Docket Nos.: 50-528, 50-529 and 50-530

LICENSEE: Arizona Public Service Company

FACILITY: Palo Verde, Units 1, 2 and 3

SUBJECT: SUMMARY OF MEETING HELD IN PHOENIX, ARIZONA, AUGUST 11 - 13, 1987

### 1. Management Meeting

A management meeting was held on August 12, 1987, in the licensee's office in Phoenix, Arizona, with representatives of the licensee and NRC representatives from the Offices of NRR and Region V. The purpose of the meeting was to discuss NRC concerns generated during assessment of the operating performance of Palo Verde Units 1 and 2 in consideration of the pending licensing decision for Unit 3, and to identify needed licensee actions for improvement prior to the licensing decision.

The list of attendees and slides used in the licensee's presentation are enclosed as Enclosures 1 and 2 respectively.

The licensee presented responses to the concerns identified by the NRC in the proposed agenda provided by letter dated July 22, 1987.

In the area of Compliance Trend Data, the licensee feels that downward trends have been established in the number of Licensee Event Reports, personnel errors, unplanned reactor trips, and ESF actuations. The staff commented that improvement in these areas had been noted, but in recent months a slight trend upward in several areas is occurring. Additional management emphasis is needed to ensure that a significant upward trend does not occur with the startup of Unit 3.

The licensee reviewed Fuel Performance figures for Units 1 and 2 and discussed actions taken over the past five years by the licensee and the fuel manufacturer to reduce fuel pin failures. The licensee has visited other utilities that have experienced fuel failures to discuss their problems and corrective actions. The licensee stated that with the actions taken Unit 3 will not be as susceptible to fuel failures, and fuel performance should improve.

The Unit 1 Circulatory Water (CW) system water hammer event was discussed. The root cause of the condenser waterbox damage was the failure of the quadrant gear bolts on the '2C' waterbox outlet valve. The bolts failed due to improper setting of the electrical limit switches and

mechanical stops caused by incorrect interpretation of the valve technical manual. The valve manufacturer provided training of licensee personnel and the electrical limit switches and mechanical stops were reset in Unit 1. The licensee stated that the use of locktite and hardened washers to distribute torque loading and an increase in preloading on the bolts should reduce the vibration induced fatigue loading on the bolts. Similar actions will be taken on Unit 2 CW valves at the next outage and on Unit 3 CW valves prior to power ascension.

In the area of Secondary Water Chemistry Control the licensee discussed activities in progress to reduce radwaste generation and to enhance radwaste processing capability when primary to secondary leakage occurs.

The Unit 1 LPSI pump seal failures of early July 1987 were discussed. When the Unit remained at approximately 325°F for steam generator cleanup following the condenser water hammer event, the continued operation at high temperature caused degradation of the seals. The swelling of the 'O' rings was apparently due to the application of a solvent to the 'O' ring surface. On the 'A' LPSI pump the increased seal leakage up the pump shaft displaced bearing lubricating oil and resulted in bearing failure and motor damage. As corrective actions slingers are being installed on all LPSI and CS pumps, no solvent or petroleum lubricants are to be used on ' rings, operating time above 210°F is being restricted on LPSI pumps, and a seal life monitoring program by the system engineer has been established. The licensee is looking at different seal designs and/or materiais. The staff commented that a common theme in several recent events has been that technical manuals were not specific. Increased interaction and information feedback between vendors and the licensee, and vendor involvement in maintenance activities might preclude similar problems in the future. The staff also recommended that information about seal degradation at high temperature should be shared with the rest of the industry.

Other topics of discussion included Conduct of Radiation Protection Surveillances, System Engineer Involvement in Maintenance and Modification activities, Status of Unit 3B Emergency Diesel Generator, Operator Confidence in Control Room Indications, Unit 3 Schedule, Maintenance Backlog, Annunicator Status, Simulator Time Goals for Operator Retraining, and the licensee's proposed Organizational Changes.

The staff briefly discussed the issuance of Commission Paper 87-188 on New Plant Operating experience that addresses a number of initiatives to improve new plant performance. The staff will formally transmit a copy of this document to the licensee to obtain their response.

The NRC staff appreciated the presentation and will watch for evidence of success of corrective actions in improving performance trends.

### 2. Unit 3 Plant Tour

On August 11, 1987, the staff members toured Unit 3 and talked with plant personnel to assess Unit readiness for initial criticality. The licensee expects to be ready for startup in early October, and this appears to the staff to be reasonable.

15/

Michael J. Davis, Project Manager Project Directorate V Division of Reactor Projects - III, IV, V and Special Projects

Enclosures: As stated

cc: See next page

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DRSP/PDV MGS MDavis:ca 8/24/87 DRSP/D/PDV GWKM ightor 8/2 Y/87 Mr. E. E. Van Brunt, Jr. Arizona Nuclear Power Project

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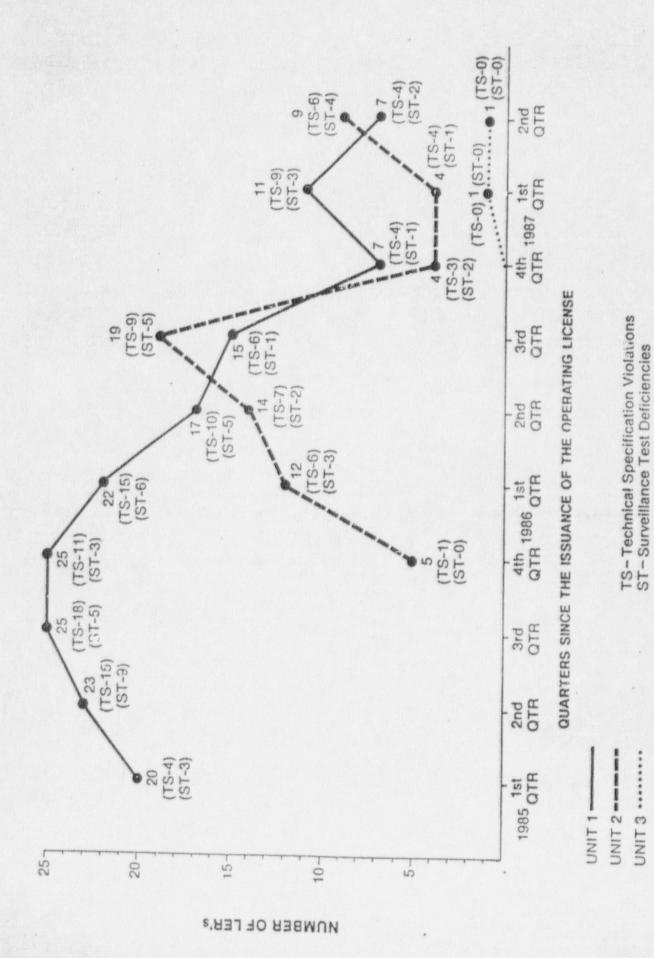
ANPP - NRC MANAGEMENT MEETING PHOENIX, ARIZONA AUGUST 12, 1987

### AGENDA ANPP - NRC MANAGEMENT MEETING AUGUST 12, 1987

	DOCCENTED
1. OPENING REMARKS	PRESENTER
- AVPP	E. E. VAN BRUNT
- NRC	NRC NRC
	INIC
11. OVERALL PLANT PERFORMANCE	
- COMPLIANCE TREND DATA	J. R. BYNUM
- FUEL INTEGRITY	P. F. CRAWLEY
	···· OVALLI
111. UPDATE ON RECENT EVENTS	
- CONDENSER WATER HAMMER	W. M. SIMKO
- SECONDARY WATER CHEMISTRY CONTROL	J. R. MANN
- UNIT 1 LPSI PUMPS	W. M. SIMKO
IV. UPDATE ON PREVIOUS ISSUES	
- CONDUCT OF RP SURVEILLANCES	J. R. BYNUM
- SYSTEM ENGINEER INVOLVEMENT IN	O. J. ZERINGLE
MAINTENANCE AND MODIFICATIONS	
- OPERATOR CONFIDENCE IN CONTROL	O. J. ZERINGUE
ROOM INDICATIONS	
V. PRINCIPLE PROBLEMS ENCOUNTERED DURING	
UNIT 3 TESTING	
- EMERGENCY DIESEL GENERATOR B	
B ENGENOT BIESEL GENERATOR B	O. J. ZERINGUE
VI. UNIT 3 LOW POWER TESTING PROGRAM	
	J. R. BYNUM
VII. ADDITIONAL ITEMS	
- MAINTENANCE BACKLOG	J. R. BYNUM
- AVNUNCIATOR IMPROVEMENT PROGRAM	J. R. BYNUM
- SIMULATOR TIME	W. F. FERNOW .
111. ANPP ORGANIZATION	E. E. VAN BRUNT/
	J. G. HAYNES

