

BRAIDWOOD NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-456

LICENSE NO. NPF-72

8708180119 870731
PDR ADDCK 05000456
R PDR

(4957z/90z)

I. Monthly Report for Braidwood Unit 1

A. Summary of Operating Experience

The unit entered the month of July in Mode 2. On July 1, the unit was shutdown due to rod control problems. Mode 2 was reentered on July 3. On July 5, a manual reactor trip was initiated after an inadvertent opening of the control rod drive motor generator breakers caused the control rods to de-energize and insert into the core. Mode 2 was re-entered on July 7. Initial synchronization to the grid took place on July 12 and turbine power was increased to approximately 10%. On July 15, a planned turbine trip was undertaken as part of testing. On July 19, a normal unit shutdown and cooldown was initiated to repair the 1B diesel generator and to perform other maintenance activities. Mode 2 was re-entered on July 30. The unit was again synchronized to the grid on July 31 and remained at 6% turbine power through the end of the month.

B. OPERATING DATA REPORT

DOCKET NO.: 50-456
 UNIT: Braidwood 1
 DATE: 08/10/87
 COMPILED BY: B. M. Peacock
 TELEPHONE: (815)458-2801
 ext. 2480

OPERATING STATUS

1. Reporting Period: July, 1987 Gross Hours: 480.0

2. Currently Authorized Power Level (MWt): 3411
 Design Electrical Rating (MWe-gross): 1175
 Design Electrical Rating (MWe-net): 1120
 Max Dependable Capacity (MWe-gross): 1175
 Max Dependable Capacity (MWe-net): 1120

3. Power level to which restricted (If Any): None

4. Reasons for restriction (If Any): None

	THIS MONTH	YR TO DATE	CUMULATIVE
5. Report period Hours:	480.0	480.0	480.0
6. Hours Reactor Critical:	225.4	225.4	225.4
7. RX Reserve Shutdown Hours:	0.0	0.0	0.0
8. Hours Generator on Line:	74.2	74.2	74.2
9. Unit Reserve Shutdown Hours:	0.0	0.0	0.0
10. Gross Thermal Energy (MWH):	64085	64085	64085
11. Gross Elec. Energy (MWH):	7185	7185	7185
12. Net Elec. Energy (MWH):	0.0	0.0	0.0
13. Reactor Service Factor:	N/A	N/A	N/A
14. Reactor Availability Factor:	N/A	N/A	N/A
15. Unit Service Factor:	N/A	N/A	N/A
16. Unit Availability Factor:	N/A	N/A	N/A
17. Unit Capacity Factor (MDC net):	N/A	N/A	N/A
18. Unit Capacity Factor (DER net):	N/A	N/A	N/A
19. Unit Forced Outage Rate:	N/A	N/A	N/A
20. Unit Forced Outage Hours:	405.8	405.8	405.8
21. Shutdowns Scheduled Over Next 6 Months:			

22. If Shutdown at End of Report Period,
Estimated Date of Startup: _____23. Units in Test Status (Prior to
Commercial Operation):

	FORECAST	ACHIEVED
Initial Criticality	05/23/87	05/29/87
Initial Electricity	07/11/87	07/12/87
Commercial Operation	09/26/87	_____

C. AVERAGE DAILY UNIT NET POWER LEVEL LOG

DOCKET NO.: 50-456
UNIT: Braidwood 1
DATE: 08/10/87
COMPILED BY: B. M. Peacock
TELEPHONE: (815)458-2801
ext. 2480

MONTH: July, 1987

1. _____	N/A	17. _____	0
2. _____	N/A	18. _____	0
3. _____	N/A	19. _____	0
4. _____	N/A	20. _____	0
5. _____	N/A	21. _____	0
6. _____	N/A	22. _____	0
7. _____	N/A	23. _____	0
8. _____	N/A	24. _____	0
9. _____	N/A	25. _____	0
10. _____	N/A	26. _____	0
11. _____	N/A	27. _____	0
12. _____	0	28. _____	0
13. _____	79	29. _____	0
14. _____	67	30. _____	0
15. _____	0	31. _____	21
16. _____	0		

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

D. UNIT SHUTDOWNS/REDUCTIONS

DOCKET NO.: 50-456
 UNIT: Braidwood 1
 DATE: 08/10/87
 COMPILED BY: B. M. Peacock
 TELEPHONE: (815)458-2801
 ext. 2480

REPORT PERIOD: July, 1987

<u>NO</u>	<u>DATE</u>	<u>TYPE</u>	<u>HOURS</u>	<u>REASON</u>	<u>METHOD</u>	<u>LER NUMBER</u>	<u>SYSTEM</u>	<u>COMPONENT</u>	<u>CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE</u>
N/A	870701	F	67.1	A	1	87-032	AA	FU	Rod control urgent alarm due to loose connections and blown fuses in the IAC movable gripper disconnect switch cabinet. Fuses were replaced and connections tightened. The remaining switch boxes were inspected and loose connections tightened.
N/A	870705	F	24.5	G	2	87-035	AA	MG	Inadvertent opening of the control rod drive motor generator breakers. Miscommunication between personnel. Personnel are being trained.
1	870715	F	405.8	B	1		TB		Planned turbine trip for testing. Repair of the 1B Diesel Generator and 1B MSIV.

 * SUMMARY *

<u>TYPE</u>	<u>REASON</u>	<u>METHOD</u>	<u>SYSTEM & COMPONENT</u>
F-Forced	A-Equipment Failure	1 - Manual	Exhibit F & H
S-Scheduled	B-Maint or Test	2 - Manual Scram	Instructions for Preparation of
	C-Refueling	3 - Auto Scram	Data Entry Sheet
	D-Regulatory Restriction	4 - Continued	Licensee Event Report
	E-Operator Training & License Examination	5 - Reduced Load	(LER) File (NUREG-0161)
	F-Administration	9 - Other	
	G-Oper Error		
	H-Other		

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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None

2. Licensee generated changes to ODCM.

None

F. LICENSEE EVENT REPORTS

The following is a tabular summary of all Licensee Event Reports submitted during the reporting period, July 1 through July 31, 1987. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10 CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Report Date</u>	<u>Title of Occurrence</u>
87-025-00	07/01/87	Inadvertent opening of main steam isolation valve during maintenance activities due to procedural error.
87-025-01	07/29/87	Inadvertent opening of main steam isolation valve during maintenance activities due to procedural error.
87-027-00	07/01/87	Reactor trip signal generated during solid state protection system logic testing due to a procedural deficiency.
87-029-00	07/14/87	Missed technical specification surveillance due to procedural deficiency.
87-031-00	07/02/87	Control room ventilation shift to the emergency makeup mode as a result of spurious actuation of a radiation monitor due to inadvertent radio operation.
87-032-00	07/22/87	Manual reactor trip due to loose connections and bad fuses in a rod control disconnect switch cabinet.
87-033-00	07/29/87	Control room ventilation system inoperable due to inadvertent de-energized damper.
87-034-00	07/29/87	Analysis frequency exceeded due to data storage limitations in the counting room computer.
87-036-00	07/29/87	Missed circulation water blowdown composite sample due to inadequate procedure.



Commonwealth Edison
Braidwood Nuclear Power Station
Route #1, Box 84
Braceville, Illinois 60407
Telephone 815/458-2801

EEF/87-1151

August 10, 1987

Director, Office of Resource Management
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the Monthly Performance Report covering Braidwood Nuclear Power Station for the period July 1 through July 31, 1987.

Very truly yours,

E. E. Fitzpatrick
Station Manager
Braidwood Nuclear Power Station

EEF/RCB/mjv
(4957z)

Attachments

cc: A. B. Davis, NRC, Region III
NRC Resident Inspector Braidwood
Gary Wright, Ill. Dept. of Nuclear Safety
T. J. Maiman
K. L. Graesser
L. D. Butterfield
Nuclear Fuel Services, PWR Plant Support
L. Anastasia, Station Nuclear Engineering
INPO Records Center
Performance Monitoring Group, Tech Staff Braidwood Station
Nuclear Group, Tech Staff Braidwood Station
J. Stevens - USNRC
P. L. Barnes

(4957z/90z)

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