LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

MARCH 1989

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373 LICENSE NO. NPF-11

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TABLE OF CONTENTS

I. INTRODUCTION

II. REPORT

- SUMMARY OF OPERATING EXPERIENCE A.
- PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED В. MAINTENANCE
 - 1. Amendments to Facility License or Technical Specifications
 - 2. Changes to procedures which are described in the Safety Analysis Report
 - 3. Tests and Experiments not covered in the Safety Analysis Report
 - 4. Corrective Maintenance of Safety-Related Equipment
 - 5. Completed Safety-Related Modifications
- C. LICENSEE EVENT REPORTS
- DATA TABULATIONS D.
 - 1. Operating Data Report
 - 2. Average Daily Unit Power Level
 - 3. Unit Shutdowns and Power Reductions
- UNIQUE REPORTING REQUIREMENTS
 - 1. Main Steam Relief Valve Operations

 - ECCS System Outages
 Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System
 - 5. Indications of Failed Fuel Elements

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by John W. Thunstedt, telephone number (815)357-6761, extension 2463.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE

Day	Time	Event
1	0000	Reactor critical, generator on-line, 960 MWE.
2	2302	Reactor scram, turbine trip, due to fault of U-2 SAT lightning arrestor.
6	0042	Reactor critical.
	1055	Generator on-line, ramping to 1050 MWE.
8	2200	Reduced load to 600 MWE for rod-set.
9	0030	Ramping to 1130 MWE.
31	2400	Reactor critical, generator on-line, 1080 MWE.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

- Amendments to the Facility License or Technical Specification. (None)
- Changes to procedures which are described in the Safety Analysis Report.
 (None)
- Tests and Experiments not described in the Safety Analysis Report. (None)
- 4. Major corrective maintenance to Safety-Related Equipment. (See Table 1)
- Completed Safety-Related Modifications. (None)

B.4 TABLE 1

MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT

CORRECTIVE ACTION		Replaced valves.		Replaced Cell	Replaced missing ball
RESULTS AND EFFECTS ON SAFE PLANT OPERATION		Slight leak-through, no significant operational effect		(None)	CRD Inoperable
CAUSE OF MALFUNCTION		Ruptured diaphram		Low Specific Gravity	Missing internal component (ball)
COMPONENT		"0" D/G Air-Start Regulators		Div I Battery Cell #8	HCU 42-27 Charging Water Check Valve
WORK REQUEST NUMBER	(UNIT 0)	L83707	(UNIT 1)	L33949	L88050

C. LICENSEE EVENT REPORTS

LER Number	Date	Description
89-009-00	03/02/89	Reactor Scram Due to Loss of Transformer 242/Fault in Lightning Arrestor.
89-010-00	03/03/89	Reactor Vessel Low Water Level Confirmed ADS/Setpoint Draft
89-011-00	03/04/89	HPCS System Inoperable
89-012-00	03/09/89	RCIC Hi Steam Flow Isolation SOR Failure
89-013-00	03/31/89	Suppression Pool Hi Level Alarm Setpoint Non-conservative

D. DATA TABULATIONS

- 1. Operating Data Report (attached)
- 2. Average Daily Unit Power Level (attached)
- 3. Unit Shutdowns and Power Reductions (attached)

D.1 OPERATING DATA REPORT

DOCKET NO. 050-373 UNIT LASALLE ONE DATE April 10, 1989 COMPLETED BY J.W. THUNSTEDT TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: MARCH 1989

GROSS HOURS IN REPORTING PERIOD:

744

2. CURRENTLY AUTHORIZED POWER LEVEL (MW+): 3,323

MAX DEPEND CAPACITY (MWe-Net): DESIGN ELECTRICAL RATING (MWe-Net) 1,078

1,036

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):

(None)

4. REASONS FOR RESTRICTION (IF ANY): (N/A)

REPORTING PERIOD DATA

		THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. TIME	REACTOR CRITICAL (HOURS)	670.3	2,086.3	28,105.6
6. TIME	REACTOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	1,641.2
7. TIME	GENERATOR DN-LINE (HOURS)	660.1	2,076.1	27,390.6
8. TIME	GENERATOR RESERVE SHUTDOWN (HOURS)	0.0	0.9	1.0
9. THERM	MAL ENERGY GENERATED (MWHt-Gross)	2,068,008	6,647,664	75,426,242
10. ELECT	RICAL ENERGY GENERATED (NWHe-Gross)	767,667	2,278,676	24,924,430
11. ELECT	RICAL EMERGY GENERATED (MWHe-Net)	683,022	2,193,413	23,768,942
12. REACT	OR SERVICE FACTOR (%)	90.1	96.6	61.1
13. REACT	OR AVAILABILITY FACTOR (%)	90.1	96.6	64.7
14. SERVI	CE FACTOR (%)	88.7	96.1	59.5
15. AVAIL	IBILITY FACTOR (Z)	88.7	96.1	59.5
16. CAPAC	ITY FACTOR .JSING MDC) (%)	88.6	98.0	49.9
17. CAPAC	TITY FACTOR (USING DESIGN HWe) (%)	85.2	94.2	47.9
18. FORCE	D DUTAGE FACTOR (Z)	11.3	3.9	11.5