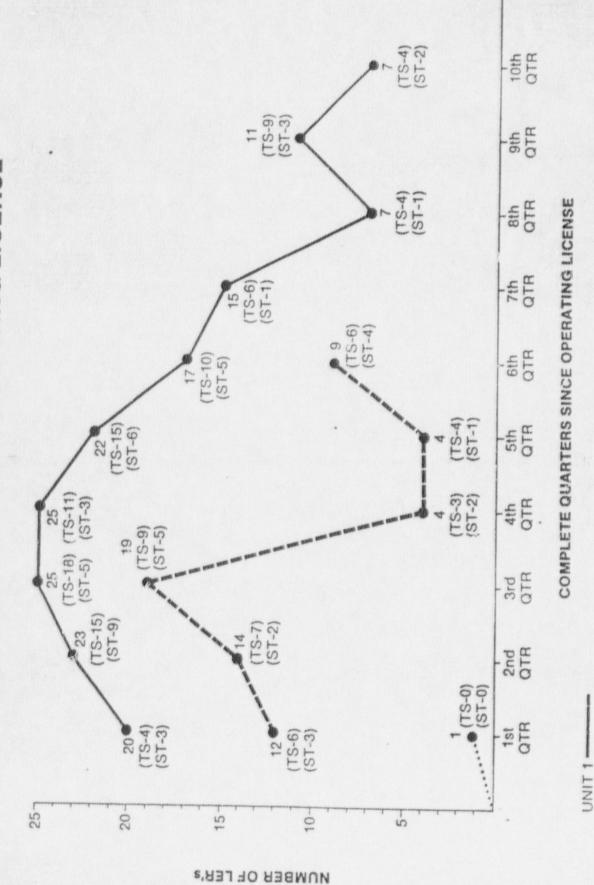
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UNIT 3 .....

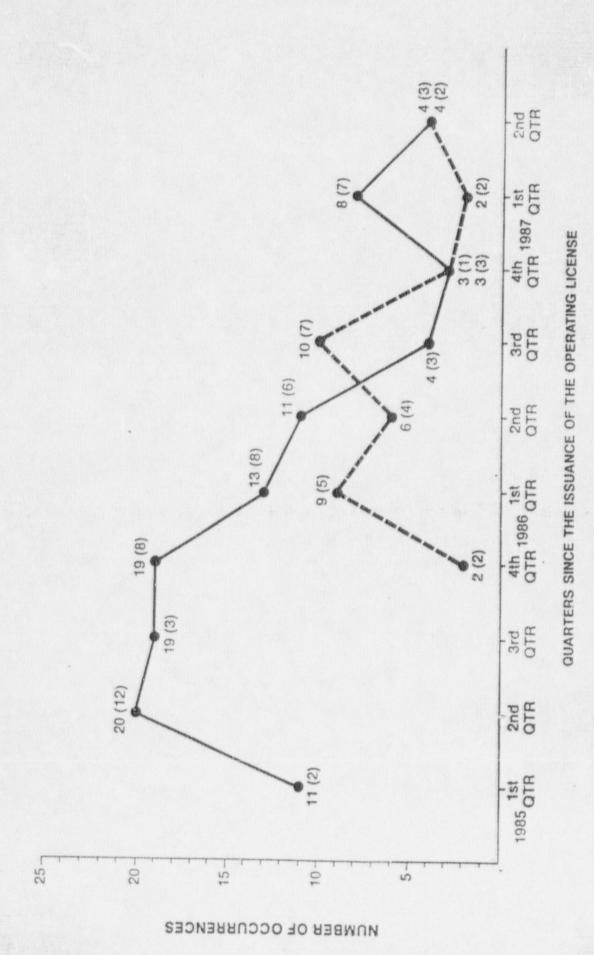
### SINCE ISSUANCE OF OPERATING LICENSE LER'S PER COMPLETE QUARTER **UNIT COMPARISON**



TS - Technical Specification Violations ST - Surveillance Test Deficiencies

UNIT 2 mm mm mass

UNIT 3

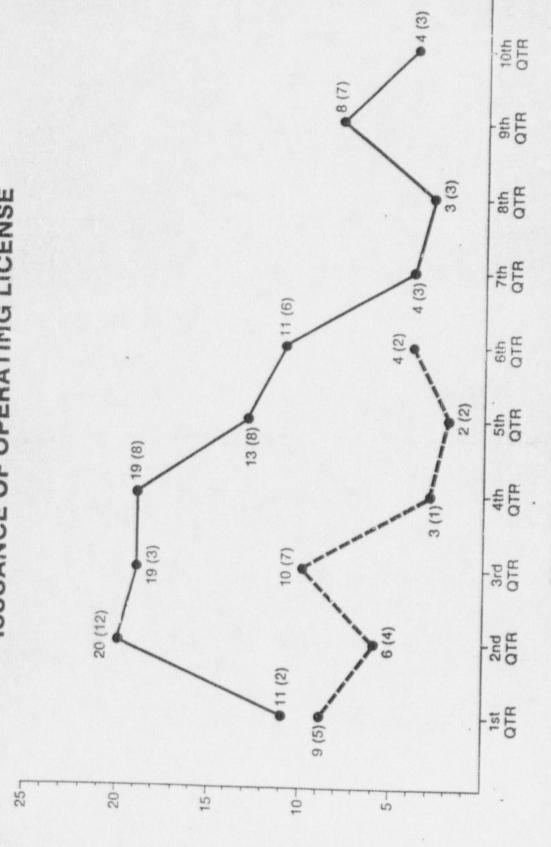


) Procedure Violations

UNIT 2 messes messes

UNIT 1

# OCCURRENCES INVOLVING PERSONNEL PER COMPLETE QUARTER SINCE ISSUANCE OF OPERATING LICENSE



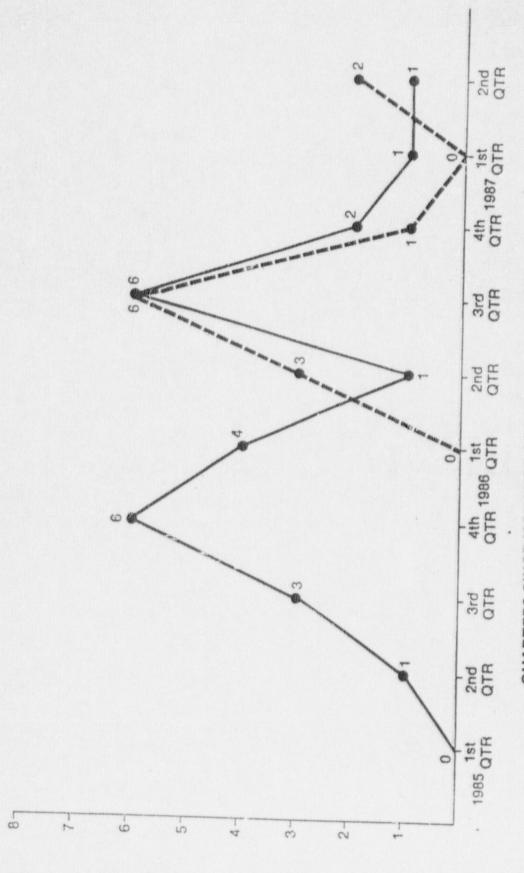
COMPLETE QUARTERS SINCE OPERATING LICENSE

( ) Procedure Violations

UNIT 2 ----

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### UNPLANNED REACTOR TRIPS PER UNIT



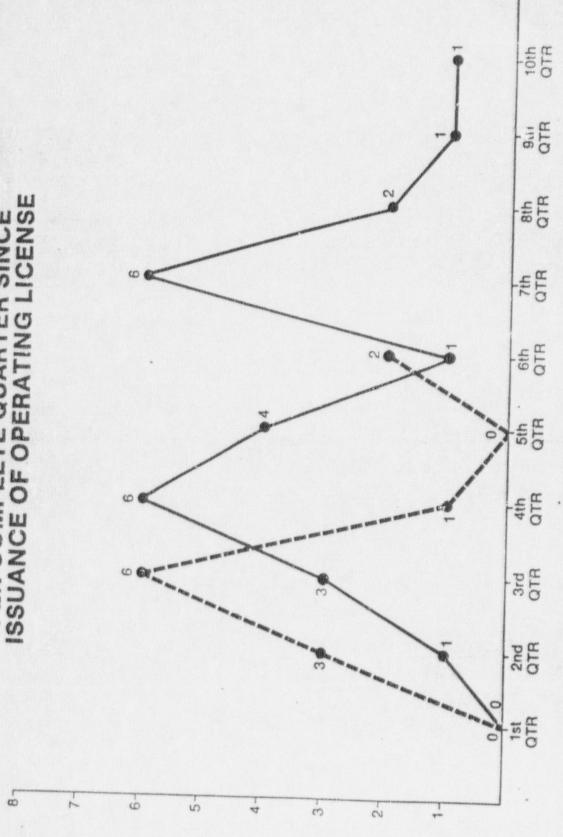
NUMBER OF REACTOR TRIPS

QUARTERS SINCE ISSUANCE OF THE OPERATING LICENSE

UNIT 2

NUMBER OF REACTOR TRIPS

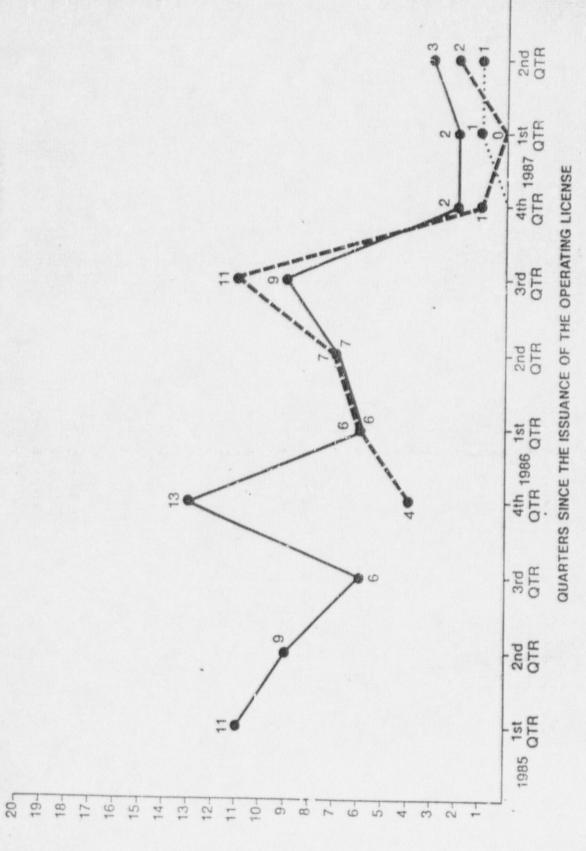




COMPLETE QUARTERS SINCE OPERATING LICENSE

UNIT 2

## (Includes RPS Actuations)

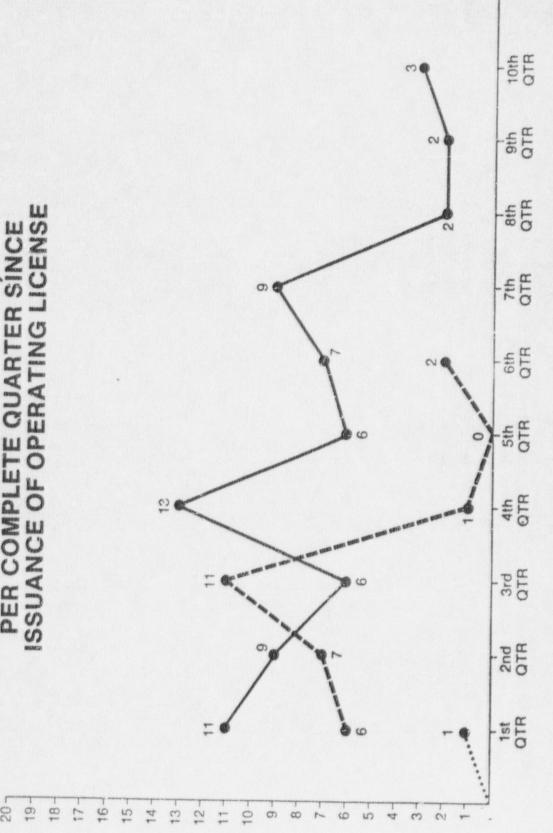


UNIT 2 ----

### NUMBER OF ACTUATION REPORTS

TOTAL ESF EVENT REPORTS

(Includes RPS Actuations)
PER COMPLETE QUARTER SINCE



COMPLETE QUARTERS SINCE OPERATING LICENSE



### UNIT 1 FUEL PERFORMANCE

### FUEL STATUS

- ESTIMATED NUMBER OF FAILED FUEL PINS 5-10
- COOL ANT ACTIVITY IODINE-131 0.025 UCI/ML

0.050 UCI/ML

- 105INE-131 DOSE EQUIVALENT
- · FUEL INSPECTION AT REFUELING
  - VISUAL INSPECTION
  - GUIDE TUBE EDDY CURRENT EXAMINATION
  - SHOULDER GAP MEASUREMENT
  - CLAD OXIDE LAYER THICKNESS MEASUREMENT
- ADDITIONAL FUEL OPERATIONAL PRACTICES
  - GUIDELINES HAVE BEEN ISSUED TO PLANT OPERATIONS LIMITING THE RATE OF PLANT POWER INCREASE TO 10% PER HOUR COMPARED TO THE CE CORE OPERATING GUIDELINES LIMIT OF 30% PER HOL
  - DAILY MONITURING OF REACTOR COOLANT SYSTEM ACTIVITY WITH REPORTS TO ANPP MANAGEMENT WEEKLY.

### UNIT 2 FUEL PERFORMANCE

### FUEL STATUS

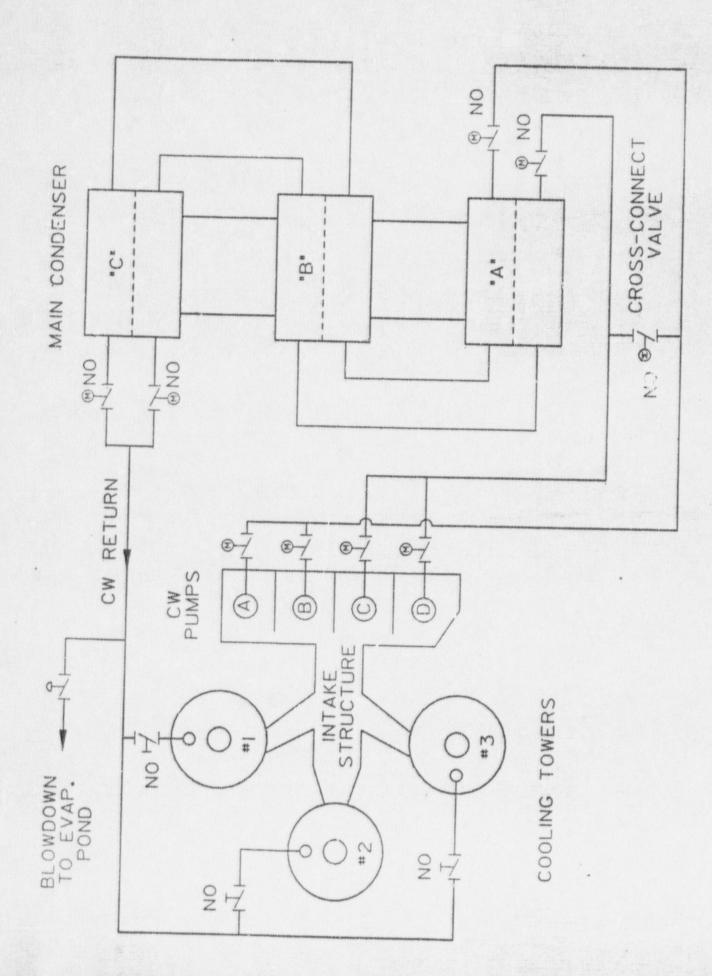
- ESTIMATED NUMBER OF FAILED FUEL PINS 25 35 - COOLANT ACTIVITY - IODINE-131 0.15 UCI/ML
- IODINE-131 DOSE EQUIVALENT 0.22 UCI/ML

### FUEL INSPECTION AT REFUELING

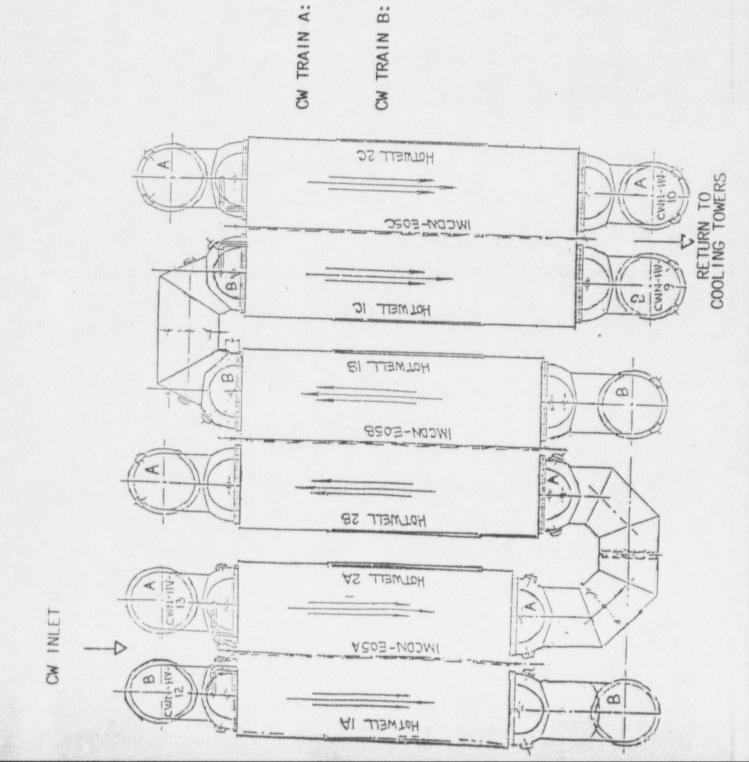
- VISUAL INSPECTION
- GUIDE TUBE EDDY CURRENT EXAMINATION
- SHOULDER GAP MEASUREMENT

### ADDITIONAL FUEL OPERATIONAL PRACTICES

- GUIDELINES HAVE BEEN ISSUED TO PLANT OPERATIONS LIMITING
  THE RATE OF PLANT POWER INCREASE TO 10% PER HOUR COMPARED
  TO THE CE CORE OPERATING GUIDELINES LIMIT OF 30% PER
  HOUR
- DAILY MONITORING OF REACTOR COOLANT SYSTEM ACTIVITY WITH REPORTS TO ANPP MANAGEMENT WEEKLY



CIRCULATING WATER SYSTEM



2C

CONSISTS OF COPUMPS C & D
PASSES THROUGHOTWELLS 1A,

CONSISTS OF CW PUMPS A & B PASSES THROUGH HOTWELLS 2A, 2B,

### WATER SYSTEM EVENT

### DESCRIPTION OF EVENT:

- TRAIN 'A' OF CW SYSTEM EXPERIENCES PIPE WELD FAILURES PLANT IS SHUTDOWN.
- EVENT IS CAUSED BY HOTWELL '2C' OUTLET VALVE CLOSING.
- WELDS ON WATERBOX '2C' INLET AND OUTLET FAILED WATERBOX '2B' OUTLET AND '2A' INLET CRACKED.
- EXPANSION JOINT STUDS BENT, STRETCHED AND DAMAGED ON BOTH LOOPS.

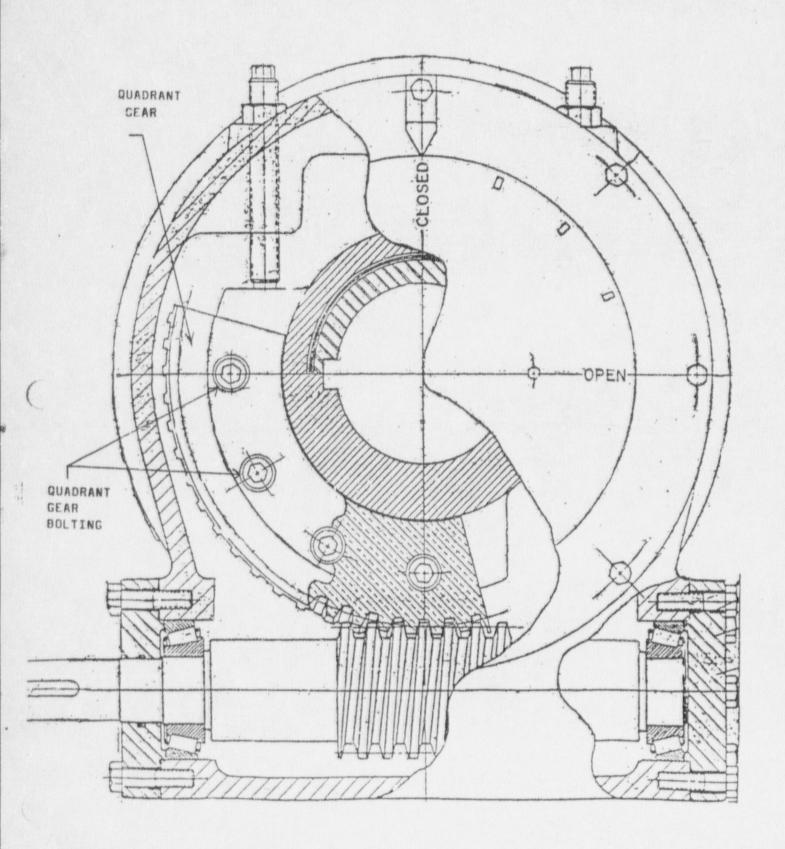
### UNIT 1 CIRCULATING WATER SYSTEM EVENT (CONT)

### ROOT CAUSE (MULTIPLE):

- ROOT CAUSE OF THE WATERBOX DAMAGE WAS FAILURE OF THE QUADRANT GEAR BOLTS
- ROOT CAUSE OF THE BOLT FAILURE WAS IMPROPER SETTING OF THE ELECTRICAL AND MECHANICAL STOPS
- ROOT CAUSE OF THE IMPROPERLY SET STOPS WAS INADEQUATE/INCORRECT INTERPRETATION OF THE TECHNICAL MANUAL.

### CORRECTIVE ACTIONS:

- ELECTRICAL LIMIT SWITCHES AND MECHANICAL STOPS
  WERE RESET IN UNIT 1
- '2C' OUTLET VALVE REPAIRED. BOLTING MATERIAL REPLACED. LOCKTITE AND HARDENED WASHERS ADDED TO DISTRIBUTE TORQUE LOADING AND ASSIST IN MAINTAINING TORQUE LOADING.
- ALL OTHER SIMILAR VALVES IN UNIT 1 CHECKED FOR WEAR,
  TORQUE, AND WASHERS AND LOCKTITE USED AS DESCRIBED ABOVE.
- BOLTS IN UNITS 2 AND 3 CHECKED TO VERIFY TIGHTNESS
- WATERBOX REPAIRS COMPLETED
- ONE TUBE LEAK IDENTIFIED AND PLUGGED
- NO INDICATIONS OF TUBE SHEET LEAKS UBSERVED



### SECONDARY WATER CHEMISTRY CONTROL

### BACKGROUND

- TECHNICAL SPECIFICATION LIMITS AND BASIS FOR RADIOACTIVITY DISCHARGES TO EVAPORATION PONDS
- OPERATION OF SECONDARY AND CIRCULATING WATER SYSTEMS

### RECENT OPERATING HISTORY

- HIGH ANTIMONY ACTIVITY LEVELS IN RCS (UNITS 1 AND 2)
- THE PRIMARY SOURCE OF ANTIMONY IS THE RCP JOURNAL BEARINGS
- ANTIMONY ACTIVITY TRANSFERRED TO SECONDARY SIDE DUE TO UNIT 1 STEAM GENERATOR TUBE LEAK IN JANUARY, 1987

### ACTIVITIES IN PROGRESS

- CONTINUE INVESTIGATING MEANS TO INCREASE RADWASTE PROCESSING CAPABILITY
- CONTINUE INVESTIGATING MEANS TO OPERATE WITH MINUTE PRIMARY TO SECONDARY LEAK RATES

