



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY

DOCKET NO. 50-352

LIMERICK GENERATING STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 18
License No. NPF-39

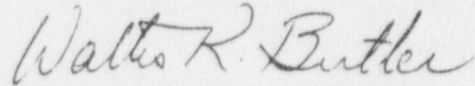
1. The Nuclear Regulatory Commission (the Commission) has found that
 - A. The application for amendment by Philadelphia Electric Company (the licensee) dated March 23, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-39 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 18, are hereby incorporated into this license. Philadelphia Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 14, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 18

FACILITY OPERATING LICENSE NO. NPF-39

DOCKET NO. 50-352

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. Overleaf page is provided to maintain document completeness.*

Remove

3/4 3-37*
3/4 3-38

Insert

3/4 3-37*
3/4 3-38

TABLE 3.3.3-2

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
1. CORE SPRAY SYSTEM		
a. Reactor Vessel Water Level - Low Low Low, Level 1	> -129 inches*	> -136 inches
b. Drywell Pressure - High	< 1.68 psig	< 1.88 psig
c. Reactor Vessel Pressure - Low	> 455 psig, (decreasing)	> 435 psig, (decreasing)
d. Manual Initiation	N.A.	N.A.
2. LOW PRESSURE COOLANT INJECTION MODE OF RHR SYSTEM		
a. Reactor Vessel Water Level - Low Low Low, Level 1	> -129 inches*	> -136 inches
b. Drywell Pressure - High	< 1.68 psig	< 1.88 psig
c. Reactor Vessel Pressure - Low	> 455 psig, (decreasing)	> 435 psig, (decreasing)
d. Injection Valve Differential Pressure - Low	> 74 psid, (decreasing)	> 64 psid and < 84 psid
e. Manual Initiation	N.A.	N.A.
3. HIGH PRESSURE COOLANT INJECTION SYSTEM		
a. Reactor Vessel Water Level - (Low Low, Level 2)	> -38 inches*	> -45 inches
b. Drywell Pressure - High	< 1.68 psig	< 1.88 psig
c. Condensate Storage Tank Level - Low	> 167.8 inches**	> 164.3 inches
d. Suppression Pool Water Level - High	< 24 feet 1.5 inches	< 24 feet 3 inches
e. Reactor Vessel Water Level - High, Level 8	< 54 inches	< 60 inches
f. Manual Initiation	N.A.	N.A.
4. AUTOMATIC DEPRESSURIZATION SYSTEM		
a. Reactor Water Level - Low Low Low, Level 1	> -129 inches*	> -136 inches
b. Drywell Pressure - High	< 1.68 psig	< 1.88 psig
c. ADS Timer	< 105 seconds	< 117 seconds
d. Core Spray Pump Discharge Pressure - High	> 145 psig, (increasing)	> 125 psig, (increasing)
e. RHR LPCI Mode Pump Discharge Pressure-High	> 125 psig, (increasing)	> 115 psig, (increasing)
f. Reactor Vessel Water Level-Low, Level 3	> 12.5 inches	> 11.0 inches
g. Manual Initiation	N.A.	N.A.
h. ADS Drywell Pressure Bypass Timer	< 420 seconds	< 450 seconds

*See Bases Figure B 3/4.3-1.

**Corresponds to 2.25 feet indicated.

TABLE 3.3.3-2 (Continued)
EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	RELAY	TRIP SETPOINT	ALLOWABLE VALUE
5. LOSS OF POWER			
a. 4.16 kV Emergency Bus Undervoltage (Loss of Voltage)	127-11X	NA	NA
b. 4.16 kV Emergency Bus Undervoltage (Degraded Voltage)	RELAY 127-11X0X and 102-11X0X	a. 4.16 kV Basis 2905 $\pm$ 115 volts b. 120 V Basis 83 $\pm$ 3 volts c. < 1 second time delay	2905 $\pm$ 145 volts 83 $\pm$ 4 volts < 1.5 second time delay
	127Y-11X0X** and 127Y-1-11X0X	a. 4.16 kV Basis 3640 $\pm$ 91 volts b. 120 V Basis 104 $\pm$ 3 volts c. < 52 second time delay	3640 $\pm$ 182 volts 104 $\pm$ 5.2 volts < 60 second time delay
	127Z-11X0X and 162Y-11X0X	a. 4.16 kV Basis 3910 $\pm$ 11 volts b. 120 V Basis 111.7 $\pm$ 0.3 volts c. < 10 second time delay	3910 $\pm$ 19 volts 111.7 $\pm$ 0.5 volts < 11 second time delay
	127Z-11X0X and 162Z-11X0X	a. 4.16 kV Basis 3910 $\pm$ 11 volts b. 120 V Basis 111.7 $\pm$ 0.3 volts c. < 61 second time delay	3910 $\pm$ 19 volts 111.7 $\pm$ 0.5 volts < 64 second time delay

**This is an inverse time delay voltage relay. The voltages shown are the maximum that will not result in a trip. Some voltage conditions will result in decreased trip times.