COM	PONENT	MA T	NUFAC URER	REPORTAL TO NPRO	LES			CAUSE	SYSTEM	COMPONENT	MANUFAC TURER	5	REPORTABLE TO NPRDS	*******	
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Te Co On Sy pr of au si Re ma a fi wh 2 id	chni ld L 3-l ject stem oced thi toma gnal sidu nipu shif llin ile accu	cal eg I 9-89 ion (RC lure s va tic Cal H lati t su g SI in M mula fied	Speci nject (SI) (S) co to fi lve p injec n 3-2 leat F ons) pervi accu lode 3 tor f	2033 pump old 1 ill S prevection 26-89 Remov and isor umula 3. U fills Unit	tion 3 valve, CST, cold egs (2 I accu nts bo to th , whil al ren simili realiz tors a pon di was i l whe	/4.5.2 HV-88 the sh leg ir -HV-88 mulato th SI e RCS e cons dered ar sit ed tha t low scover nitiat n 1-HV	re ift jec 35) rs pum col ide ino uat t t RCS ing ed, -88	qui be opp tio wh at ps d l rin per th N 335	res t open erati n val ile p low R from egs u g LER s for syste essur is, a ine s was c	hat t while ng cru ve to erfor CS pro being pon r 2-89 due to othe m ope e requ revio epara	he Saf in Mc ew clo the R ning t essure capab eccipt -003 (o comm r safe rating uired ew of te ins while	fety Inject odes 1, 2 osed the S Reactor Co the system of a SI both train on valve ety-relate procedur closure of the Unit stances we in Mode	tion (SI and 3. afety oolant operati 3. Clos oviding actuatio ns of d system re for of 2-HV-8 1 and Un ere 3, in) ng ur n s, 83 it	e 5		
Th	e ca	use	of th	nese	events	is ir	ade	qua	te pr	ocedu	res wh	nich did r	isty. Not preve	nt			
Mo	osur de 3 adec	re of ent juaci	HV-8 ry. es.	8835 The	during proced	Mode ures a	3 o ire	r r bei	equir ng ch	e acci anged	umulat to co	tor fill p prrect the	ese ese				
			890 PDR	4260 AD	072 8 0CK 0	90421 50004	25			-		n dala kalam mandan dalam baharan dala baharan yana	al Aquadre va MAT sa Crantas que sonante parante d	00000000	12	1	Construction of the local division of the lo

48C Form 9-83)	LICENSEE EVENT REP	ORT (LER) TEXT CONTINU	JATION	APPROVED C EXPIRES: 8/31	GULATORY COMMISSIO DMB NO. 3150-0104 1/88			
ACILITY	NAME (1)	DOCKET NUMBER (2)	LER NUMBER () PAGE (3)				
	NT VOOTLE UNIT O		YEAR SEQUENTIA	REVISION	-			
PLA	INT VOGILE - UNIT 2	0 15 10 10 10 14 12 5	8 9 - 0 1 1	- 010	0 2 OF 0 1			
EXT (II m	ore space is required, use additional NRC Form 386A's/ (17)	annan 1997 an an an Anna an Ann	al and a surface of the second standards					
Α.	REQUIREMENT FOR REPORT							
	This report is required per 10 operated in Mode 3 with Safety valve HV-8835 closed, a condit (TS) 3/4.5.2 requirements.	OCFR50.73(a)(2)(i) be / Injection Pump Cold tion outside of Techn	cause the unit Leg Injectior ical Specifica	t ition				
В.	UNIT STATUS AT TIME OF EVENT							
	At the time of this event on 3 (Hot Standby) at 0% rated ther (RCS) pressure and temperature degrees F. The operating crew plateau of 950 psig and 450 de inoperable equipment which cor event.	8-19-89, Unit 2 was o mal power. Reactor was at approximatel had initiated a RCS grees F. Other than atributed to the occu	perating in Mc Coolant System y 500 psig and heatup to a ECCS, there w rrence of this	ode 3 1 1 375 vas no				
С.	DESCRIPTION OF EVENT							
	Technical Specification 3/4.5. require that valve HV-8835, th Injection valve, be open with the lock out position while in	2 and surveillance m me Safety Injection (its associated power Modes 1, 2, and 3.	equirement 4.5 SI) Pump Cold lock out swit	.2a. Leg ch in				
	On 3-19-89, t 2033 CST, while pressure at 500 psig. Work wa restraint items. The shift cr heatup to a plateau of 950 psi operator was instructed to fil accordance with the system ope procedure included a note rega requirements. However, the no gave direction as to when to p fill, rather than when not to 3-19-89, the operating crew cl legs in accordance with proced for fill of SI accumulators at 3-19-89, the SI accumulator fi	in Mode 3 while Real s in progress to clear rew was directed to i g and 450 degrees F. 1 and pressurize the erating procedures. The ording SI system oper- operform the low RCS p perform the fill. osed valve 2-HV-8835 ure 13105-2, "Safety 1 was completed and wor lock out switch	ctor Coolant S ar existing mo nitiage a RCS The reactor SI accumulato he system oper ability h a way that i ressure accumu At 2033 CST, o to the RCS co Injection Sys At 2115 CST, o 2-HV-8835 was	ystem de rs in sating t lator n ld tem",				

out position.