PVNGS LIMIT FOR DISCHARGES
BY NRC
LIMIT REGULATIONS

MOST ISOTOPES

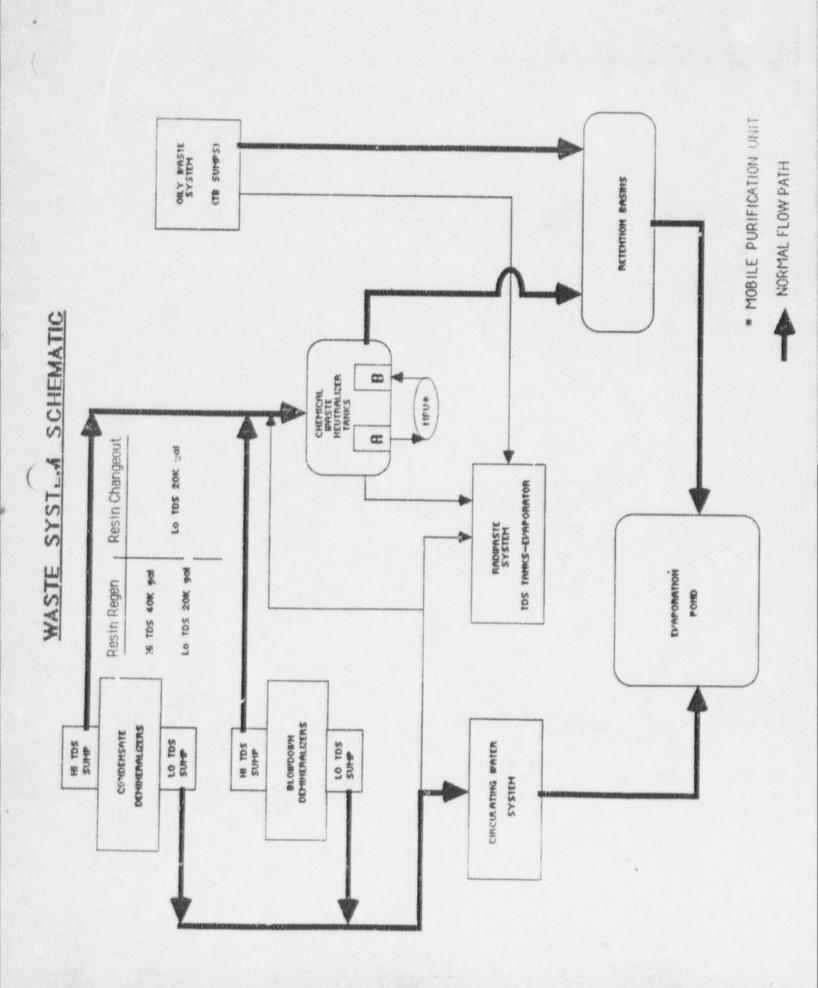
5X10<sup>-7</sup> UCI/ML

1x10<sup>-7</sup> TO 1x10<sup>-4</sup>

ANTIMONY

2x10<sup>-5</sup> UCL/ML

2x10<sup>-5</sup>



### UNIT 1 LPSI SEAL FAILURE

- JUNE 29. 1987 ENTERED MODE 4 TO REPAIR CIRCULATING WATER SYSTEM
- JUNE 30, 1987 PLACED SHUTDOWN COOLING (SDC) IN SERVICE
  ON 'B' LPSI PUMP RX. COOLANT 325°F-MAINTAIN
  TEMPERATURE/PRESSURE FOR SG CHEMISTRY CLEANUP
- JULY 1, 1987 RCS LEAKAGE INCREASED TO 2.5 GPM (ESTIMATED 1.6 GPM FROM 'B' LPSI) PLACED 'A' TRAIN SDC IN SERVICE PER NORMAL OPERATING PROCEDURE. STOPPED 'B' LPSI
- JULY 3, 1987 TOTAL RCS LEAKAGE AT 4.7 GPM 4.0 GPM
  LEAKAGE FROM 'A' AND 'B' LPSI PUMPS-CONTINUED
  OPERATION ON 'A' LPSI PUMP BASED ON:
  - 1) LEAKAGE . TE CONSTANT
  - 2) EXPECTED MODE 3 ENTRY WITHIN 24 HOURS
  - 3) MOTORS QUALIFIED FOR 100% HUMIDITY
- JULY 4, 1987 LOSS OF 'A' LPSI MOTOR DUE TO GROUND FAULT
   RESTARTED 'B' LPSI COMMENCED COOLDOWN
  OF RCS TO LESS THAN 210° TO ENTER MODE 5
  AND PLACED CONTAINMENT SPRAY PUMP IN SERVICE
- JULY 5, 1987 CONCLUDED LOWER MOTOR BEARING FAILURE CAUSED

  BY WATER SPRAY UP THE SHAFT FROM THE

  MECHANICAL SEAL. NEED TO PLACE SLINGER RING

  ON PUMP SHAFTS TO STOP WATER LEAK PATH

### LPSI A - ROOT CAUSE ANALYSIS

### SEAL FAILURE

- VENDOR INVOLVEMENT (PUMP VENDOR AND SEAL VENDOR)
- SWOLLEN 'O' RINGS APPARENTLY DUE TO SOLVENT APPLICATION
- LOWER (ROTATING) SEAL SEIZES AXIALLY ON SHAFT PREVENTING WEAR COMPENSATION
- HIGH TEMPERATURE ACCELERATES WEAR RATE

### MOTOR FAILURE

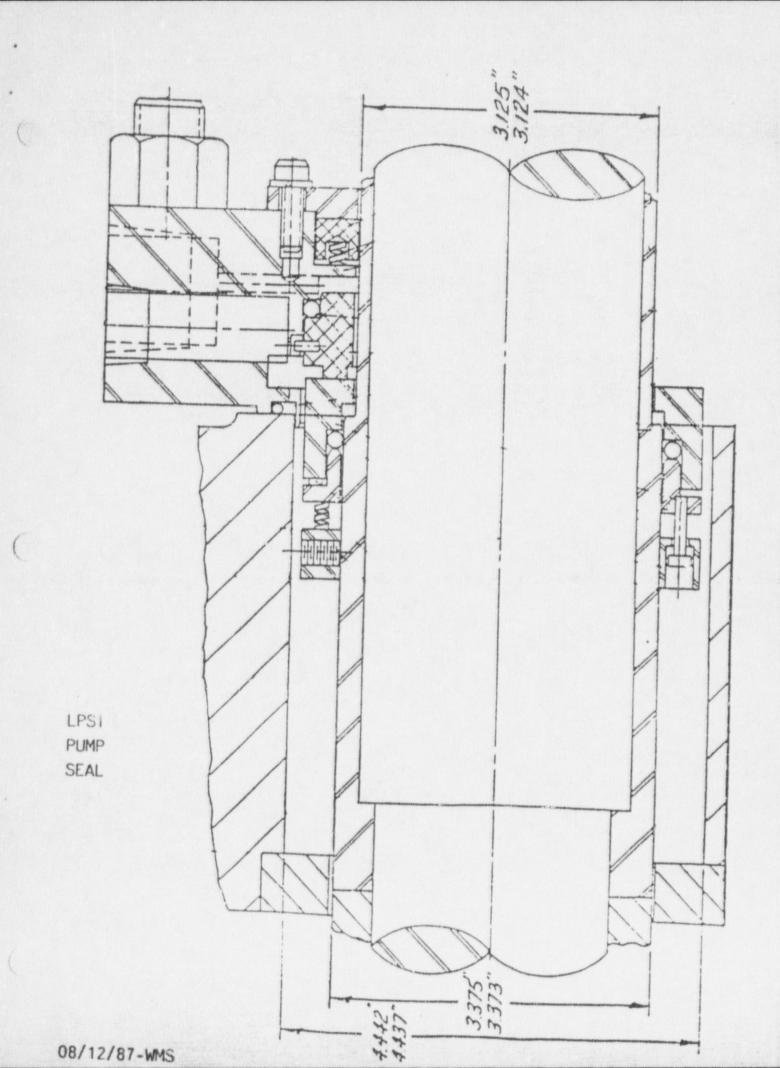
- VENDOR INVOLVEMENT
- SEAL LEAKAGE UP PUMP SHAFT TO LOWER MOTOR SEARING
- BEARING OIL DISPLACED BY SEAL LEAKAGE WATER
- BEARING FAILURE
- MOTOR UPLIFT & DAMAGE

### LPSI B - ROOT CAUSE ANALYSIS

- " VENDOR INVOLVEMENT (PUMP VENDOR AND SEAL VENDOR)
- \* DEBRIS IN ROTATING SEAL CAUSES MISALIGNMENT OF SEAL FACES
- "BEVELED" WEAR PATTERN ON STATIONARY CARBON SEAL FACE -SEALING SURFACE REDUCED
- " HIGH TEMPERATURE ACCELERATES WEAR RATE
- CHIPPING ON CARBON SEALING FACE
- \* SEAL LEAKAGE UP PUMP SHAFT TO LOWER MOTOR BEARING

