



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 14, 1989

Docket Nos. 50-277/278

Mr. George A. Hunger, Jr.
Director-Licensing
Philadelphia Electric Company
Correspondence Control Desk
P. O. Box 7520
Philadelphia, Pennsylvania 19101

Dear Mr. Hunger:

RE: PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3

The purpose of this letter is to request your prompt review and reporting of the status of implementation of TMI Action Plan Items at your facility. As an enclosure we are providing a printout of all TMI items annotated where our tracking system indicates that implementation is not complete. For the purpose of this request, implementation should be considered complete when, to the best of your knowledge, all actions necessary to satisfactorily meet the requirements of NUREG-0737 and NUREG-0737 Supplement 1 have been completed.

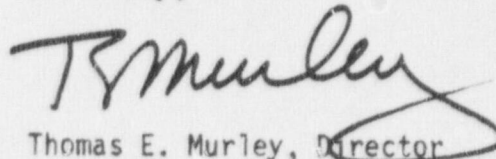
Where you have not fully completed an item as described above, we request that you mark-up the enclosure to reflect your projected implementation date. Add a short note identifying remaining work (e.g., hardware, procedures, training, technical specifications). More explicit instructions are provided as part of the enclosure. We request that this information reach this office by April 18, 1989.

The information we are requesting will be utilized to validate our existing data base so that we can accurately respond to congressional inquiries concerning the status of implementation of TMI Action Plan Items. In view of the importance of the information we request your personal attention and certification that the information is correct to the best of your knowledge.

This request is covered by Office of Management and Budget Clearance Number 3150-0011, which expires December 31, 1989. The estimated average burden hours is 40 person hours per owner response, including searching data sources, gathering and analyzing the data, and preparing the required letter. Send comments regarding this burden estimate or any other aspect of this collection of information, including suggestions for reducing this burden, to the Records and Reports Management Branch, Division of Information Support Services, Office of Information Resources Management, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555; and to the Paperwork Reduction Project (3150-C011), Office of Management and Budget, Washington, D.C. 20503.

Your prompt attention to this matter within the requested time frame is appreciated.

Sincerely,

A handwritten signature in dark ink, appearing to read "T. E. Murley", with a large, sweeping flourish extending from the end of the name.

Thomas E. Murley, Director
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

Mr. George A. Hunger, Jr.
Philadelphia Electric Company

Peach Bottom Atomic Power Station,
Units 2 and 3

cc:

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ENCLOSURE
PEACH BOTTOM
Units 2 + 3

GUIDELINES FOR UPDATING TMI STATUS

1. The enclosure provides a complete listing of all TMI requirements from NUREG-0737 and NUREG-0737 Supplement 1. Regulatory Guide 1.97 requirements, although addressed in Supplement 1 to NUREG-0737, are excluded since they did not originate from the TMI Action Plan. This listing consists of a matrix of items applicable to both operating reactors and operating license applicants (NTOLS) as of November, 1980, as shown in NUREG-0737 Enclosure 1 and 2, respectively. All items which are not listed identically in Enclosure 1 and 2 are marked with asterisks to indicate differences between operating reactor and NTOL requirements. Whenever these differences occur, enter data in accordance with the appropriate title, regardless of the TMI Action Plan item number listed. Your NRC Project Manager has indicated which issues are shown as unimplemented in the NRC tracking system. Please review the entire listing for each licensed reactor unit. Where an item was not applicable for your facility, mark N/A in the implementation status column.
2. Where an item was applicable to your facility but no changes were necessary, mark NC in the implementation status column.
3. Where an item was applicable to your facility and changes are complete mark C in the implementation status column. If the date is readily available (month and year) we request you provide it, however, C is sufficient.
4. Where an item was applicable to your facility and is not fully implemented, provide your projected implementation date (month and year) and a short note identifying the outstanding item (e.g. hardware, procedures, training, technical specifications).

