



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 137
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated August 18, 1986 and supplemented January 25, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.137, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 10, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 137

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages

Table 3.5-2 (2 of 3)
Table 3.5-2 (3 of 3)
Table 4.1-1 (page 2)

Insert Pages

Table 3.5-2 (2 of 3)
Table 3.5-2 (3 of 3)
Table 4.1-1 (page 2)
Table 4.1-1 (new page 6 of 6)

TABLE 3.5-2 (Continued)

	1	2	3	4	5
10. Low Flow Loop $\geq 75\%$ F.P.	3/loop	2/loop(any loop)	2/operable loop	1/operable loop	Maintain hot shutdown
Low Flow Two Loops 10-75% F.P.	3/loop	2/loop(any two loops)	2/operable loop	1/operable loop	
11. Lo Lo Steam Generator Water Level	3/loop	2/loop	2/loop	1/loop	Maintain hot shutdown
12. Undervoltage 6.9 KV Bus	1/bus	2	3	2	Maintain hot shutdown
13. Low frequency 6.9 KV Bus	1/bus	2	3	2	Maintain hot shutdown***
14. Quadrant power tilt monitors	2	NA	1	0	Log individual upper and lower ion chamber currents once/shift and after load change $> 10\%$
15. Turbine trip (overspeed protection)****	3	2	2	1	Maintain hot shutdown
16. Control Rod Protection****	3	2	2	1	During RCS cooldown, manually open reactor trip breakers prior to Tcold decreasing below 350°F. Maintain reactor trip breakers open during RCS cooldown when Tcold is less than 350°F.
17. Turbine Trip $\geq 35\%$ F.P. A. Low Auto Stop Oil Pressure	3	2	2	1	Maintain reactor power below 35% F.P.
18. Reactor Trip Logic	2	1	2#	1#	Be in Hot Shutdown within the next six hours.

TABLE 3.5-2 (Continued)

19. Reactor Trip Breakers	1	2	3	4	5
	2	1	2#	1#	

With either diverse trip feature inoperable, or the breaker incapable of tripping for any other reason, restore it to operable condition or, be in hot shutdown within the next six hours and open both reactor trip breakers. The breaker shall not be bypassed except for the time required for performing maintenance and/or testing to restore it to operability.

F.P. = Rated Power

* If two of four power channels greater than 10% F.P., channels are not required.

** If one of two intermediate range channels greater than 10-10 amps, channels are not required.

*** 2/4 trips all four reactor coolant pumps.

**** Required only when control rods are positioned in core locations containing LOPAR fuel.

***** This will provide a turbine trip at all power levels and a reactor trip when greater than or equal to 35% F.P..

A reactor trip breaker and/or associated logic channel may be bypassed for maintenance or surveillance testing for up to eight hours provided the redundant reactor trip breaker and/or associated logic channel is operable.

TABLE 4.1-1 (CONTINUED)

Channel Description	Check	Calibrate	Test	Remarks
22. Accumulator Level and Pressure	S	R	N.A.	
23. Steam Line Pressure	S	R	M	
24. Turbine First Stage Pressure	S	R	M	
25. Reactor Trip Logic Channel Testing	N.A.	N.A.	M#	
26. Turbine Overspeed Protection Trip Channel (Electrical)	N.A.	R	M	
27. Turbine Trip a. Low Auto Stop Oil Pressure	N.A.	R	N.A.	
28. Control Rod Protection (for use with LOPAR fuel)	N.A.	R	*	
29. Loss of Power a. 480v Emergency Bus Undervoltage (Loss of Voltage)	N.A.	R	R	
b. 480v Emergency Bus Undervoltage (Degraded Voltage)	N.A.	R	R	
c. 480v Emergency Bus Undervoltage (Alarm)	N.A.	R	M	
30. Auxiliary Feedwater: a. Steam Generator Water Level (Low-Low)	S	R	R	

*Within 31 days prior to entering a condition in which the Control Rod Protection System is required to be operable unless the reactor trip breakers are manually opened during RCS cooldown prior to T_{cold} decreasing below 350°F and the breakers are maintained opened during RCS cooldown when T_{cold} is less than 350°F.

TABLE 4.1-1 (CONTINUED)

<u>Channel Description</u>	<u>Check</u>	<u>Calibrate</u>	<u>Test</u>	<u>Remarks</u>
42. Manual Reactor Trip	N.A.	N.A.	R	Includes: 1) Independent verification of reactor trip and bypass breakers undervoltage trip circuit operability up to and including matrix contacts of RT-11/RT-12 from both manual trip initiating devices, 2) independent verification of reactor trip and bypass breaker shunt trip circuit operability through trip actuating devices from both manual trip initiating devices.
43. Reactor Trip Breaker	N.A.	N.A.	M#	Includes independent verification of undervoltage and shunt trip attachment operability.
44. Reactor Trip Bypass Breaker	N.A.	N.A.	M#	Includes: 1) Automatic undervoltage trip, 2) Manual shunt trip from either the logic test panel or locally at the switchgear prior to placing breaker into service.

Each train shall be tested at least every 62 days on a staggered test basis (i.e., one train per month).