ENCLOSURE 1

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
NRC DOCKETS 50-325 & 50-324
OPERATING LICENSES DPR-71 AND DPR-62
INDEX REVISION
(NRC TAC NOS. 69048 AND 69049)

UNIT 1 TECHNICAL SPECIFICATION PAGES

SUMMARY LIST OF REVISIONS

UNIT 1

Pages	Description of Changes
III	Since only reformatting is involved, no change bars have been provided. The page has been updated to incorporate changes to pagination of Sections 2.1 and 2.2 (approved by Amendment 124 dated February 6, 1989) and to remove the word processing code number in upper right-hand corner of the page.
IV	The page has been updated to incorporate changes to pagination of Section 3/4.2 (approved by Amendment 124 dated February 6, 1989) and to remove the word processing code number in upper right-hand corner of the page.
V	The changes originally requested to this page have already been incorporated through the issuance of Amendment 123 dated February 6, 1989. Therefore, issuance of this page is no longer needed and the page has been omitted from this package.
VII	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
VIII	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.

UNIT 1

Pages	Description of Changes
IX	Since only reformatting is involved, no change bars have been provided. The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
IXa	Since only reformatting is involved, no change bars have been provided. The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
X	Since only reformatting is involved, no change bars have been provided. The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
XI	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
XII	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
XIIa	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
XIV	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.

UNIT 1

Pages	Description of Changes
XV	Since only reformatting is involved, no change bars have been provided. The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
XVI	Since only reformatting is involved, no change bars have been provided. The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
3/4 11-9	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
B 3/4 5-4	This page was listed in the Instructions for Incorporation (Enclosure 3)and the Summary List of Revisions (Enclosure 4), but the revised page was inadvertently omitted from Enclosure 5. The page has been revised as originally described, except for the omission of the word processing code number in the upper right-hand corner of the page.
6-9	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.

INDEX

SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

BASES

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

INDEX

BASES

BASES

INDEX

BASES

INDEX

BASES

INDEX

ADMINISTRATIVE CONTROLS

INDEX

ADMINISTRATIVE CONTROLS

INDEX

ADMINISTRATIVE CONTROLS

RADIOACTIVE EFFLUENTS

LIQUID RADWASTE TREATMENT SYSTEM

LIMITING CONDITION FOR OPERATION

3.11.1.3 The liquid radwaste treatment system shall be used to reduce the radioactive materials in liquid wastes prior to their discharge when the projected doses due to the liquid effluent from the site to UNRESTRICTED AREAS (see Figure 5.1.3-1) would exceed 0.12 mrem to the total body or 0.4 mrem to any organ in a 31-day period.

APPLICABILITY: At all times.

ACTION:

a. With radioactive liquid waste being discharged without treatment and in excess of the above limits, in lieu of a Licensee Event Report, prepare and submit to the Commission within 30 days, pursuant to Specification 6.9.2, a Special Report that includes the following information:

   1. Explanation of why liquid radwaste was being discharged without treatment, identification of any inoperable equipment or subsystem, and reason for the inoperability.

   2. Action(s) taken to restore the inoperable equipment to OPERABLE status, and

   3. Summary of description of action(s) taken to prevent a recurrence.

b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.11.1.3 Doses due to liquid releases from the site to UNRESTRICTED AREAS shall be projected at least once per 31 days in accordance with the ODCM.

NOTE: See Bases 3/4.11.1.3

3/4.5.4 SUPPRESSION POOL

The OPERABILITY of the suppression pool in CONDITIONS 1, 2, or 3 is required by Specification 3.6.2.2. The suppression pool, as part of the ECCS, ensures that a sufficient supply of water is available in the event of a LOCA. This limit or suppression pool minimum water volume ensures that sufficient water is available to permit recirculation cooling flow to the core.

The technical specifications for cooling the reactor core do not make allowances for repair work that might require making the suppression pool inoperable unless all irradiated fuel is removed. This specification will permit those repairs to be made and at the same time give assurance that the irradiated fuel has an adequate cooling water supply when the suppression pool must be made inoperable, including draining.

6.5 REVIEW AND AUDIT

6.5.1 NUCLEAR SAFETY REVIEWERS

6.5.1.1 Individuals shall be designated/approved by the General Manager - Brunswick Plant for performing nuclear safety reviews.

6.5.1.2 Individuals designated under Specification 6.5.1.1 above shall have an academic degree in an engineering or related field or equivalent and two years related experience.

6.5.1.3 A list shall be maintained of individuals qualified to perform nuclear safety reviews, including additional individuals whose expertise may be necessary during the reviews to assure that the reviewers collectively possess the background and qualifications in the disciplines necessary and important to the specific review.

6.5.1.4 The list specified in Specification 6.5.1.3 above shall include the disciplines for which each individual is qualified.

6.5.1.5 For those cases where interdisciplinary reviews are required, as many individuals as necessary shall be used to perform the nuclear review function.

6.5.1.6 One of the two nuclear safety reviewers shall be an individual other than the original preparer of the individual approving the action.