PEACH BOTTOM Atomic Power Station
Unit: 2 and 3

ISSUE NUMBER	MULTI-PLANT ACTION NO.	ISSUE TITLE	LICENSEE IMPLEMENTATION STATUS
1 A 1.1.1		SHIFT TECHNICAL ADVISOR - ON DUTY	
1 A 1.1.2		SHIFT TECHNICAL ADVISOR - TECH SPECS.	
1 A 1.1.3	F001	SHIFT TECHNICAL ADVISOR - TRAINED PER LL CAT B	
1 A 1.1.4		SHIFT TECHNICAL ADVISOR - DESCRIBE LONG TERM PROGRAM	
1 A 1.2		SHIFT SUPERVISOR RESPONSIBILITIES	
1 A 1.3.1	F002	SHIFT MANNING - LIMIT OVERTIMES	
1 A 1.3.2	F002	SHIFT MANNING - MIN SHIFT CREW	
1 A 2.1.1		IMMEDIATE UPGRADING OF RO & SRO TRAINING AND QUAL. - SRO EXPR.	
1 A 2.1.2		IMMEDIATE UPGRADING OF RO & SRO TRAINING AND QUAL. - SRO'S BE RO'S 1YR.	
1 A 2.1.3		IMMEDIATE UPGRADING OF RO & SRO TRAINING AND QUAL. - 3 MO. TRAINING	
1 A 2.1.4	F003	IMMEDIATE UPGRADING OF RO & SRO TRAINING AND QUAL. - MODIFY TRAINING	
1 A 2.1.5		IMMEDIATE UPGRADING OF RO & SRO TRAINING AND QUAL. - FACILITY CERTIF.	
1 A 2.3		ADMINISTRATION OF TRAINING PROGRAMS	
1 A 3.1.1		REVISE SCOPE & CRITERIA FOR LICENSING EXAMS - INCREASE SCOPE	
1 A 3.1.2		REVISE SCOPE & CRITERIA FOR LICENSING EXAMS - INCREASE PASSING GRADE	
1 A 3.1.3.A		REVISE SCOPE & CRIT. FOR LIC. EXAMS - SIMULATOR PLANTS WITH SIMULATORS	
1 A 3.1.3.B		REVISE SCOPE & CRIT. FOR LIC. EXAMS - SIMULATOR - OTHER PLANTS	
1 B 1.2		EVALUATION OF ORGANIZATION & MANAGEMENT	
1 C 1.1		SHORT-TERM ACCIDENT & PROCEDURES REVIEW - SB LOCA	
1 C 1.2.A	F004	SHORT-TERM ACCID. & PROCEDURES REV. - INADEQ. CORE COOL. REANAL. GUIDELINES	
1 C 1.2.B	F004	SHORT-TERM ACCID. & PROCEDURES REV. - INADEQ. CORE COOL. REANAL. GUIDELINES	
1 C 1.3.A	F005	SHORT-TERM ACCID. & PROCEDURES REV. - TRANSIENTS & ACCORDS. REANAL GUIDELINES (PROC. GEN. PKG.)	
1 C 1.3.B	F005	SHORT-TERM ACCID. & PROCEDURES REV. - TRANSIENTS & ACCORDS. REVISE PROCEDURES (UPGRADED EOP'S)	
1 C 2		SHIFT & RELIEF TURNOVER PROCEDURES	
1 C 3		SHIFT-SUPERVISOR RESPONSIBILITY	
1 C 4		CONTROL-ROOM ACCESS	
1 C 5	F006	FEEDBACK OF OPERATING EXPERIENCE	
1 C 6	F007	VERIFY CORRECT PERFORMANCE OF OPERATING ACTIVITIES	
1 C 7.1		NSSS VENDOR REV. OF PROC - LOW POWER TEST PROGRAM	
1 C 7.2		NSSS VENDOR REV. OF PROC - POWER ASCENSION & EMER. PROCS.	
1 C 8		PILLOT MON OF SELECTED EMERGENCY PROC FOR NTOLS	
1 D 1 (See F008 & F071)		CONTROL-ROOM DESIGN REVIEWS (ENTER DATA FOR MPA F008 & MPA F-071)	
1 D 2.1	F009	** PLANT-SAFETY PARAMETER DISPLAY CONSOLE - DESCRIPTION	

