LICENSEE EVENT REPORT (LER)

Facility Name (1)	Docket Number (2)	Page (3)
LaSalle County Station Unit 2	0 5 0 0 0 3 7 4	1 of 0 6

Title (4) Failure of Type C Leak Rate Test

Event Date (5)			LER Number (6)			Report Date (7)			Other Facilities Involved (8)	
Month	Day	Year	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)
										0 5 0 0 0
0 2	2 6	8 5	8 5	0 1 1	0 1	1 1	0 4	8 7		0 5 0 0 0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)								
1	___ 20.402(b)	___ 20.405(c)	___ 50.73(a)(2)(iv)	___ 73.71(b)
	___ 20.405(a)(1)(i)	___ 50.36(c)(1)	___ 50.73(a)(2)(v)	___ 73.71(c)
POWER LEVEL (10) 0 9 9	___ 20.405(a)(1)(ii)	___ 50.36(c)(2)	___ 50.73(a)(2)(vii)	___ Other (Specify in Abstract below and in Text)
	___ 20.405(a)(1)(iii)	___ 50.73(a)(2)(i)	___ 50.73(a)(2)(viii)(A)	
	___ 20.405(a)(1)(iv)	X 50.73(a)(2)(ii)	___ 50.73(a)(2)(viii)(B)	
	___ 20.405(a)(1)(v)	___ 50.73(a)(2)(iii)	___ 50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

Name	TELEPHONE NUMBER
	AREA CODE
Paul S. Watford, Technical Staff Engineer, extension 323	8 1 5 3 5 7 - 6 7 6 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS
X	W K	I S V	A 3 9 1	Y	X	S B	I S V	A 3 9 1	Y
X	S B	L 0 V	A 3 9 1	Y	X	S B	I S V	A 3 9 1	Y

SUPPLEMENTAL REPORT EXPECTED (14)

		Expected Submission Date (15)	Month	Day	Year
___ Yes (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e, approximately fifteen single-space typewritten lines) (16)

On February 26, 1985, at 1155 hours with the unit at 99% power, the 2FC086 and 2FC115 Reactor Well Drain Valves failed their Local Leak Rate Test (exceeded 0.6 La leakage rate for worst valve in line). The Actions of Technical Specification 3.6.3 were taken as appropriate.

Initial inspection showed that foreign matter on seat and seat irregularities caused the valves to fail the Local Leak Rate Test. The valves were disassembled and the seat surfaces lapped. A table is included to summarize all other Local Leak Rate Test failures during the Unit 2 outage.

LICENSEE EVENT REPORT (LER) FAILURE CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6) Year	Sequential Number	Revision Number	Page (3)
LaSalle County Station Unit 2	0 5 0 0 0 3 7 4	8 5	- 0 1 1	- 0 1	0 2 OF 0 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	S B	I S V	A 3 9 1	Y					
X	S B	I S V	A 3 9 1	Y					
X	W K	I S V	M 0 4 0	Y					
X	W K	I S V	M 0 4 0	Y					
X	W K	I S V	M 0 4 0	Y					
X	C E	I S V	A 3 9 1	Y					
X	N H	C P L G		N					
X	K M	I S V	A 3 9 1	Y					
X	K M	I S V	F 1 3 0	Y					
X	B B	I S V	A 3 9 1	Y					
X	B N	I S V	A 3 9 1	Y					
X	B N	I S V	R 3 4 0	Y					
X	L E	I S V	W 2 5 5	Y					
X	B N	I S V	R 3 4 0	Y					

TEXT

PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor

Energy Industry Identification System (EIIS) codes are identified in the text as [XX].

A. CONDITION PRIOR TO EVENT

Unit(s): 2 Event Date: 2/26/85 Event Time: 1155 Hours

Reactor Mode(s): 1 Mode(s) Name: Run Power Level(s): 99%

B. DESCRIPTION OF EVENT

On February 26, 1985, at 1155 hours with the Unit 2 reactor at 99% power, the 2FC086 and 2FC115 Reactor Well Drain Valves (DA) failed their Local Leak Rate Test (exceeded 0.6 La leakage rate for worst valve in line). The valves were declared inoperable and isolated per Technical Specification 3/4.6.3. These valves were the first ones tested that failed during the scheduled Unit 2 outage. During the Unit 2 scheduled outage which commenced on February 27, 1985, additional Type B and C test failures occurred which exceeded the 0.6 La limit (Technical Specification 3.6.2.1 and 10CFR50 Appendix J.) Attachment A contains the summary of valves that failed.

C. APPARENT CAUSE OF EVENT

Attachment A contains the summary of containment isolation valves with excessive leakage and indicates the cause of failure for each LLRT failure encountered during the Unit 2 outage.

D. SAFETY ANALYSIS OF EVENT

Using single valve failure criterion (maximum pathway leakage), the allowable primary containment leakage of 0.6 La as limited by 10CFR50 Appendix J and Technical Specifications was exceeded. Single valve failure criterion is used for tracking and reporting purposes. The consequences of this occurrence is that it was necessary to repair a number of containment isolation valves to bring the combined measured leak rate below the Technical Specification limit prior to resuming power operation. Exceeding this limit however, did not pose any significant risks or hazards to the public because the total leakage determined by Type B and C Tests does not represent the probable leakage from the containment under accident conditions. The actual leakage consequences for a single valve failure is maintained at a low limit due to the proper operation of the second isolation valve. All valves that failed in this report had a second isolation valve or system that would have prevented excessive leakage out of the primary containment in the case of an accident.

TEXT

E. CORRECTIVE ACTIONS

Attachment A lists this summary of corrective action undertaken to reduce the leakage rate of valves which failed their Local Leak Rate Test.

F. PREVIOUS EVENTS

LER Number	Title
LER 373/83-146/03X-1	Inboard Feedwater Check Valve Fail Leak Rate Test
LER 373/84-012-00	Feedwater Check Valve Failed Local Leak Rate Test
LER 373/84-064-01	Feedwater Valves Fail Local Leak Rate Test

G. COMPONENT FAILURE DATA

Manufacturer	Nomenclature	Model Number	MFG Part Number
Anchor/Darling Valve Co.	10" Valve	93-14413	
Anchor/Darling Valve Co.	24" Valve	3600-3	
Anchor/Darling Valve Co.	24" Valve	94-13789	
Magnetrol Intl. Inc.	2" Valve	70-29-1	
Anchor/Darling Valve Co.	6" Valve	94-13753	
Anchor/Darling Valve Co.	10" Valve	93-13769	
Anchor/Darling Valve Co.	6" Valve	93-14409	
Anchor/Darling Valve Co.	3" Valve	94-13751	
Anchor/Darling Valve Co.	8" Valve	93-14412	
WKM Div/ACF Ind. