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08/27/2020

Attn: Document Control Desk U. S. Nuclear Regulatory Commission Washington, DC 20555-0001

SUSQUEHANNA STEAM ELECTRIC STATION LICENSEE EVENT REPORT 50-387/2020-002-00 UNIT 1 LICENSE NO. NPF-14 PLA-7889

Docket No. 50-387

10 CFR 50.73

Attached is Licensee Event Report (LER) 50-387/2020-002-00. The LER reports an event involving failure of the Unit 1 "B" Residual Heat Removal Service Water pump to start that was determined to be reportable in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by Technical Specifications.

There were no actual consequences to the health and safety of the public as a result of this event.

This letter contains no new or revised regulatory commitments.

K. Cimorelli

Attachment: LER 50-387/2020-002-00

Copy: NRC Region I Mr. M. Hardgrove, NRC (Acting) Senior Resident Inspector Ms. S. Goetz, NRC Project Manager Mr. M. Shields, PA DEP/BRP

NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION						APPROVED BY OMB: NO. 3150-0104 EXPIRES: 08/31/2023										
(08-2020)								Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported								
LICENSEE EVENT REPORT (LER)								lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6								
E DZ	IS.	6		lired number of		•	•	ch)	A10M), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to							
	Ľ,	5			-				Infocollects.Resource@nrc.gov, and the OMB reviewer at: OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk ail: <u>oira_submission@omb.eop.gov</u> . The NRC may not conduct or							
(See NUREG-1022, R.3 for instruction and guidance for completing this form <u>https://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/</u>) Affairs, (310-0104), Affairs								ot required to re	spond to, a	a collection of I	nformation un	less the document				
1	1. Facility Name									ocket Number			3. Page	3. Page		
Susq	ueh	anna Stea	m Elec	tric Statio	n Unit	1			05	000387				1 OF	3	
4. Title Inop	4. Title Inoperability of Unit 1 "B" Residual Heat Removal Service Water Pump															
	5. Event Date 6. LER Number 7. Report Date 8. Other Facilities Involved															
										Facility Name	_	I			ocket Number	
Month	Day	Year	Year	Sequential Number	Rev No.	Month	Day	Ye	ear					05000		
07	01	2020	2020		00		07	20	200	Facility Name				Docket N	umber	
07	01	2020	2020	- 002 -	00	08	27	20	020 05000							
9. Operating Mode 10. Power Level																
	1 100															
	11. This Report is Submitted Pursuant to the Requirements of 10 CFR §: (Check all that apply)															
10	CF	R Part 20		20.2203(a)(2)		50.36				50.73(a)(2)(i			73(a)(2)(x)			
20.2	201(b)		20.2203(a)(3)(i) 50.46(a)(3)(ii)				□ 50.73(a)(2)(v)(A)			10 CFR Part 73					
20.2	201(d)		□ 20.2203(a)(3)(ii) □ 50.69(g)					50.73(a)(2)(v)(B)			☐ 73.71(a)(4)				
20.2	203(a)(1)		20.2203(a)(4)			50.73(a)(2)(i)(A)			50.73(a)(2)(v)(C)		[] 73.71(a)(5)				
20.2	203(a)(2)(i)	10	10 CFR Part 21			⊠ 50.73(a)(2)(i)(B)						□ 73.77(a)(1)(i)			
								☐ 50.73(a)(2)(i)(C)					□ 73.77(a)(2)(i)			
	□ 20.2203(a)(2)(ii) □ 20.2203(a)(2)(iii)						☐ 50.73(a)(2)(ii)(A)						77(a)(2)(ii)			
			0,009,29,9							□ 50.73(a)(2)(viii)(B)						
20.2203(a)(2)(iv)							50.73(a)(2)(ii)(B)									
	□ 20.2203(a)(2)(v) □ 50.36(c)(1)(ii)(A) □ 50.73(a)(2)(iii) □ 50.73(a)(2)(ix)(A) □ Other (Specify here, in Abstract, or in NRC 366A). -															
			Abstract,		ionj.	12 Lice	nsoo Co	ntact	forf	this LER						
Licensee C	ontact					12, LICE	11366 00	maor	101 1			Phon	e Number (inc	lude Area Co	de)	
C. E. Manges, Jr, Senior Engineer – Nuclear Regulatory Affairs (570) 542-3089																
	¥		U						nt Fail	lure Described in t	his Report					
Cause System		System	Component Manufacturer		Reportable To IRIS		Ca	use	System Compor		ent	Manufactu	rer Rep	Reportable To IRIS		
		14. Ourmlan			- 4					<u>l</u>			Month	Day	Year	
		14. Supplen								15. Expected Submission Da				-		
	I	Yes (If y		-								10	09	2020		
16. Abstract (Limit to 1560 spaces, i.e., approximately 15 single-spaced typewritten lines)																
	On July 1, 2020 at approximately 20:26, the Unit 1 "B" Residual Heat Removal Service Water (RHRSW) pump did															
	not start when being placed into service for Spray Pond Cooling testing, resulting in the pump being declared															
inoperable. Upon the discovery of the condition, Susquehanna entered Technical Specification (TS) 3.7.1 Condition B for one RHRSW subsystem being inoperable. Investigation at the pump circuit breaker identified the																
closing springs did not recharge following the circuit breaker's last closing operation. Following maintenance, the																
spring charging motor operated as expected and the breaker was tested successfully. TS 3.7.1 Condition B was																
exited on July 2, 2020 at approximately 02:44.																
Based on information available, the condition is considered to have existed for longer than allowed by TS 3.7.1.																
The condition is being reported in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by																
Techni	cal	Specificati								gation and v						
to this	LEF	λ .														
There v	There were no actual consequences to the health and safety of the public as a result of this event.															