** Shown as unimplemented in
NRC training system

ISSUE NUMBER MULTI-PLANT ACTION NO. ISSUE TITLE LICENSEE IMPLEMENTATION STATUS

I.D.2.2	F009	* * PLANT-SAFETY PARAMETER DISPLAY CONSOLE - INSTALLED.	
I.D.2.3	F009	* * PLANT-SAFETY PARAMETER DISPLAY CONSOLE - FULLY IMPLEMENTED.	
*I.G.1.1		TRAINING DURING LOW-POWER TESTING - PROPOSE TESTS.	
*I.G.1.2		TRAINING DURING LOW-POWER TESTING - SUBMIT ANAL. & PROCS.	
*I.G.1.3		TRAINING DURING LOW-POWER TESTING - TRAINING & RESULTS.	
II.B.1.1		REACTOR-COOLANT SYSTEM VENTS - DESIGN VENTS.	
II.B.1.2	F010	REACTOR-COOLANT SYSTEM VENTS - INSTALL VENTS (LL CAT B)	
II.B.1.3	F010	REACTOR-COOLANT SYSTEM VENTS - PROCEDURES.	
II.B.2.1		PLANT SHIELDING - REVIEW DESIGNS.	
*II.B.2.2		PLANT SHIELDING - CORRECTIVE ACTIONS TO ASSURE ACCESS.	
*II.B.2.3		PLANT SHIELDING - PLANT MODIFICATIONS (LL CAT B).	
*II.B.2.4	F011	PLANT SHIELDING - EQUIPMENT QUALIFICATION- NOT TRACKED AS A TMI ACTION ITEM.	
*II.B.3.1		POSTACCIDENT SAMPLING - INTERIM SYSTEM.	
*II.B.3.2		POSTACCIDENT SAMPLING - CORRECTIVE ACTIONS.	
*II.B.3.3		POSTACCIDENT SAMPLING - PROCEDURES.	
*II.B.3.4	F012	POSTACCIDENT SAMPLING - PLANT MODIFICATIONS (LL CAT B).	
II.B.4.1	F013	TRAINING FOR MITIGATING CORE DAMAGE - DEVELOP TRAINING PROGRAM.	
*II.B.4.2.A	F013	TRAINING FOR MITIGATING CORE DAMAGE - INITIAL.	
*II.B.4.2.B	F013	TRAINING FOR MITIGATING CORE DAMAGE - COMPLETE.	
II.D.1.1		RELIEF & SAFETY VALVE TEST REQUIREMENTS - SUBMIT PROGRAM.	
*II.D.1.2.A		RELIEF & SAFETY VALVE TEST REQUIREMENTS - COMPLETE TESTING.	
*II.D.1.2.B	F014	RELIEF & SAFETY VALVE TEST REQUIREMENTS - PLANT SPECIFIC REPORT.	
II.D.1.3		RELIEF & SAFETY VALVE TEST REQUIREMENTS - BLOCK-VALVE TESTING.	
*II.D.3.1		VALVE POSITION INDICATION - INSTALL DIRECT INDICATIONS OF VALVE POS.	
*II.D.3.2		VALVE POSITION INDICATION - TECH SPECS.	
*II.E.1.1.1	F015	AFS EVALUATION-ANALYSIS.	
(SEE NOTE 1)			
*II.E.1.1.2	F015	AFS EVALUATION-SHORT TERM MODS.	
(SEE NOTE 2)			
II.E.1.1.3	F015	AFS -LONG TERM MODS.	
(SEE NOTE 3)			