### CORRECTIVE ACTIONS

- 1. SLINGERS ON ALL LPSI AND CONTAINMENT SPRAY PUMPS
- 2. NO SOLVENT OR PETROLEUM LUBRICANTS ON O-RING/RUBBER MATERIAL
- 3. LIMIT OPERATION TIME ON LPSI PUMPS WHEN ABOVE 210°F
- 4. SEAL LIFE MONITORING BY SYSTEM ENGINEER



### CONDUCT OF RP SURVEILLANCES

\* APRIL 1982 - FALSIFICATION OF ELECTRICAL TERMINATION CARDS BY BECHTEL CONSTRUCTION PERSONNEL

TURLEY TAPE 1 - ATTENTION TO DETAIL INDIVIDUAL
RESPONSIBILITY FOR COMPLETE AND PROPER DOCUMENTATION

- \* NOVEMBER 1984 TURLEY TAPE II FITNESS FOR DUTY VERBATIM COMPLIANCE WITH PROCEDURES
- \* APRIL 1985 VAN BRUNT TAPE I SAFETY AND QUALITY ATTENTION TO DETAIL KNOW AND FOLLOW PROCEDURES
- " JANUARY 1986 PUBLISHED ARTICLE IN 'REACTOR' DEALING WITH SEABROOK EVENT
- \* FEBRUARY 1986 QUALITY TALKS FRAUD AND FALSIFICATION INDIVIDUAL RESPONSIBILITIES
- \* MARCH 1986 QUALITY TALKS NRC PERSPECTIVE OF FALSIFICATION DISCUSSED SEABROOK EVENT
- \* DECEMBER 1986 RADIATION PROTECTION TECHNICIANS FALSIFICATION OF SURVEILLANCE DATA
  - SAMPLED RP, FIRE PROTECTION, OPERATOR LOGS AND COMPARED AGAINST ACAD TRANSACTIONS
  - PUBLISHED ARTICLE IN 'REACTOR' THIS EVENT AND SEABROOK EVENT
  - QUALITY TALKS ISSUED ON SEABROOK EVENT EMPHASIZING INDIVIDUAL RESPONSIBILITY
  - PERSONNEL DEPARTMENT EVALUATION CONDUCTED TO DETERMINE
    IF HUMAN FACTORS SUCH AS MORALE, WORKING CONDITIONS COULD
    BE CONTRIBUTORY CAUSE

### CONDUCT OF RP SURVEILLANCES

- MAY 1987 SECURITY GUARD FALSIFICATION OF ROUNDS
  - REVIEWED SIGNIFICANT SAMPLE OF SECURITY LOGS AND ACAD TRANSACTIONS
- \* MAY 1987 RADIATION PROTECTION TECHNICIAN FALSIFIED SAMPLE FLUW VERIFICATION
  - VAN BRUNT LETTER REITERATING FALSIFICATION SERIOUSNESS
  - REVIEWED SAMPLE OF RP LOGS AND ACAD TRANSACTIONS
  - EVALUATION BY COMPLIANCE OF OTHER ACTIONS TO MINIMIZE FALSTFICATIONS
- \* CORRECTIVE ACTIONS TAKEN FOR SPECIFIC EVENTS WERE APPROPRIATE
   ALL PERSONNEL RECOGNIZE SERIOUSNESS OF FALSIFICATION

### SYSTEM ENGINEER INVOLVEMENT IN MAINTENANCE AND MODIFICATION ACTIVITIES

### \* DESCRIPTION OF EVENT:

- ASME SECTION XI TESTING WAS CONDUCTED ON THREE VALVES IN NOVEMBER, 1986.
- THE VALVES MET THE ACCEPTANCE CRITERIA HOWEVER THE STROKE TIMES HAD INCREASED BY MORE THAN 50% FROM THE PREVIOUS TESTS.
- WHEN STROKE TIMES INCREASE BY 50% OR MORE RELATIVE TO PREVIOUS TESTS. SECTION XI REQUIRES TESTING FREQUENCY TO BE INCREASED TO A MONTHLY INTERVAL.
- IN JANUARY, 1987, THE MODIFIED SURVEILLANCE INTERVAL WAS EXCEEDED.
- THE VALVES WERE SATISFACTORILY TESTED ON MARCH 6, 1987 AS PART OF THE REGULARLY SCHEDULED SURVEILLANCE TEST.
- ON MARCH 31, 1987 THE ERROR WAS DISCOVERED AND THE VALVES
  WERE ADDED TO THE MONTHLY TESTING SCHEDULE AND SATISFACTORILY
  TESTED ON APRIL 2, 1987.

### AND MODIFICATION ACTIVITIES (CONT)

### EVALUATION OF EVENT:

- THE CAUSE OF THE EVENT WAS A PERSONNEL ERROR ON THE PART OF THE ENGINEER RESPONSIBLE FOR THE SECTION XI TESTS.

  THE ENGINEER DID NOT EVALUATE THE STROKE TIMES WITHIN SUFFICIENT TIME TO MODIFY THE TESTING INTERVAL.
- THAT THESE THREE VALVES WERE THE ONLY VALVES TESTED LATE.
- ANPP IS CURRENTLY EVALUATING PROPOSED CHANGES TO THE ROLE
  OF THE SYSTEM ENGINEER IN THE PREPARATION AND CONDUCT
  OF SURVEILLANCE TESTING.
- REVIEW OF SURVEILLANCE TEST PROCEDURES.
- REVIEW OF SURVEILLANCE TEST RESULTS TO EVALUATE SYSTEM PERFORMANCE.
- REVIEW OF PREVENTATIVE MAINTENANCE PROCEDURES AND PERFORMANCE FREQUENCY.
- REVIEW OF PREVENTATIVE MAINTENANCE RESULTS.

### OPERATOR CONFIDENCE IN CONTROL ROOM INDICATIONS

### DESCRIPTION OF EVENT:

- ON JUNE 10, 1987, A S/G COLD LEG BLOWDOWN SAMPLE
  ISOLATION VALVE WAS STROKED CLOSED TO PERFORM A STROKE
  TIME SURVEILLANCE TEST
- THE "OPEN" INDICATION LIGHT EXTINGUISHED BUT NO "CLOSED" INDICATION WAS RECEIVED
- THE TEST WAS DISCONTINUED AND A WORK REQUEST WAS GENERATED TO TROUBLESHOOT THE INDICATION PROBLEM. HOWEVER, BASED ON PREVIOUS PROBLEMS EXPERIENCED WITH THIS TYPE OF VALVE'S POSITION INDICATION SWITCHES, THE VALVE WAS NOT DECLARED INOPERABLE
- ON JUNE 15, 1987, THE VALVE WAS STROKED TESTED AGAIN.
  HOWEVER, DURING THIS TEST THE "OPEN" INDICATION LIGHT
  DID NOT EXTINGUISH
- THE OPERATING CREW CHECKED THAT THE VALVE DID NOT MOVE.

  DECLARED THE VALVE INOPERABLE. AND CLOSED AND REMOVED

  POWER FROM A SECOND ISOLATION VALVE IN THE SAMPLE

  LINE IN ACCORDANCE WITH THE APPLICABLE ACTION STATEMENT
- SUBSEQUENT INVESTIGATION REVEALED THAT A SPRING, WHICH ACTS ON THE VALVE STEM TO CLOSE THE VALVE WHEN POWER IS REMOVED, WAS BROKEN

# OPERATOR CONFIDENCE IN CONTROL ROOM INDICATIONS (CONTINUED)

## EVALUATION OF EVENT:

- THE VALVE SHOULD HAVE BEEN DECLARED INOPERABLE ON JUNE 10. SINCE THE VALVE WAS NOT VERIFIED FULLY CLOSED
- SUBSEQUENT INVESTIGATION OF THE EVENT SHOWED COMPLIANCE WITH THE ACTION STATEMENT SINCE A MANUAL ISOLATION VALVE IN THE BLOWDOWN LINE WAS CLOSED AT ALL TIMES DURING THE "INOPERABILITY" OF THE VALVE. HOWEVER, THIS WAS NOT CONSIDERED AT THE TIME OF THE EVENT.
- MAINTAINING THE VALVE OPERABLE, WITHOUT POSITIVE

