

UNIT ED STATES NUCLEAR REGULATORY COMMISSION

# TENNESSEE VALLEY AUTHORITY

# DOCKET NO. 50-327

# SEQUOYAH NUCLEAR PLANT, UNIT 1

# AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 235 License No. DPR-77

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Tennessee Valley Authority (the licensee) dated February 13, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.



- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-77 is hereby amended to read as follows:
  - (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 235 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

In addition, paragraph 2.C.(19) of Facility Operating License No. DPR-77 is hereby amended to read as follows: \*

(19) Mechanical Snubbers

This condition is deleted

3. This license amendment is effective as of its date of issuance, to be implemented no later than 45 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Frederick J. Hebdon, Director Project Directorate II-3 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachments: 1. Page 8 of License 2. Changes to the Technical Specifications

Date of Issuance: August 28, 1998

\* Page 8 of the composite license is attached to reflect this change

#### (19) Mecchanical Snubbers

This condition is deleted.

## (20) Low Temperature Overpressure Protection (Section 5.2.2)

At the first outage of sufficent duration but no later than startup following the second refueling outage, TVA shall install an overpressure mitigation system which meets NRC requirements.

#### (21) Control Rod Guide Thimble (Section 4.2)

Prior to startup after first refueling, TVA shall submit the details of the inspection program for ccontrol rod guide thimble tube wall wear for NRC approval.

#### (22) TMI Action Plan Full Power Conditions

Each of the following conditions shall be completed to the satisfaction of the NRC by the times indicated:

#### A. Safety Engineering Group (Section 22.2.1.B.1.2)

Prior to exceeding five percent power, TVA is required to have an onsite Safety Engineering Group. NRC will verify the adequacy of the Safety Engineering Group and its independence.

## B. Short-Term Accident Analysis and Procedure Revision (Section 22.2.I.C.1)

Within thirty effective full-power days, TVA shall revise Emergency Operating Procedures and brief the operators on the revision.

## C. Control Room Design (Section 22.2.I.D.1)

TVA shall consider the benefits of installing data recording and logging equipment in the control room to correct the deficiencies associated with the trending of important parameters on strip chart recorders used in the control room as part of the Detailed Control Room Design Review. Implementation shall be carried out in accordance with SECY 82-111B.

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# ATTACHMENT TO LICENSE AMENDMENT NO. 235

# FACILITY OPERATING LICENSE NO. DPR-77

#### DOCKET NO. 50-327

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE	INSERT
Index Page IX	Index Page IX
Index Page XIV	Index Page XIV
3/4 7-21	3/4 7-21
3/4 7-22	-
3/4 7-23	-
3/4 7-24	-
3/4 7-25	-
B 3/4 7-5	B 3/4 7-5
B 3/4 7-6	B 3/4 7-6
B 3/4 7-7	B 3/4 7-7

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NOTE: Page 6-23 is also affected by the licensee's request; however, page 6-23 was removed in it's entirety by License Amendment 233, dated July 1, 1998.

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

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3/4.7.8	AUXILIARY BUILDING GAS TREATMENT SYSTEM	
3/4.7.9	SNUBBERS (DELETED)	
3/4.7.10	SEALED SOURCE CONTAMINATION	1
3/4.7.11	FIRE SUPPRESSION SYSTEMS (DELETED)	1
3/4.7.12	FIRE BARRIER PENETRATIONS (DELETED)	R231
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3/4.8.1	A.C. SOURCES	R65
	OPERATING	
	SHUTDOWN	
3/4.8.2	ONSITE POWER DISTRIBUTION SYSTEMS	
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	A.C. DISTRIBUTION - SHUTDOWN	
	D.C. DISTRIBUTION - OPERATING	
	D.C. DISTRIBUTION - SHUTDOWN	
3/4.8.3	ELECTRICAL EQUIPMENT PROTECTIVE DEVICES	
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SEQUOYAH - UNIT 1

Amendment No. 61, 664, 235

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3/4.9.1	BORON CONCENTRATION
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3/4.9.5	COMMUNICATIONS
3/4.9.6	MANIPULATOR CRANE
3/4.9.7	CRANE TRAVEL - SPENT FUEL PIT AREA (DELETED)
3/4.9.8	RESIDUAL HEAT REMOVAL AND COOLANT CIRCULATION
3/4.9.9	CONTAINMENT VENTILATION SYSTEM

SEQUOYAH - UNIT 1

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3/4.7.9 SNUBBERS

LIMITING CONDITION FOR OPERATION -

3.7.9. This specification is deleted.

(Pages 3/4 7-21 through 3/4 7-25 are deleted)

BASES

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## 3/4.7.8 AUXILIARY BUILDING GAS TREATMENT SYSTEM

The OPERABILITY of the auxiliary building gas treatment system ensures that radioactive materials leaking from the ECCS equipment following a LOCA are filtered prior to reaching the environment. The operation of this system and the resultant effect on offsite dosage calculations was assumed in the accident analyses. ANSI N510-1975 will be used as a procedural guide for surveillance testing. Cumulative operation of the system with the heaters on for 10 hours over a 31 day period is sufficient to reduce the buildup of moisture on the adsorbers and HEPA filters.

The minimum vacuum relief flow requirement in TS Surveillance Requirement 4.7.8.d.3 is for test purposes only. It is intended to demonstrate an acceptable level of ABGTS performance margin by simulating an ABSCE boundary breach. The inability to meet the specified minimum test condition under other circumstances does not challenge the operability of the ABGTS.

3/4.7.9 SNUBBEPS

This specification is deleted.