NOTE 1 - THE ITEM LISTED IS FROM NUREG-0737, ENCLOSURE 2 AND IS APPLICABLE TO NTOL'S ONLY

NOTE 2 - THE ITEM LISTED IS FOR ALL PLANTS (OPERATING REACTORS AND NTOL'S)

NOTE 3 - THE ITEM LISTED IS FOR ALL PLANTS (OPERATING REACTORS AND NTOL'S)

ISSUE NUMBER MULTI-PLANT ACTION NO. ISSUE TITLE LICENSEE IMPLEMENTATION STATUS

II.E.1.2.1.A		AFS INITIATION & FLOW-CONTROL GRADE.	
II.E.1.2.1.B	F016	AFS INITIATION & FLOW - SAFETY GRADE.	
II.E.1.2.2.A		AFS INITIATION & FLOW - FLOW INDICATION CONTROL GRADE.	
*II.E.1.2.2.B		AFS INITIATION & FLOW - LL CAT A TECH SPECS.	
*II.E.1.2.2.C	F017	AFS INITIATION & FLOW - SAFETY GRADE.	
*II.E.3.1.1		EMERGENCY POWER FOR PRESSURIZER HEATERS - UPGRADE POWER SUPPLY.	
*II.E.3.1.2		EMERGENCY POWER FOR PRESSURIZER HEATERS - TECH SPECS.	
II.E.4.1.1		DEDICATED HYDROGEN PENETRATIONS - DESIGN.	
*II.E.4.1.2		DEDICATED HYDROGEN PENETRATIONS - REVIEW & REVISE H2 CONTROL PROC.	
*II.E.4.1.3	F018	DEDICATED HYDROGEN PENETRATION - INSTALL.	
II.E.4.2.1-4		CONTAINMENT ISOLATION DEPENDABILITY - IMP. DIVERSE ISOLATION.	
*II.E.4.2.5.A		CONTAINMENT ISOLAT. DEPENDABILITY - CNTMT PRESS. SECT. SPECIFY PRESS.	
*II.E.4.2.5.B		CONTAINMENT ISOLATION DEPENDABILITY - CNTMT PRESSURE SEPT. MODS.	
II.E.4.2.6	F019	CONTAINMENT ISOLATION DEPENDABILITY - CNTMT PURGE VALVES.	
II.E.4.2.7	F019	CONTAINMENT ISOLATION DEPENDABILITY - RADIATION SIGNAL ON PURGE VALVES.	
*II.E.4.2.8		CONTAINMENT ISOLATION DEPENDABILITY - TECH SPECS.	
*II.F.1.1	F020	ACCIDENT - MONITORING - PROCEDURES.	
*II.F.1.2.A	F020	ACCIDENT - MONITORING - MOBILE GAS MONITOR.	
*II.F.1.2.B	F021	ACCIDENT - MONITORING - IODINE/PARTICULATE SAMPLING.	
*II.F.1.2.C	F022	ACCIDENT - MONITORING - CONTAINMENT HIGH-RANGE MONITOR.	
*II.F.1.2.D	F023	ACCIDENT - MONITORING - CONTAINMENT PRESSURE.	
*II.F.1.2.E	F024	ACCIDENT - MONITORING - CONTAINMENT WATER LEVEL.	
*II.F.1.2.F	F025	ACCIDENT - MONITORING - CONTAINMENT HYDROGEN.	
*II.F.2.1		INSTRUMENTATION FOR DETECT. OF INADEQUATE CORE COOLING - PROCEDURES.	
*II.F.2.2		INSTRUMENTATION FOR DETECT. OF INADEQUATE CORE COOL'G - SUBCOOL METER.	
*II.F.2.3	F026	INSTRUMENTATION FOR DETECT. OF INADEQUATE CORE COOLING - DESC. OTHER.	
*II.F.2.4	F026	INSTRUMENTATION FOR DETECT. OF INADEQUATE CORE CLING INSTLL ADD'L INSTRUMENTATION.	
*II.G.1.1		POWER SUPP. FOR PRESSURIZER RELIEF, BLOCK VALVES & LEVEL IND. - UPGRADE.	
*II.G.1.2		POWER SUPP. FOR PRESSURIZER RELIEF, BLOCK VALVES & LEVEL IND. - TECH SP.	
II.K.1 (Oper. Reactors Only)		IE BULLETINS - 79-05, 79-06, 79-08.	