NRC Form 366A (9-83)	LICENSEE EVI	U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/86						
FACILITY NAM	£ (1)	DOCKET NUMBER (2)	LER NUM	BER (6)		PAGE (3)		
			YEAR SEQUE	ABER REVISIO	2 1			
PLAN	T VOGTLE - UNIT 2	0 15 10 10 10 14 12 15	819 - 01	1 1 -0 0	0 13	OF () .	
Or He ar su pr wh th pr cc Up Ur Wr or	n 3-26-89, while consider al Removal rendered in ad similar situations pervisor realized that rocedures required close and filling SI accumula- nat the accumulators we ressure was in excess of cisted for HV-8835 to H omply with TS 3/4.5.2 bon discovering this, a nit 2 accumulator fills here 1-HV-8835 was close of the following dates:	dering LER 2-89-003 (both tr noperable due to common valv for other safety-related sys t both the Unit 1 and Unit 2 sing the SI pump cold leg in ators at low RCS pressure. ere not required to be opera of 1000 psig in Mode 3, and be closed while in Mode 3, t a review was initiated for p s. Nine separate instances sed while in Mode 3. These	rains of Res re manipulat stems, a shi system ope ojection val He also rea ble until F the potenti the potenti thus failing orior Unit 1 were identi instances o	sidual tions) ift erating lve alized RCS ial g to l and ified occurred				
3- 5- 10	-18-87 20-87 (2 times))-29-87	2-8-88 11-23-88 (3 times) 1-26-89						
Tł	e instance previously	described was the only occu	rrence for	Unit 2.				
D. CA	USE OF EVENTS							
Tł	ne cause of these event	ts was inadequate procedures	. Procedur	res				

13105-1 and 13105-2, "Safety Injection System", provide instruction for filling accumulators but restrictions for filling while in Mode 3 were confusing and inadequate. Also, procedure 12002-C, "Unit Heatup To Normal Operating Temperature and Pressure (Mode 4 To Mode 3)" does not require accumulator fill to be completed prior to Mode 3 entry.

E. ANALYSIS OF EVENTS

While valve HV-8835 valve is closed, the safety injection pumps are unable to automacically provide flow into the reactor vessel and manual action would have been necessary to provide flow. However, the centrifugal charging pumps and residual heat removal pumps were available to automatically provide flow into the reactor vessel. The nine Unit 1 events represent a total of 5 hours, 51 minutes, that 1-HV-8835 was closed during Mode 3. The safety risk imposed by this condition is considered to be small due to the relatively short period of time involved. The 42 minutes that 2HV-8835 was closed has no safety significance because Unit 2 had not yet achieved initial criticality and there was no radioactive decay heat in the reactor core. Finally, no event occurred in either unit which required actuation of safety injection during the time periods involved. Based on these considerations, there was no adverse affect to plant safety or to public health and safety as a result of these events.

NixC Form (9-81)	366A	LICENSEE EVENT R	EPORT (LER) TEXT CONTINU	JATION	APPROVED O EXPIRES: 8/3	GULATOF DMB NO. 1 1/88	3150-010	11551ON 14
ACILITY	NAME (1)	None and an operation of the same members and before a state of the second state of the second of the second st	DOCKET NUMBER (2)	LER NUMBER	(6)	1	PAGE (3)	
				YEAR SEQUENTI, NUMBER	AL REVISION NUMBER	-		
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DCT (II ma	ve space is	rneguit xd, use additional NRC Form 386A's) (17)						
F.	COR	RECTIVE ACTIONS						
	1.	The inadequate restrict were changed to precise operators not to perfor process while in Mode 3 reviewed which would no RCS pressure. This rev	tions in procedures 1310 e caution statements whi rm the low RCS pressure 3. An alternative fill ot require closure of HV view will be completed b	05-1 and 1310 ich direct accumulator process is b V-8835 while by May 21, 19	5-2 fill eing at low 89.			
	2.	Procedure 12002-C was m that SI accumulator fil	revised to include a sig Il is completed prior to	gn off ensuri o Mode 3 entr	ng y.			
	3.	A copy of this LER will Reading Book.	l be placed in the Opera	ations Requir	ed			
	4.	A copy of this LER and the Training department Operator Requalificatio	all reference documents t for evaluation of incl on Program.	s will be sen lusion into t	t to he			
G.	ADD	ITIONAL INFORMATION						
	1.	Failed Components None						
	2.	Previous Similar Events None	5					
	3.	Energy Identification S Safety Injection - BQ	System Code					

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Georgia Power Company 333 Piedmont Avenue Atlanta, Georgia 30308 Jelephone 404 526 3195

Mailing Address. 40 Inverness Center Parkway Post Office Box 1295 Birmingham, Alabs the 35201 Telephone 205 888-5581

W. G. Hairston, III Senior Vice President Nuclear Operations ELV-00451 1408n

April 21, 1989

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D. C. 20555

PLANT VOGTLE - UNIT 2 NRC DOCKET 50-425 OPERATING LICENSE NPF-81 LICENSEE EVENT REPORT VALVE CLOSURE LEADS TO NON-COMPLIANCE WITH TECHNICAL SPECIFICATIONS

Gentlemen:

In accordance with 10 CFR 50.73, Georgia Power Compnay hereby submits the enclosed report related to an event which was discovered on March 26, 1989.

Sincerely,

W. S. Hours Th

W. G. Hairston, III

TEW/PAH/gm

Enclosure: LER 50-425/1989-011

xc: Georgia Power Company Mr. P. D. Rice Mr. C. K. McCoy Mr. G. Bockhold, Jr Mr. M. Sheibani Mr. J. P. Kane NORMS

> U. S. Nuclear Regulatory Commission Mr. S. Ebneter, Regional Administrator Mr. J. B. Hopkins, Lirensing Project Manager, NRR Mr. J. F. Rogge, Senior Resident Inspector, Vogtle

1222