6.5.2 SAFETY EVALUATIONS AND INDEPENDENT REVIEW CONTROL

SAFETY EVALUATIONS

6.5.2.1 A safety evaluation shall be prepared for each of the following:

a. Changes to procedures required by Specification 6.8, or changes to other procedures that affect nuclear safety.

b. Proposed tests or experiments that affect nuclear safety.

c. Proposed modifications to plant systems or equipment that affect nuclear safety.

d. Proposed changes to the Technical Specifications.

e. Proposed changes to the Operating License.

6.5.2.2 Two nuclear safety reviews of the item and safety evaluation(s) prepared in accordance with Specification 6.5.2.1 above shall be performed prior to approval and implementation.

6.5.2.3 The item and associated safety evaluation(s) shall be examined in order to determine whether an interdisciplinary review is required in accordance with Specification 6.5.1.5 above.

ENCLOSURE 2

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
NRC DOCKETS 50-325 & 50-324
OPERATING LICENSES DPR-71 AND DPR-62
INDEX REVISION
(NRC TAC NOS. 69048 AND 69049)

UNIT 2 TECHNICAL SPECIFICATION PAGES

SUMMARY LIST OF REVISIONS

UNIT 2

Pages	Description of Changes
III	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
IV	Updated to incorporate changes to pagination of Section 3/4.2 (approved by Amendment 149 dated February 6, 1989) and to remove word processing code number in upper right-hand corner of the page.
V	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
VI	Since only reformatting is involved, no change bars have been provided. The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
VII	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
VIII	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.

UNIT 2

Pages	Description of Changes
IX	Since only reformatting is involved, no change bars have been provided. The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
IXa	Since only reformatting is involved, no change bars have been provided. The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
X	This page was listed in the Instructions for Incorporation (Enclosure 3)and the Summary List of Revisions (Enclosure 4), but the revised page was inadvertently omitted from Enclosure 5. The page is unchanged from the original submittal except for removal of the word processing code number in the upper right-hand corner of the page has also been omitted.
XI	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
XII	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
XIIa	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.

UNIT 2

Pages	Description of Changes
XIV	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
XV	Since only reformatting is involved, no change bars have been provided. The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
XVI	Since only reformatting is involved, no change bars have been provided. The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
3/4 1-13	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
3/4 11-9	The page is unchanged from the original submittal except for removal of the word processing code number in upper right-hand corner of the page.
B 3/4 5-4	This page was listed in the Instructions for Incorporation (Enclosure 3)and the Summary List of Revisions (Enclosure 4), but the revised page was inadvertently omitted from Enclosure 5. The page has been revised as originally described, except for the omission of the word processing code number in the upper right-hand corner of the page.

INDEX

SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

BASES

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

BASES

BASES

BASES

BASES

INDEX

ADMINISTRATIVE CONTROLS

REACTIVITY CONTROL SYSTEMS

CONTROL ROD DRIVE HOUSING SUPPORT

LIMITING CONDITION FOR OPERATION

3.1.3.8 The control rod drive housing support shall be in place when there is fuel in the reactor vessel.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

With the control rod drive housing support not in place, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.1.3.8 The control rod drive housing support shall be inspected after reassembly and verified to be in place prior to start-up any time it has been disassembled or when maintenance has been performed in the control rod drive housing support area.

RADIOACTIVE EFFLUENTS

LIQUID RADWASTE TREATMENT SYSTEM

LIMITING CONDITION FOR OPERATION

3.11.1.3 The liquid radwaste treatment system shall be used to reduce the radioactive materials in liquid wastes prior to their discharge when the projected doses due to the liquid effluent from the site to UNRESTRICTED AREAS (see Figure 5.1.3-1) would exceed 0.12 mrem to the total body or 0.4 mrem to any organ in a 31-day period.

APPLICABILITY: At all times.

ACTION:

a. With radioactive liquid waste being discharged without tre. tment and in excess of the above limits, in lieu of a Licensee Event Report, prepare and submit to the Commission within 30 days, pursuant to Specification 6.9.2, a Special Report that includes the following information:

   1. Explanation of why liquid radwaste was being discharged without treatment, identification of any inoperable equipment or subsystem, and reason for the inoperability.

   2. Action(s) taken to restore the inoperable equipment to OPERABLE status, and

   3. Summary of description of action(s) taken to prevent a recurrence.

b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.11.1.3 Doses due to liquid releases from the site to UNRESTRICTED AREAS shall be projected at least once per 31 days in accordance with the ODCM.

NOTE: See Bases 3/4.11.1.3

3/4.5.4 SUPPRESSION POOL

The OPERABILITY of the suppression pool in CONDITIONS 1, 2, or 3 is required by Specification 3.6.2.2. The suppression pool, as part of the ECCS, ensures that a sufficient supply of water is available in the event of a LOCA. This limit on suppression pool minimum water volume ensures that sufficient water is available to permit recirculation cooling flow to the core.

The technical specifications for cooling the reactor core do not make allowances for repair work that might require making the suppression pool inoperable unless all irradiated fuel is removed. This specification will permit those repairs to be made and at the same time give assurance that the irradiated fuel has an adequate cooling water supply when the suppression pool must be made inoperable, including draining.