ISSUE NUMBER	MULTI-PLANT ACTION NO.	ISSUE TITLE	LICENSEE IMPLEMENTATION STATUS
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*11.K.1.5		*IE BULLETINS - REVIEW ESF VALVES.	
*11.K.1.10		IE BULLETINS - OPERABILITY STATUS.	
*11.K.1.20		IE BULLETINS - PROMPT MANUAL REACTOR TRIP.	
*11.K.1.21		IE BULLETINS - AUTO SG ANTICIPATORY REACTOR TRIP.	
*11.K.1.22		IE BULLETINS - AUX. HEAT REM SYSTEM, PROC.	
*11.K.1.23		IE BULLETINS - RV LEVEL, PROCEDURES.	
*11.K.2.2		ORDERS ON B&W PLANTS - PROCEDURES TO CONTROL AFW IND OF ICS.	
*11.K.2.8		ORDERS ON B&W PLANTS - UPGRADE AFW SYSTEM.	
11.K.2.9	F027	ORDERS ON B&W PLANTS - FEMA ON ICS.	
11.K.2.10	F028	ORDERS ON B&W PLANTS - SAFETY-GRADE TRIP.	
*11.K.2.11	F029	ORDERS ON B&W PLANTS - OPERATOR TRAINING.	
11.K.2.13	F030	ORDERS ON B&W PLANTS - THERMAL MECHANICAL REPORT (CE & W PLANTS ALSO).	
11.K.2.14	F031	ORDERS ON B&W PLANTS - LIFT FREQUENCY OF PORV'S & SV'S.	
11.K.2.15		ORDERS ON B&W PLANTS - EFFECTS OF SLUG FLOW.	
11.K.2.16	F032	ORDERS ON B&W PLANTS - RCP SEAL DAMAGE.	
11.K.2.17	F033	ORDERS ON B&W PLANTS - VOIDING IN RCS (CE & W PLANTS ALSO).	
11.K.2.19		BENCHMARK ANALYSIS OF SEQUENTIAL AFW FLOW TO ONCE THROUGH STM GENERATOR.	
*11.K.2.20	F035	ORDERS ON B&W PLANTS - SYSTEM RESPONSE TO SB LOCA.	
*11.K.3.1.A	F036	B&O TASK FORCE - AUTOMATIC PORV ISOLATION DESIGN.	
*11.K.3.1.B		FINAL RECOMMENDATIONS, B&O TASK FORCE - P2 TO PORV ISO TEST/INSTALL.	
11.K.3.2	F037	B&O TASK FORCE - REPORT ON PORV FAILURES.	
11.K.3.3	F038	B&O TASK FORCE - REPORTING SV & RV FAILURES AND CHALLENGES.	
11.K.3.5.A	F039	B&O TASK FORCE - AUTO TRIP OF RCP'S PROPOSED MODIFICATIONS.	
11.K.3.5.B	F039	B&O TASK FORCE - AUTO TRIP OF RCP'S MODIFICATIONS.	
11.K.3.7		B&O TASK FORCE - EVALUATION OF PORV OPENING PROBABILITIES.	
11.K.3.9	F040	B&O TASK FORCE - PID CONTROLLER MODIFICATION.	
11.K.3.10	F041	B&O TASK FORCE - PROPOSED ANTICIPATORY TRIP MODIFICATIONS.	
11.K.3.11		B&O TASK FORCE - JUSTIFY USE OF CERTAIN PORV.	
11.K.3.12.A		B&O TASK FORCE - ANTICIPATORY TRIP ON TURBINE TRIP PROPOSED MODS.	
11.K.3.12.B	F042	B&O TASK FORCE - ANTICIPATORY TRIP ON TURBINE TRIP INSTALL MODS.	
11.K.3.13.A	F043	B&O TASK FORCE - HPCI & RCIC SYSTEM INITIATION LEVELS ANALYSIS.	
11.K.3.13.B	F043	B&O TASK FORCE - HPCI & RCIC INITIATION LEVELS MODIFICATION.	