BR-6

PLANT SYSTEMS	
BASES	

This page is deleted

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#### BASES

# 3/4.7.10 SEALED SOURCE CONTAMINATION

The limitations on removable contamination for sources requiring leak testing, including alpha emitters, is based on 10 CFR 70.39(c) limits for plutonium. This limitation will ensure that leakage from byproduct, source, and special nuclear material sources will not exceed allowable intake values. Sealed sources are classified into three groups according to their use, with surveillance requirements commensurate with the probability of damage to a source in that group. Those sources which are frequently handled are required to be tested more often than those which are not. Sealed sources which are continuously enclosed within a shielded mechanism (i.e., sealed sources within radiation monitoring or boron measuring devices) are considered to be stored and need not be tested unless they are removed from the shielded mechanism.

## 3/4.7.11 FIRE SUPPRESSION SYSTEMS

This Specification is deleted.

R231

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#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

# TENNESSEE VALLEY AUTHORITY

# **DOCKET NO. 50-328**

## SEQUOYAH NUCLEAR PLANT, UNIT 2

## AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 225 License No. DPR-79

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Tennessee Valley Authority (the licensee) dated February 13, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I:
  - The facility will operate in conformity with the application, the provisions of the Act, B. and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-79 is hereby amended to read as follows:
  - (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 225, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance, to be implemented no later than 45 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Frederick J. Hebdon, Director Project Directorate II-3 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: August 28, 1998

# ATTACHMENT TO LICENSE AMENDMENT NO. 225

## FACILITY OPERATING LICENSE NO. DPR-79

#### DOCKET NO. 50-328

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

#### REMOVE

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#### INSERT

Index Page IX	Index Page IX
Index Page XIV	Index Page XI
3/4 7-21	3/4 7-21
3/4 7-22	-
3/4 7-23	
3/4 7-24	
3/4 7-25	-
B 3/4 7-5	B 3/4 7-5
B 3/4 7-6	B 3/4 7-6
B 3/4 7-7	B 3/4 7-6a

NOTE: Page 6-24 is also affected by the licensee's request; however, page 6-24 was removed in it's entirety by License Amendment 223, dated July 1, 1998.

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

#### SECTION

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3/4.7.8	AUXILIARY BUILDING GAS TREATMENT SYSTEM
3/4.7.9	SNUBBERS (DELETED)
3/4.7.10	SEALED SOURCE CONTAMINATION
3/4.7.11	FIRE SUPPRESSION SYSTEMS (DELETED)
3/4.7.12	FIRE BARRIER PENETRATIONS (DELETED)
3/4.8 ELEC	TRICAL POWER SYSTEMS
3/4.8.1	A.C. SOURCES
	Operating
	Shutdown
3/4.8.2	ONSITE POWER DISTRIBUTION SYSTEMS
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	A.C. Distribution - Shutdown
	D.C. Distribution - Operating
	D.C. Distribution - Shutdown

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Amendment No. \$18, 225

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## SECTION

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3/4.7.6	FLOOD PROTECTION
3/4.7.7	CONTROL ROOM EMERGENCY VENTILATION SYSTEM
3/4.7.8	AUXILIARY BUILDING GAS TREATMENT SYSTEM
3/4.7.9	SNUEBERS (DELETED)
3/4.7.10	SEALED SOURCE CONTAMINATION
3/4.7.11	FIRE SUPPRESSION SYSTEMS (DELETED)
3/4.7.12	FIRE BARRIER PENETRATIONS (DELETED)

# 3/4.8 ELECTRICAL POWER SYSTEMS

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		SYSTEMS .	• •	•	• •	•	•	•	•	• •		• •	•			•						B	3/4	8.	-1
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3/4.9.6	MANIPULATOR CRANE				в	3/4	9-2
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SEQUOYAH - UNIT 2 XIV Amendment No. 194, 218, 225

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3/4.7.9 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.9 This specification is deleted.

(Pages 3/4 7-21 through 3/4 7-25 are deleted)

SEQUOYAH - UNIT 2

3/4 7-21 Amendment Nos. 14, 143, 225

#### BASES

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# 3/4.7.8 AUXILIARY BUILDING GAS TREATMENT SYSTEM

The OPERABILITY of the auxiliary building gas treatment system ensures that radioactive materials leaking from the ECCS equipment following a LOCA are filtered prior to reaching the environment. The operation of this system and the resultant effect on offsite dosage calculations was assumed in the accident analyses. ANSI N510-1975 will be used as a procedural guide for surveillance testing. Cumulative operation of the system with the heaters on for 10 hours over a 31 day period is sufficient to reduce the buildup of moisture on the adsorbers and HEPA filters.

The minimum vacuum relief flow requirement in TS Surveillance Requirement 4.7.8.d.3 is for test purposes only. It is intended to demonstrate an acceptable level of ABGTS performance margin by simulating an ABSCE boundary breach. The inability to meet the specified minimum test condition under other circumstances does not challenge the operability of the ABGTS.

3/4.7.9 SNUBBERS

This specification is deleted.

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BASES

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This page is deleted

SEQUOYAH - UNIT 2

B 3/4 7-6 Amendment Nos. p. 14/3, 225

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#### 3/4.7.10 SEALED SOURCE CONTAMINATION

The limitations on removable contamination for sources requiring leak testing, including alpha emitters, based on 10 CFR 70.39(c) limits for plutonium. This limitation will ensure that leakage from byproduct, source, and special nuclear material sources will not exceed allowable intake values. Sealed sources are classified into three groups according to their use, with surveillance requirements commensurate with the probability of damage to a source in that group. Those sources which are frequently handled are required to be tested more often than those which are not. Sealed sources which are continuously enclosed within a shielded mechanism (i.e., sealed sources within radiation monitoring or boron measuring devices) are considered to be stored and need not be tested unless they are removed from the shielded mechanism.