



George S. Thomas
Vice President-Nuclear Production

August 10, 1987

Public Service of New Hampshire

NYN-87092

New Hampshire Yankee Division

United States Nuclear Regulatory Commission
Washington, DC 20555

Attention: Document Control Desk

References: (a) Facility Operating License NPF-56, Construction Permit
CPPR-136, Docket Nos. 50-443 and 50-444
(b) PSNH Letter (NYN-87056), dated April 22, 1987, "Comments
on Final Brookhaven National Laboratory Technical Eval-
uation", George S. Thomas to the USNRC
(c) PSNH Letter (NYN-87077), dated June 10, 1987, "Radio-
logical Emergency Response Plan for Commonwealth of
Massachusetts, Request for Meeting", W. B. Derrickson
to Dr. T. E. Murley

Subject: Assessment of Induced Steam Generator Tube Rupture (ISGTR)
and Direct Containment Heating (PLG-0550)

Gentlemen:

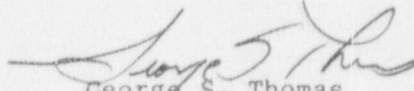
In Reference (b) and during a meeting with the NRC Staff and Brook-
haven National Laboratory on March 25, 1987, New Hampshire Yankee (NH)
committed to perform supplemental reviews of the Seabrook Station Risk
Management and Emergency Planning Study to resolve technical areas iden-
tified by the NRC Staff and the Brookhaven National Laboratory during
their assessment of the results and conclusions. In Reference (c), NH
indicated that such prior commitments to provide additional information
to the NRC would be kept, notwithstanding NH's present intention not to
resubmit a EPZ reduction waiver request. NH believes that this addi-
tional information will be of value to the NRC in updating its under-
standing of Seabrook Station's operation and safety. Accordingly, en-
closed for your review please find a detailed assessment (PLG-0550) of
the Induced Steam Generator Tube Rupture (ISGTR) and Direct Containment
Heating (DCH) phenomenological issues regarding the probability of early
Containment bypass and failure of the Seabrook Station Containment. Based
on this study, NH is convinced that neither issue poses a significant
threat to containment integrity.

The enclosed report provides a significant level of detailed infor-
mation regarding these phenomenological issues. The probabilistic anal-
ysis of Reactor Coolant System failure modes and Containment failure,
including uncertainties, is included in the report. The ISGTR and DCH
issues are re-examined in light of the results of this probabilistic
analysis. These analyses include consideration of relevant results of
NUREG-1150 and NUREG/CR-4896.

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NHY is prepared to discuss this work with the NRC Staff and respond to questions as appropriate. NHY requests that the Staff and its consultants review the study and provide us with any comments regarding the report's conclusions. Should you have any questions, please contact Mr. W. J. Daley, Jr., at (603) 474-9521, extension 2057.

Very truly yours,


George S. Thomas

Enclosure

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