ISSUE NUMBER	MULTI-PLANT ACTION NO.	ISSUE TITLE	LICENSEE IMPLEMENTATION STATUS
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*11 K.3.14	F044	B&O TASK FORCE - ISO CONDENSER ISOLATION ON HIGH RAD.	
11 K.3.15	F045	B&O TASK FORCE - MODIFY HPCI & RCIC BRK DETECTION CIRCUITRY	
11 K.3.16A	F046	B&O TASK FORCE - CHALLENGE & FAILURE OF RELIEF VALVE'S STUDY	
11 K.3.16.B	F046	B&O TASK FORCE - CHALLENGE & FAILURE OF RELIEF VALVE'S STUDY	
11 K.3.17	F047	B&O TASK FORCE - ECC SYSTEM OUTAGES	
11 K.3.18.A	F048	B&O TASK FORCE - ADS ACTUATION STUDY	
11 K.3.18.B	F048	B&O TASK FORCE - ADS ACTUATION PROPOSED MODIFICATIONS	
11 K.3.18.C	F048	B&O TASK FORCE - ADS ACTUATION MODIFICATIONS	
*11 K.3.19	F049	B&O TASK FORCE - INTERLOCK RECIRCULATORY PUMP MODIFICATIONS	
*11 K.3.20	F049	B&O TASK FORCE - LOSS OF SVC WATER AT BRP	
11 K.3.21.A	F050	B&O TASK FORCE - RESTART OF CSS & LPCI LOGIC DESIGN	
11 K.3.21.B	F050	B&O TASK FORCE - RESTART OF CSS & LPCI LOGIC DESIGN MODIFICATIONS	
11 K.3.22.A	F051	B&O TASK FORCE - RCIC SUCTION VERIFICATION PROCEDURES	
11 K.3.22.B	F051	B&O TASK FORCE - RCIC SUCTION MODIFICATION	
11 K.3.24	F052	B&O TASK FORCE - SPACE COOLING FOR HPCI/RCI LOSS OF AC POWER	
11 K.3.25.A	F053	B&O TASK FORCE - POWER ON PUMP SEALS PROPOSED MODIFICATIONS	
11 K.3.25.B	F053	B&O TASK FORCE - POWER ON PUMP SEALS MODIFICATIONS	
11 K.3.27	F054	B&O TASK FORCE - COMMON REFERENCE LEVEL FOR BWRs	
11 K.3.28	F055	B&O TASK FORCE - QUALIFICATION OF ADS ACCUMULATORS	
*11 K.3.29	F056	B&O TASK FORCE - PERFORMANCE OF ISOLATION CONDENSERS	
11 K.3.30.A	F057	B&O TASK FORCE - SCHEDULE FOR OUTLINE OF SB LOCA MODEL	
11 K.3.30.B	F057	B&O TASK FORCE - SB LOCA MODEL, JUSTIFICATION	
11 K.3.30.C	F057	B&O TASK FORCE - SB LOCA METHODS NEW ANALYSES	
11 K.3.31	F058	B&O TASK FORCE - COMPLIANCE WITH CFR 50.46	
*11 K.3.40	F059	B&O TASK FORCE - RCP SEAL DAMAGE - COVERED BY 11 K.2.16 AND 11 K.3.25	
*11 K.3.43	F060	B&O TASK FORCE - EFFECTS OF SLUG FLOW - COVERED BY 11 K.2.15	
11 K.3.44	F061	B&O TASK FORCE - EVALUATE TRANSIENT WITH SINGLE FAILURE	
11 K.3.45	F061	B&O TASK FORCE - ANALYSES TO SUPPORT	
11 K.3.46	F062	RESPONSE TO LIST OF CONCERNS FROM ACRS CONSULTANT	
*11 K.3.57	F062	IDENTIFY WATER SOURCES PRIOR TO MANUAL ACTIVATION OF ADS	
11 A.1.1		EMERGENCY PREPAREDNESS, SHORT TERM	

