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LICENSEE EVENT REPORT (LER)

U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3180-0104 EXPIRES 8/31/88

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On September 8, 1987 at 1330, it was determined by the Maintenance section that the results of a Local Leak Rate Test (LLRT) of check valve 1T48*01V-0016A (Post Accident Sampling Containment Atmosphere Sample Return), showed leakage that, when combined with all "B" and "C" penetration leakages exceeded the Tech. Spec. limit of 0.6 La. During the LLRT program, the plant was in Operational Condition 4 (Cold Shutdown) with the mode switch in Shutdown and all rods inserted in the core. Subsequent testing identified six additional valves which had excessive leakage. Maintenance Work Requests (MWRs) were generated to investigate the leakage and to repair the valves as needed. The 1T48 valve was repaired and retested with acceptable results. The remaining valves are scheduled to be repaired. A supplemental report will be generated upon completion of the LLRT and will identify all valves which significantly contributed to the combined leakage along with any corrective action taken.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor

Energy Industry Identification System (EIIS) codes are identified in the text as [xx].

IDENTIFICATION OF THE EVENT

Results of LLRT for check valve showed leakage that, when combined with all Type B and C penetration leakage, exceeded the Tech. Spec. limit of 0.6 La.

Event Date: 9/8/87

Report Date: 12/4/87

CONDITIONS PRIOR TO THE EVENT

Operational Condition 4 (Cold Shutdown)

Mode Switch - Shutdown

RPV Pressure = 0 psig RPV Temperature = 101 Degrees F

POWER LEVEL - 0

All rods inserted in the core

DESCRIPTION OF THE EVENT

On September 8, 1987 at 1330, it was determined by the Maintenance section that the results of a Local Leak Rate Test (LLRT) of check valve 1T48*O1V-0016A [BD] (Post Accident Sampling Containment Atmosphere Sample Return), showed leakage that, when combined with all "B" and "C" valve leakages exceeded the Tech. Spec. limit of C.6 La. The penetration was tested on September 3, 1987 to satisfy Tech. Spec. requirement 3.6.1.2.b. Upon identification of the

TLT IF mean species a resource: was enterespress AcR. Agent Missel a 117.

excessive leakage, a Maintenance Work Request was generated to investigate the cause and subsequent repair of the valve. Subsequent testing identified six additional valves which had excessive leakage. They are;

1T46*AOV-039D [BD] (Purge Air from Suppression Pool Isolation Valve)

1B21*18V-1103A [SJ] ("A" Inboard Feed Water Check Valve)

1E11*01V-047 [BO] (PASS Sample Return Check Valve)

1E11*MOV-031A [BO] (RHR "A" Pump Suppression Pool Suction Valve)

1E11*AOV-081A [BO] (RHR LPCI "A" to Recirc. Syst. Valve)

1E11*MOV-042A [BO] (RHR Containment Spray Valve)

A supplemental report will be issued upon completion of the LLRT to identify all valves which significantly contributed to the combined leakage.

CAUSE OF THE EVENT

The cause of the leakage for the 1T48 valve was due to normal valve degradation (excessive valve disc to seat leakage). The 1T46 valve had oxidation products lodged in the seat of the valve, preventing a tight seal. The cause for the leakage for the remaining five valves has not yet been determined. This will be discussed upon completion of the LLRT in the supplemental report.

ANALYSIS OF THE EVENT

This event is reportable per 10CFR50,73(a)(2)(ii) as identified in NUREG 1022, page 12. There is minimal safety significance to this event. All seven valves are set up in a double valve configuration and their adjacent valves leak tested satisfactorily, and would have provided a means for containment isolation upon an accident signal. Had a LOCA occurred during 5% power operation with the leakage rate above the allowable rate, the radiation doses would have been well below the limits as required by 10CFR100 and the General Design Criterion 19 of Appendix A of 10CFR50.

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			9048 8167141 MI. 8.C.	
Shoreham Nuclear	Power Station Unit #1	0 6 0 0 0 3 2 2	8 7 -0 2 9 - 0 1	0 4 0 0 0 5

CORRECTIVE ACTIONS

MWRs were generated for all seven valves. The T48 valve was repaired and retested with acceptable results. The remaining valves are scheduled to be repaired and will be retested upon completion of the repairs. The supplemental report will identify those actions taken to repair the valves along with corrective actions taken for any additional valves which significantly contributed to the combined leakage.

ADDITIONAL INFORMATION

a. Manufacturer and model number of failed component (s)

Manufacturer	Model No.
Velan Valve Co. Fisher Controls Co. Anchor-Darling Valve Co. Velan Valve Co. Anchor-Darling Valve Co. Velan Valve Co.	299-1 9220 E5836-14-2 3026X E5836-1-1 B24-7114-255P E5836-4-1
	Velan Valve Co. Fisher Controls Co. Anchor-Darling Valve Co. Velan Valve Co. Anchor-Darling Valve Co.

b. LER numbers of previous similar events 85-049

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LONG ISLAND LIGHTING COMPANY

SHOREHAM NUCLEAR POWER STATION . P.O. BOX 628 - WADING RIVER, NEW YORK 11792

TEL. (516) 929-8300

December 4. 1987

PM-87-293

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Dear Sir:

In accordance with 10CTR50.73, enclosed is a revision to Shoreham Nuclear Power Station's Licensee Event Report LER 87-029 Rev. 1. This revision is being submitted to identify additional containment isolation valves which significantly contributed to the combined leakage during their respective LLRTs

Sincerely yours.

William E Steiger, Jr.

Plant Manager

WES/pz

Enclosure

cc: William T. Russell, Regional Administrator

Frank Crescenzo, Resident Inspector

Institute of Nuclear Power Operations, Records Center

American Nuclear Insurers

SR. A21.0200

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