  VERIFICATION OF VALVE POSITION, AFTER CONTROL ROOM

  INDICATION SHOWED THE VALVE DID NOT CLOSE IS CONTRARY

  TO ANPP OPERATING PHILOSOPHY

## CORRECTIVE ACTION

- LETTER WAS ISSUED TO UNIT 2 LICENSED PERSONNEL (COPIES
  TO UNIT 1 AND 3) BY THE UNIT 2 OPERATIONS SUPERINTENDENT
  TO REINFORCE THE RESPONSIBILITIES WITH REGARD TO
  DETERMINATION OF OPERABILITY OF TECHNICAL SPECIFICATION
  EQUIPMENT
- THE RESPONSIBLE OPERATOR AND THE UNIT SUPERINTENDENTS WERE COUNSELED

## DIESEL GENERATOR "B" - CURRENT STATUS

- NRC APPROVED SPECIAL TEST PROGRAM COMPLETED JULY 5
- INSPECTION OF #2 MAIN BEARING AND #9 ROD BEARING BY NRC, COOPER, CLEVITE (BEARING MANUFACTURER), AND ANPP COMPLETED JULY 10. BEARINGS WERE DETERMINED TO BE ACCEPTABLE
- DURING INSPECTION OF THE GENERATOR ON JULY 15, A LOOSE POLE WINDING WAS DISCOVERED
- PREVIOUS FAILURE: IN NOVEMBER, 1986, PRIOR TO INITIAL ENERGIZATION, SEPARATED WINDING WAS DISCOVERED ON THE #12 POLE PIECE. POLE PIECE WAS SHIPPED TO THE GENERATOR MANUFACTURER (PARSON-PEEBLES) TO DETERMINE THE ROOT CAUSE OF THE FAILURE. ROOT CAUSE WAS EVALUATED TO BE LOCAL CONTAMINATION OF THE RESIN FROM CLEANING SOLVENT ON THE BRUSH USED TO APPLY THE RESIN.

#### DEDUCTIVE EVALUATION:

- AMOUNT OF RESIN OK
- QUALITY OF RESIN OK
- RESIN CORE OK
- INSPECTION OF UNIT 1, UNIT 2 AND UNIT 3 'A' DIESEL GENERATOR POLES WAS CONDUCTED NO FAILURES
- FOLLOWING THE JULY 15TH FAILURE:
  - DAMAGED POLE REMOVED
  - NATIONAL ELECTRIC COIL WAS CONTACTED TO PERFORM ROOT CAUSE ANALYSIS (IN PROGRESS) INCLUDING DESIGN EVALUATION OF FORCES ON POLE PIECE

## DIESEL GENERATOR 'B' - CURRENT STATUS (CONT)

- 'B' GENERATOR ROTOR SHIPPED TO WESTINGHOUSE FOR REQUALIFICATION TESTING
  - X AUG 1 125% OVERSPEED TEST (COLD)
  - X AUG 2 172% OVERSPEED TEST AT 130°C
- TESTING SUCCESSFULLY COMPLETED
- FINAL ROOT CAUSE NOT COMPLETE: PRELIMINARY RESULTS INDICATE
  THAT NOT ENOUGH RESIN, WHICH PROVIDES THE BONDING THAT HOLDS
  WINDING TO POLE, WAS APPLIED. QUALITY OF RESIN APPEARED GOOD.
  RESIN SUITABLE FOR THIS APPLICATION
- CONSULTANT IS BEING SOUGHT TO EVALUATE THE CORRELATION, IF ANY, OF THE ENGINE FAILURE OF 12/23 TO THE POLE WINDING FAILURE OF JULY 15
- WORKING WITH THE VENDOR TO DETERMINE AN APPROPRIATE INSPECTION FREQUENCY OF THE POLES

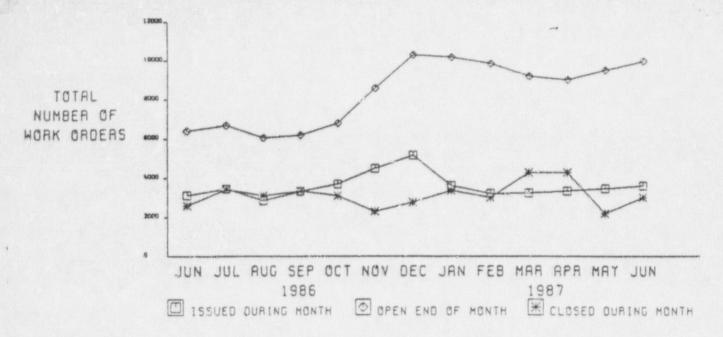
# STATUS OF UNIT 3 LOW POWER TESTING PROGRAM

- " MAJOR TEST PHASES
  - 1. POST-CORE HOT FUNCTIONAL TESTING (HFT)
    - PERFORM R.G. 1.68 TESTING AT SPECIFIED TEMPERATURE/ PRESSURE CONDITIONS BETWEEN MODE 5 (COLD SHUTDOWN-210°F) AND HOT ZERO POWER (565°F/2250 PSIA)
  - 2. INITIAL CRITICALITY/LOW POWER PHYSICS TESTING (IC/LPPT)
    - \* PERFORM R.G. 1.68 TESTING BY TAKING THE REACTOR CRITICAL AND VERIFYING LOW POWER REACTOR PHYSICS PREDICTIONS.
- ° STATUS
  - 1. SCHEDULE
  - 2. PERSONNEL

## PERSONNEL STATUS

BASICALLY SAME PERSONNEL INVOLVED WITH UNIT 1 AND 2
HFT/IC/LPPT THEREFORE STRONG EXPERIENCE LEVEL EXISTS

#### MAINTENANCE BACKLOG



EXPLANATION

TOTALS INCLUDE WORKTYPE OTHER THAN PM AND CM WORKORDERS TO MAINTENANCE ENGINEERING.

DATA PROVIDED BY MAINTENANCE.