ISSUE NUMBER	MULTI-PLANT ACTION NO.	ISSUE TITLE	LICENSEE IMPLEMENTATION STATUS
III.A.1.2.1		*UPGRADE EMERGENCY SUPPORT FACILITIES - INTERIM TSC OSC & EOF.	
III.A.1.2.2	F064, F065)	UPGRADE EMERGENCY SUPPORT FACILITIES-DESIGN-INCORP. INTO F063/F064/F065	
(SEE F063,			
III.A.1.2.3	F064, F065)	UPGRADE EMER SUPPORT FACILITIES - MODS INCORPOR. INTO F063, F064 & F065	
(SEE F063,			
III.A.2.1	F067	UPGRADE PREPAREDNESS - UPGRADE EMERGENCY PLANS TO APP. E. 10 CFR 50.	
III.A.2.2	F068	UPGRADE PREPAREDNESS - METEOROLOGICAL DATA.	
*III.D.1.1.1		PRIMARY COOLANT OUTSIDE CONTAINMENT - LEAK REDUCTION.	
*III.D.1.1.2		PRIMARY COOLANT OUTSIDE CONTAINMENT - TECH SPECS.	
III.D.3.3.1		INPLANT RAD. MONIT. - PROVIDE MEANS TO DETER. PRESENCE OF RADIOIODINE.	
III.D.3.3.2	F069	INPLANT RADIATION MONIT. - MODIFICATIONS TO ACCURATELY MEAS. IODINE.	
III.D.3.4.1	F070	CONTROL ROOM HABITABILITY - REVIEW.	
*III.D.3.4.2	F070	CONTROL ROOM HABITABILITY - SCHEDULE MODIFICATIONS.	
*III.D.3.4.3		CONTROL ROOM HABITABILITY - IMPLEMENT MODIFICATIONS.	
MPA-F008	F008	I.D.1.1 DETAILED CONTROL ROOM DESIGN REVIEW PROGRAM PLAN.	
MPA-F063	F063	III.A.1.2 TECHNICAL SUPPORT CENTER.	
MPA-F064	F064	III.A.1.2 OPERATIONAL SUPPORT CENTER.	
MPA-F065	F065	III.A.1.2 EMERGENCY OPERATIONS FACILITY.	
MPA-F071	F071	* I.D.1.2 DETAILED CONTROL ROOM REVIEW (FOLLOWUP TO F-8)	
MPA-B072	B072	* NUREG-0737 TECH SPECS (GENERIC LETTERS 82-16 & 83-02).	
MPA-B083	B083	TECH SPEC COVERED BY GENERIC LETTERS 83-26 & 83-37 FOR NUREG-0737.	
67.4.1	G001	REACTOR COOLANT PUMP TRIP (GENERIC LETTER 85-12).	

Your prompt attention to this matter within the requested time frame is appreciated.

Sincerely,

/S/

Thomas E. Murley, Director
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

DISTRIBUTION: w/encl.

Docket File

NRC PDR/LPDR

PDI-2 Reading

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OGC

EJordan

BGrimes

ACRS (10)

TMurley

JSniezek

JPartlow

DF01
1/1

[PB TMI]

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