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December 17, 1987

Mr. Thomas E. Murley, Director Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, DC 20555

Attn: Document Control Desk

- Subject: Braidwood Station Unit 2 Reactor Containment Building Integrated Leakage Rate Test Supplemental Report NRC Docket No. 50-457
- Reference (a): December 7, 1987 S.C. Hunsader letter to T.E. Murley

Reference (a) provided the NRC with the Braidwood Unit 2 Report, "Reactor Containment Building Integrated Leakage Rate Test", pursuant to 10 CFR 50, Appendix J, Section V. Attached to this letter is a supplemental technical report which is also being submitted pursuant to the requirements of 10 CFR 50, Appendix J, Section V.

This report describes the initial type C containment leakage rate test conducted between October 26 and December 8, 1987. This test was conducted in accordance with the requirements of 10 CFR 50, Appendix J.

This report is being submitted for NRC review. Please address any guestions regarding this matter to this office.

Very truly yours,

S. C. Humode

S. C. Hunsader Nuclear Licensing Administrator

cc: S. Sands NRC RIII Braidwood Resident Inspector



CONTAINMENT MODIFICATION SUPPLEMENT REPORT

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SUPPLEMENT REPORT

The Hydrogen Monitoring System for Penetration 36 was not tested during the Type A test conducted September 6 through the 11, 1987 due to the long lead time required for parts from the manufacturer (Valcor) to repair valve (2PS230B). After the successful completion of the Type A test, numerous attempts were made to acquire a similar valve from various utilities around the country. Attempts were also made to expedite the delivery date of a replacement valve from the manufacturer. Due to the unusual circumstance of the long delivery time for 2PS230B valve and the economic justification of extending the N-Stamp in order to install this valve, the decision was made to remove 2PS230A from Penetration 45 and install the valve at Penetration 36.

A new valve has been installed in penetration 45 (2PS230A) and valve 2PS230B was repaired for penetration 36. The local leak rate has been performed on penetration 45 and reperformed on penetration 36 per Appendix J of 10CFR50 and ANSI N45-4-1972. The test volumes were pressurized and regulated with air to nominally 46 psig. The test volume leak rate was determined by measuring the flowrate necessary to maintain the test volume at 46 psig after a suitable stabilization period.

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Leakage Rate Summary:

1.1.1.1

Type	Penetration (P)	Leakage Rate (sccm)
С	P-36 Hydrogen Monitoring (SUC)	249 ± 20
С	P-36 Hydrogen Monitoring (RET)	305 <u>+</u> 20
С	P-36 Hydrogen Monitoring (SUC)	11.4 <u>+</u> 2
C	P-36 Hydrogen Monitoring (RET)	337 ± 20

(FINAL)