

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Perry Nuclear Power Plant, Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 4 4 0				PAGE (3) 1 OF 2		
TITLE (4) Control Room Rounds Log Deficiency Results in Technical Specification Violation Due to Surveillance Requirements Not Being Performed																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 7	1 3	8 7	7 8	0 5	3 0	0 0	8 1	1 8					0 5 0 0 0			
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8. (Check one or more of the following) (11)																
OPERATING MODE (9) 4			20.402(b)				20.405(e)				50.73(a)(2)(iv)				73.71(e)	
POWER LEVEL (10) 0 0 0			20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(e)	
			20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
			20.405(a)(1)(iii)				X 50.73(a)(2)(i)				50.73(a)(2)(viii)(A)					
			20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)					
			20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)					
LICENSEE CONTACT FOR THIS LER (12)																
NAME Gregory A. Dunn, Compliance Engineer, Extension 6484										TELEPHONE NUMBER 2 1 1 6 2 5 1 9 - 1 3 1 7 3 1 7						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs					CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs			
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 14 it was discovered that readings for the Control Room Area Radiation Monitor and the Source Range Neutron Monitors (SRM) were not recorded on the previous four shifts and appropriate actions as required by Technical Specifications 3.8.7.1 and 3.3.7.6 were not taken. The readings were taken immediately upon discovery of the overdue surveillances. The instruments were capable of performing their intended function and were found to be operating satisfactorily upon completion of the required Channel Checks.

The event was caused by the use of a new Plant Rounds Instruction (PRI). The wording and location of notes in the log caused confusion. The operators misinterpreted the notes to mean that the readings were not required.

The notes in the PRI have been changed. Other logs have been reviewed for similar discrepancies and changes incorporated. The operators involved with the event have been counseled on the need for greater attention to detail and to ensure a full understanding when performing actions of an instruction.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Perry Nuclear Power Plant, Unit 1	0 5 0 0 0 4 4 0	8 7	— 0 5 3	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On July 13, 1987 reading for the Control Room Radiation Monitor [IL] and the Source Range Neutron Monitors (SRM)[IG] were not recorded and appropriate actions as required by Technical Specification 3.3.7.1 and 3.3.7.6 were not taken. At the time of the event the plant was in Operational Condition 4 (Cold Shutdown) with reactor power at zero percent of rated. Reactor coolant temperature was approximately 130 degrees and reactor vessel [RPV] pressure was atmospheric.

At 0815 on July 14 it was discovered that the Channel Check for the Control Room Area Radiation Monitor had not been performed on the previous four shifts. Technical Specification 4.3.7.1 which requires the Control Room Area Radiation Monitor Channel Check to be performed at 12 hour intervals. At 1330 it was also discovered that SRM Channel Checks had not been performed during the same period of time. Technical Specification 4.3.7.6.1 requires SRM Channel Checks to be performed at 24 hour intervals. At the time of the event the operators were unaware that surveillance requirements had been missed, therefore, the Technical Specification required actions were not performed. The readings were taken immediately upon discovery of the overdue surveillances.

The cause for missing the readings was a Plant Rounds Instruction - Technical Specification Rounds (PRI-TSR) for Operational Conditions 4 and 5, which was being used for the first time. The format was changed from one day of readings per log to one week of daily readings per log. Notes are used to clarify when readings are required, however, the wording and location of the notes in the revised format caused confusion. The operators misinterpreted the notes to mean that the readings were not required.

The Control Room Area Radiation Monitor provides a means for monitoring gamma radiation levels in the control room and alert the operators in case of inadvertent high radiation. The SRMs provide reactor core neutron flux indication during low flux operations and detect conditions that could lead to either fuel damage or personnel radiation exposure and provide signals that can mitigate the consequences of such conditions. The instruments were capable of performing their intended function and were found to be operating satisfactorily upon completion of the required Channel Checks on July 14. Therefore, these events are considered to have no safety significance. No previous similar events have been identified.

The notes in PRI-TSR for Operational Conditions 4 and 5 have been changed to ensure no further confusion will occur. PRI-TSR for Operational Conditions 1, 2 and 3 have also been reviewed and changes incorporated to notes which may have caused confusion. The operators involved with the event have been counseled on the need for greater attention to detail and to ensure full understanding when performing actions of an instruction.

Energy Industry Identification System Codes are identified in the text as [XX].



THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

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MURRAY R. EDELMAN
SR. VICE PRESIDENT
NUCLEAR

August 11, 1987
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Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Perry Nuclear Power Plant
Docket No. 50-440
LER 87-053-00

Dear Sir:

Enclosed is Licensee Event Report 87-053-00 for the Perry Nuclear Power Plant.

Very truly yours,

Murray R. Edelman
Senior Vice President
Nuclear Group

MRE:njc

Enclosure: LER 87-053-00

cc: T. Colburn
K. Connaughton

U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

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11