NRC FORM 366A	U.S. NUCLEAR REGULA	TORY COMMISSION	APPROVED BY OMB: NO. 3150-010	EXPIRES: 08/31/2023				
	LICENSEE EVENT RE CONTINUATION S 2, R.3 for instruction and guidance f	SHEET	Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to Infocollects.Resource@nrc.gov, and the OMB reviewer at: OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk ali: <u>oira submission@omb.eop.gov</u> . The NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless the document requesting or requiring the collection displays a currently valid OMB control number.					
1. FACILITY NAME 2. DOCKI			NUMBER	3. LER NUMBER				
Susquehanna	Steam Electric Station Unit	05000-387		YEAR	SEQUENTIAL NUMBER	REV NO.		
1				2020	- 002	- 00		

NARRATIVE

CONDITIONS PRIOR TO EVENT

Unit 1 - Mode 1, approximately 100 percent Rated Thermal Power

Unit 2 – Mode 1, approximately 100 percent Rated Thermal Power

There were no structures, systems, or components that were inoperable at the start of the event that contributed to the event.

EVENT DESCRIPTION

On July 1, 2020 at approximately 20:26, the Unit 1 "B" Residual Heat Removal Service Water (RHRSW) pump (1P506B) [EIIS System Code/Component Code: BI/P] did not start when being placed into service for Spray Pond Cooling testing, resulting in the pump being declared inoperable. Upon discovery of the condition, Susquehanna entered Technical Specification (TS) 3.7.1 Condition B for one RHRSW subsystem being inoperable.

On July 2, 2020, investigation at the pump circuit breaker (1A20408) [EIIS Component Code: 72] identified the closing springs did not recharge following the circuit breaker's last closing operation on June 21, 2020 at approximately 07:35. Following maintenance, the spring charging motor operated as expected and the breaker was tested successfully. TS 3.7.1 Condition B was exited on July 2, 2020 at approximately 02:44.

Based on information available, the condition is considered to have existed for longer than allowed by TS 3.7.1. The condition is being reported in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by Technical Specification.

CAUSE OF EVENT

A cause evaluation is in progress. A supplement will be issued to provide information regarding the cause of the condition.

ANALYSIS/SAFETY SIGNIFICANCE

During the period the 1B RHRSW pump was inoperable, the 1A RHRSW pump was verified to be operable. An engineering evaluation was completed to analyze having only the 1A RHRSW pump available with the 1B RHRSW pump inoperable. The engineering evaluation concluded that, with only the 1A RHRSW pump in operation, the accident analysis assumptions were met and the RHRSW system could perform its design function. Based on the engineering evaluation and verification that the 1A RHRSW pump was operable during the period the 1B RHRSW pump was inoperable, the condition described herein did not result in a safety system functional failure. Accordingly, this event will not be counted as a safety system functional failure in the Reactor Oversight Process Performance Indicators. There were no actual consequences to the health and safety of the public as a result of this event.

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NRC FORM 366A	U.S. NUCLEAR REGULA	TORY COMMISSION	APPROVED BY OMB: NO. 3150-010)4	EXPIRES:	08/31/2023			
	LICENSEE EVENT RE CONTINUATION S 22, R.3 for instruction and guidance for c.gov/reading-rm/doc-collections/nure	SHEET	Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to Infocollects.Resource@nrc.gov, and the OMB reviewer at: OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk ali: <u>oira submission@omb.eop.gov</u> . The NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless the document requesting or requiring the collection displays a currently valid OMB control number.						
1. FACILITY NAM	1E	2. DOCKET	NUMBER	3. LER NUMBER					
Susquehanna Steam Electric Station Unit		05000-387		YEAR	SEQUENTIAL NUMBER	REV NO.			
1				2020	- 002	- 00			
CORREC	TIVE ACTIONS								

Corrective actions will be provided in the supplement to this LER.

COMPONENT FAILURE INFORMATION

Component failure information, as applicable, will be provided in the supplement to this LER.

PREVIOUS OCCURRENCES

Previous occurrences, as applicable, will be provided in the supplement to this LER.