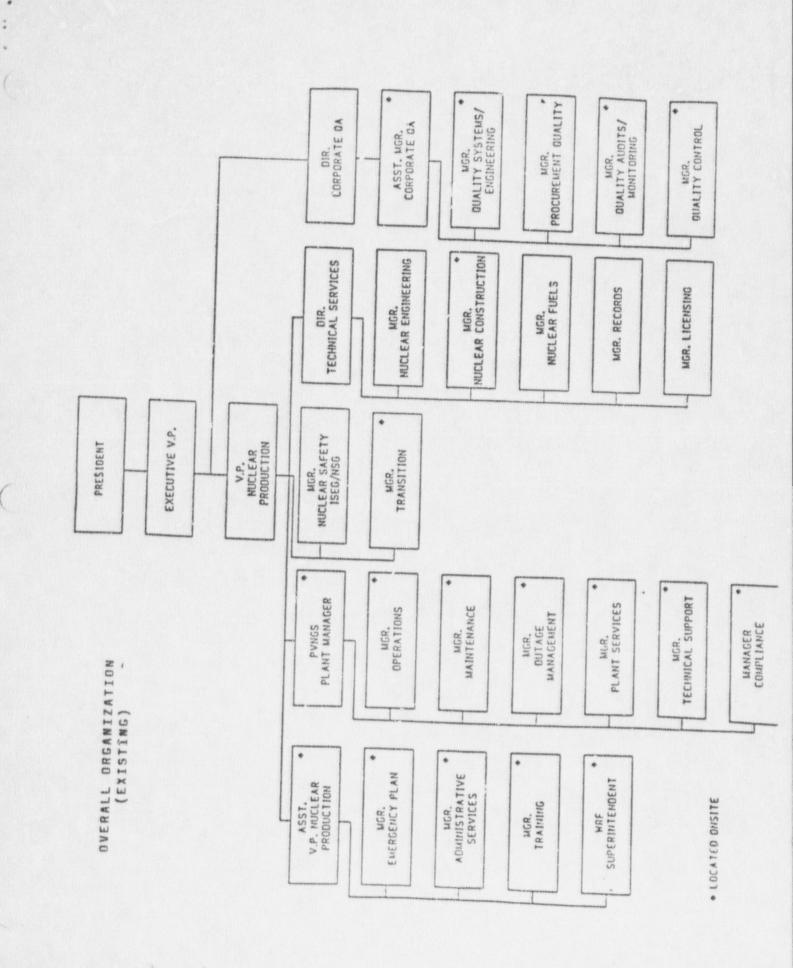
### ANNUNCIATOR STATUS

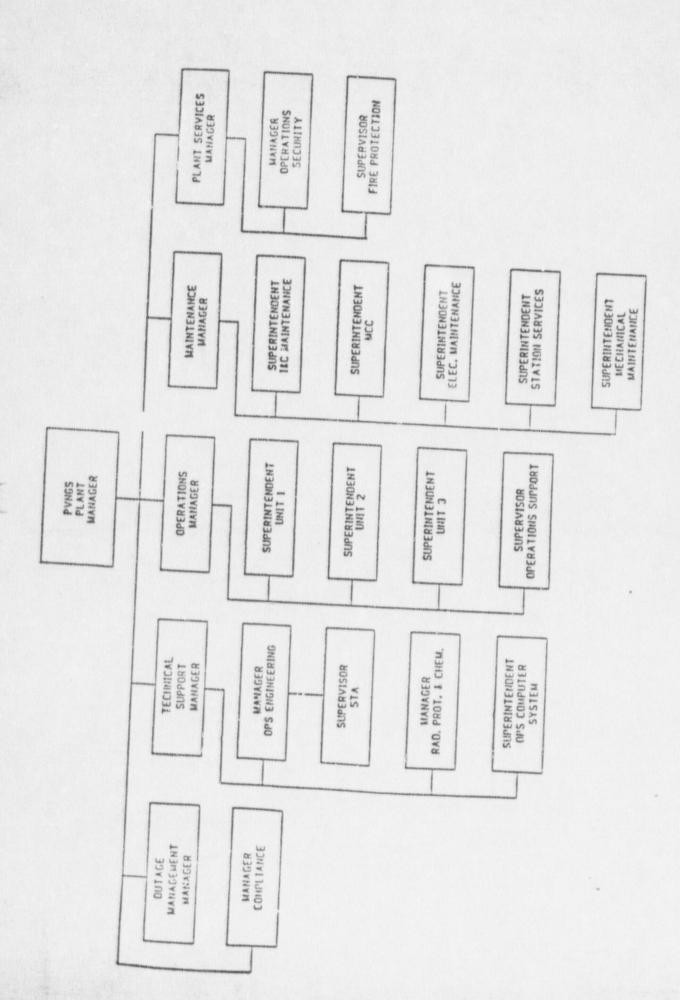
- DESCRIPTION OF PROBLEM:
- CONCERNS IDENTIFIED REGARDING NUMEROUS INVALID (BOGUS)
  CONTROL ROOM ANNUNCIATORS
- CORRECTIVE ACTIONS:
  - COMPUTERIZED LISTING TO TRACK ACTIONS TO RESOLVE
    ANNUNCIATOR PROBLEMS
  - DEDICATED OPERATIONS ENGINEER TO CENTRALIZE ENGINEERING
    RESOLUTIONS TO ANNUNCIATOR PROBLEMS
  - DEDICATED WORK CONTROL FOR EACH UN'T

## ANNUNCIATOR STATUS

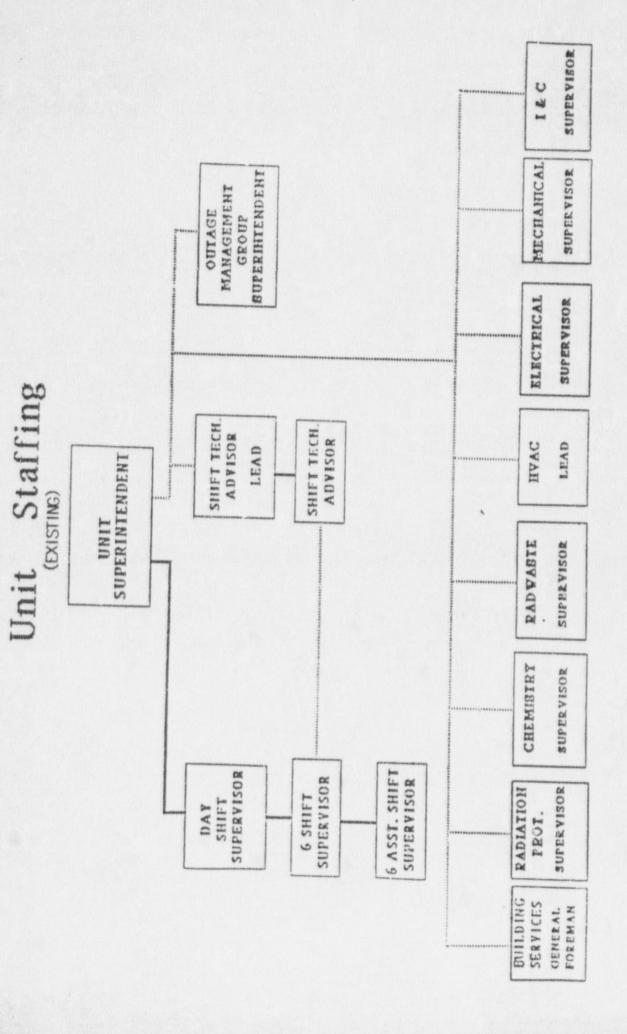
	<u>U-1</u>	<u>U-2</u>	<u>U-3</u>	TOTAL
MAY	59	41	44	144
JUNE	57	37	40	134
JULY*	59	31	35	125

\* AS OF JULY 29, 1987





ONSITE ORGANIZATION (EXISTING)



**OUALITY/AUDITS** ASSURANCE QUALITY SYS. ENGINEERING MONITORING DINECTOR POCURENENT MANAGER MANAGER OUALITY MANAGER OUALITY MANAGER QUALITY CONTROL NUCLEAR SAFETY & LICENSING DIFECTOR MANAGER NUCLEAR SUPERVISOR SAFETY MANAGER LICENSING COMPLIANCE MANAGER TECHNICAL MANAGER ISEG DATA CHIEF OPERATING VICE PRESIDENT PRESIDENT & EXECUTIVE OFFICER SITE SERVICES DIRECTOR EMERGENCY MANAGER MANAGER SERVICES SECURITY MANAGER TRAINING PLAN & MANAGER MATERIAL CONTROL MANAGER SERVICES ADMIN. ENGINEERING & CONSTRUCTION DIRECTOR ENGINEERING CONSTRUCTION MANAGER MANAGER SCHEDULE PART'SIPANT MANAGER MANAGER NUCLEAR SERVICES COST & MANAGER FUEL ORGANIZATION ANPP (NEW) VICE PRESIDENT MANAGER PLANT CIT The Item Brown PRODUCTION PLANT ASSISTANT STALIDANDS & UNIT 1 MANAGER MUCLEAR UNIT 2 SUPPORT UNIT 3 DIRECTOR TECHNICAL PLANT SUPPORT VICE PRESIDENT PRODUCTION NUCLEAR

ORGANIZATION

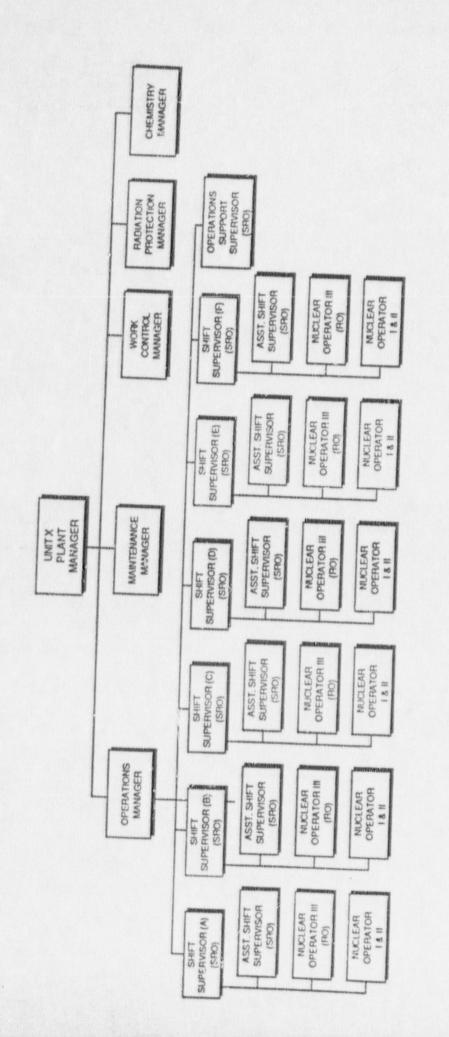
(NEM)

\*LOCATED OFFSITE

- AMIRIX

VICE PRESIDENT

NUCLEAR



(

(NEW)

UNIT SPECIFIC ORGANIZATION