^{19.} SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): Refueling Sept 5, 1989 12 Weeks

^{20.} IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: (N/A)

D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373 UNIT LASALLE ONE DATE April 10, 1989 COMPLETED BY J.W. THUNSTEDT TELEPHONE (815)-357-6761

REPORT PERIOD: MARCH 1989

DAY	POWER	DAY	POWER
1	899	17	1,099
2	882	18	1,096
3	-11	19	1,099
4	-11	20	1,100
5	-11	21	1,101
6	101	22	1,089
7	695	23	1,098
8	958	24	1,095
9	950	25	1,096
10	1,094	26	1,092
11	1,098	27	1,071
12	1,098	28	1,065
13	1,099	39	1,063
14	1,098	30	1,090
15	1,079	31	1,087
16	1,098		

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%

REPORT MONTH: MARCH, 1989

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	т	ĸ
REASON	A	æ
DURATION (HOURS)	83.9	0.0
TYPE F: FORCED S: SCHEDULED	<u>St.</u>	ω
AL DATE	7	80
YEARLY SEQUENTIAL NUMBER	w	9

DATE April 10, 1989 COMPLETED BY J. Thunstedt TELEPHONE (815)357-6761 UNIT NAME LaSalle One DOCKET NO. 050-373

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Failure of lighting arrestor on U-2 SAT

Rod-set

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations

VALVES NO & TYPE PLANT DESCRIPTION
DATE ACTUATED ACTUATION CONDITION OF EVENT

(None)

2. ECCS System Outages

OUTAGE NO.	EQUIPMENT	PURPOSE
(Unit 0)		
0-82-89	0 D/G Cooling Water Pump	Lubrication.
0-83-89	O D/G Air-Start Solenoid and Regulator.	Repair/replace regulator.
(UNIT 1) 1-122-89	HPCS D/G Strainer	Adjust valve stroke
1-125-89	JE12-D311B (RH)	Replace "B" RHR vent sight-glass.
1-126-89	"C" RHR Pump	Replace motor oil.
1-127-89	"B" RHR Pump	Replace motor oil.
1-128-89	RHR Service Water pumps C&D	Lubricate coupling.
1-130-89	1E12-F073B	Install limiter plate.
1-133-89	1A D/G	Lubricate.
1-134-89	1A D/G	Install gasket on bypass lube-oil valve.

E.2. ECCS System Outages (Cont'd)

OUTAGE NO.	EQUIPMENT	PURPOSE
1-147-89	1E12-C002A	Lubricate "1A" RHR motor.
1-148-89	1E21-C001	Lubricate HPCS motor
1-149-89	1E21-C002	Lubricate HPCS water-leg pump coupling.
1-150-89	RHR Service-Water Pumps	Lubricate coupling.
1-153-89	1E51-F059 (RCIC)	(Administrative)
1-156-89	1B D/G	Lubricate and change filters.
1-157-89	1E22-C003 HCPS Water-leg pump	Replace oil
1-158-89	1E22-C003 HPCS Water-leg pump	Lubricate coupling.
1-159-89	1E22-C003 HPCS Water-leg pump	Lubricate motor bearing.
1-160-89	1DO02F HPCS D/G Fuel Pump	Replace oil.
1-161-89	1DO02P HPCS D/G Fuel Pump	Lubricate motor.
1-162-89	1DO02P HPCS D/G Fuel Fump	Lubricate coupling.
1-163-89	HPCS Pump	Lubricate motor.
3.	Off-Site Dose Calculation Ma (None)	anual Changes
4.	Major changes to Radioactive (None)	e Waste Treatment Systems.
5.	Indications of Failed Fuel I	Elements

(None)

