

# Maine Yankee

RELIABLE ELECTRICITY FOR MAINE SINCE 1972

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December 7, 1987  
MN-87-134

GDW-87-275

Director of Nuclear Reactor Regulation  
United States Nuclear Regulatory Commission  
Washington, D. C. 20555

Attention: Acting Project Director I-3  
Division of Reactor Projects I/II

References: (a) License No. DPR-36 (Docket No. 50-309.).  
(b) Letter MYAPCo to USNRC dated September 22, 1977 (WMY-77-87).  
(c) MYAPCo Final Safety Analysis Report, Section 14.14.3.  
(d) CEN-114-P, Amendment 1-P, "Review of Small Break Transients in Combustion Engineering Nuclear Steam Supply Systems", August 3, 1979.  
(e) Letter MYAPCo to USNRC dated August 31, 1979 (WMY-79-86).  
(f) CEN-203-P, Revision 1-NP-A (Supplements 1-NP-A and 2-NP-A), and Revision 1-P (Supplements 3-A and 4-A), "Response to NRC Action Plan Item II.K.3.30: Justification of Small Break LOCA Methods", 3/82-11/86.  
(g) NRC Safety Evaluation Report, "TMI Action Item II.K.3.30 for Combustion Engineering Plants", May 23, 1985, and NRC Safety Evaluation Supplement, "TMI Action Item II.K.3.30 for Combustion Engineering Plants", transmitted via Letter from D. M. Crutchfield (NRC) to J. K. Gasper (CEOG), February 11, 1987.

Subject: TMI Action Items II.K.3.30 and II.K.3.31

Gentlemen:

The small break loss of coolant accident (SBLOCA) analysis of record for the Maine Yankee plant was provided to the NRC as Reference (b). The CE methodology and modelling logic used in generating that analysis is detailed in both that submittal and the Maine Yankee FSAR, Reference (c). This same methodology has been utilized by the CEOG to provide a generic analysis, CEN-114-P, Reference (d), to the NRC as the basis for resolution of TMI Action Item II.K.3.30. Reference (e), places this report on the Maine Yankee docket. Subsequent CEOG analysis, CEN-203-P, Reference (f), has demonstrated the appropriateness of this methodology to conservatively predict SBLOCA response.

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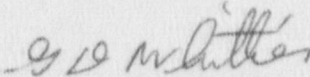
In Reference (g), the NRC has accepted the CEOG methodology as appropriate to resolve Action Item II.K.3.30. Since the Maine Yankee analysis of record was performed with the same methodology, Maine Yankee now requests the closeout of Action Item II.K.3.30.

The submittal of plant specific analysis, per Action Item II.K.3.31, is no longer required for those plants utilizing the CE methodology as approved through the resolution of Action Item II.K.3.30. Therefore, in accordance with the directions given in Reference (g), Maine Yankee additionally requests the closeout of Action Item II.K.3.31.

We trust you will find this course of action acceptable. Please feel free to contact me if you wish to discuss this matter further.

Very truly yours,

MAINE YANKEE



G. D. Whittier, Manager  
Nuclear Engineering and Licensing

GDW/bjp

cc: Mr. William T. Russell  
Mr. Pat Sears  
Mr. Cornelius F. Hoiden