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

MARCH, 1989

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

TABLE OF CONTENTS

I. INTRODUCTION

II. REPORT

- A. SUMMARY OF OPERATING EXPERIENCE
- B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE
 - 1. Amendments to Facility License or Technical Specifications
 - Changes to procedures which are described in the Safety Analysis Report.
 - 3. Tests and Experiments not covered in the Safety Analysis Report.
 - 4. Corrective Maintenance of Safety-Related Equipment
 - 5. Completed Safety Related Modifications
- C. LICENSEE EVENT REPORTS
- D. DATA TABULATIONS
 - 1. Operating Data Report
 - 2. Average Daily Unit Power Level
 - 3. Unit Shutdowns and Power Reductions
- E. UNIQUE REPORTING REQUIREMENTS
 - 1 Safety/Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System
 - 5. Indications of Failed Fuel Elements

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseillas, Illinois. Each unit is a Boiling the Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by John W. Thunstedt, telephone number (815)357-6761 extension 2463.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE

Day	Time	Event
1	0000	Reactor critical, generator on-line, 1100 MWE.
2	1230	Reduced load to 950 MWE to remove 1st-stage feedwater heater for maintenance.
	2300	Reduced load to 8850 MWE to recover internal loads lost by fault on SAT.
03	0300	Ramping to 950 MWE.
07	2000	Reduced load to 700 MWE for feedwater heater maintenance.
08	0530	Ramping to 1070 MWE.
09	0130	Reduced load to 950 MWE to remove "B" turbine-driven reactor feed pump for maintenance.
10	0400	Ramping to 1130 MWE.
11	1430	Reducing load to 850 MWE for feed-water heater maintenance.
12	0630	Ramping to 1130 MWE.
17	2130	Reducing load to 800 MWE for feedwater-heater maintenance.
18	0030	Ramping to 1130 MWE.
23	2130	Reducing load to 850 MWE for feedwater-heater maintenance.
24	0830	Ramping to 1130 MWE.
31	2400	Reactor critical, generator on-line, 1120 MWE.

- B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.
 - Amendments to the Facility License or Technical Specification. (None)

- Changes to procedures which are described in the Safety Analysis Report.
 (None)
- Tests and Experiments not described in the Safety Analysis Report.
 (None)
- 4. Major corrective maintenance to Safety-Related Equipment. See Table 1.
- Completed Safety-Related Modifications.
 See Table 2.

B.4 TABLE 1

MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT

	04	œ
RESULTS AND EFFECTS ON SAFE PLANT OPERATION	Excessive pump vibration	(None)
CAUSE OF MALFUNCTION	Throttling vibration	Throttling vibration
COMPONENT	"A" Reactor Recirc Pumping Element	"A" Reactor Recirc Pump Flow Control Valve.
WORK REQUEST NUMBER	L82239	L83606

CORRECTIVE ACTION

	B.5 TABLE 2
	COMPLETED SAFETY RELATED MODIFICATIONS
MODIFICATION NUMBER	DESCRIPTION
M01-2-86-103	Snubber reduction on subsystem 2RH08 per the snubber reduction program.
M01-2-87-050	Complete the installation of the inside Drywell Cooling area coolers.
M01-2-87-067	Snubber reduction on subsystem main steam and MSIV leakage control per the snubber reduction program.

C. LICENSEE EVENT REPORTS

LER Number Date Description

(None)

D. DATA TABULTTIONS

- 1. Operating Data Report (Attached)
- 2. Average Daily Unit Power Level (Attached)
- 3. Unit Shutdowns and Power Reductions (Attached)

D.1 OPERATING DATA REPORT

DOCKET NO. 050-374 UNIT LASAULE TWO DATE April 16, 1989 COMPLETED BY J.W. THUNSTEDT TELEFICHE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: KARCH 1989 GROSS HOURS IN REPORTING PERIOD: 744 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 MAX DEPEND CAPACITY (MWe-Net): 1,036 DESIGN ELECTRICAL RATING (MWe-Net) 1,978

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):

(None)

4. REASONS FOR RESTRICTION (IF ANY): (N/A)

REPORTING PERIOD DATA

	THIS MENTH	YEAR-TO-DATE	CUMULATIVE
5. TIME REACTOR CRITICAL (HOURS)	744.0	1,246.1	24,724.1
6. TIME REACTOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	1,716.9
7. TIME GENERATOR ON-LINE (HOURS)	744.0	1,194.5	24,285.5
8. TIME GENERATOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWHt-Gross)	2,321,964	3,349,968	69,289,576
10. ELECTRICAL ENCAGY GENERATED (MWHe-Gross)	773,972	1,106,844	22,810,182
11. ELECTRICAL ENERGY GENERATED (MWHe-Net)	747,725	1,064,936	21,884,186
12. REACTOR SERVICE FACTOR (%)	100.0	57.7	63.4
13. REACTOR AVAILABILITY FACTOR (%)	100.0	57.7	67.8
14. SERVICE FACTOR (%)	100.0	55.3	62.3
15. AVAILIBILITY FACTOR (%)	100.0	55.3	62.3
16. CAPACITY FACTOR (USING HDC) (%)	97.0	47.6	54.0
17. CAPACITY FACTOR (USING DESIGN NWe) (%)	93.2	45.7	51.9
18. FORCED DUTAGE FACTOR (%)	0.0	0.6	16.2

^{19.} SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): (Mone)

^{20.} IF SHUTDOWN AT END OF PEPORT PERIOD, ESTIMATED DATE OF STARTUP: (N/A)

DOCKET NO. 050-374 UNIT LASALLE TWO DATE April 10, 1989 COMPLETED BY J.W. THUNSTEDT TELEPHONE (815)-357-6761

REPORT PERIOD: MARCH 1989

DAY	POWER	DAY	POWER
1	1,028	17	1,059
2	942	18	941
3	904	19	1,077
4	912	20	1,079
5	912	21	1,079
6	910	22	1,077
7	887	23	1,045
8	891	24	952
9	906	25	1,071
10	1,017	26	1,069
fi	993	27	1,043
12	955	28	1,041
13	1,082	29	1,042
14	1,062	30	1,000
15	1,077	31	1,042
16	1,060		

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%

DOCKET NO. 050-374
UNIT NAME LASAlle Two
DATE April 10, 1989
COMPLETED BY J. Thunstedt
TELEPHONE (815)357-6761

RL. ORT MONTH: MARCH, 1989

N ITS	Recover from failure of lighting arrestor on SAT	leater	leater	ater	leater
CORRECTIVE ACTIONS/COMP ITS	Recover from failure lighting arrestor on	Feedwater-heater maintenance.	Feedwater-heater maintenance.	Feewater-heater maintenance.	Feedwater-heater maintenance.
METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	5	ſſ	Ŋ	so.	ĸ
REASON CODE	A	В	Ø	В	В
DURATION (HOURS)	4.0	0.0	0.0	0.0	0.0
TYPE F: FORCED S: SCHEDULED	St.	Ø	W	W	S
DATE	3-2-89	3-7-89	3/11/89	3/17/89	3/23/89
YEARLY SEQUENTIAL NUMBER	т	4	S	9	7

E. UNIQUE REPORTING REQUIREMENTS

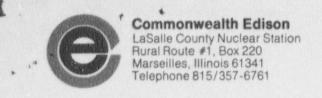
. 1. Safety/Relief Valve Operations

DATE	VALVES	NO & TYPE	PLANT	DESCRIPTION
	ACTUATED	ACTUATIONS	CONDITION	OF EVENT
(None)				

2. ECCS System Outages

OUTAGE NO.	EQUIPMENT	PURPOSE
2-391-89	2B D/G Cooling Water Pump	Replace oil.
2-392-89	2B D/G	Replace flexible conduit
2-408-89	2E51-F012 (RCIC)	Repack valve.
2-421-89	2FW01PC	Replace seals.
2-450-89	2A D/G	Replace immersion heater.
2-451-89	2A D/G	Lubricate.
2-453-89	2DG01P	Lubricate coupling.
2-454-89	2D001P	Lubricate coupling.

- 3. Off-Site Dose Calculation Manual (None)
- 4. Major changes to Radioactive Waste Treatment Systems. (None)
- 5. Indications of Failed Fuel Elements. (None)



April 10, 1989

Director of Nuclear Reactor Regulation United States Nuclear Regulatory Commission Mail Station P1-137 Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for March, 1989.

Very truly yours,

fo G. J. Diederich Station Manager

LaSalle County Station

GJD/JWT/sjc

Enclosure

xc: A. B. Davis, NRC, Region III

NRC Resident Inspector LaSalle

Gary Wright, Ill. Dept. of Nuclear Safety

P. Shemanski, NRR Project Manager

D. P. Galle, CECo

D. L. Farrar, CECo

INPO Records Center

L. J. Anastasia, AIP Coordinator, Nuclear Services

M. A. Ortin, GE Resident

H. E. Bliss, Manager of Nuclear Licensing

W. F. Naughton, Nuclear Fuel Services Manager

C. F. Dillon, Senior Financial Coordinator, LaSalle

Dennis Carlson/Tech Staff

Terry Novotney/INPO Coordinator, Tech Staff

Central File

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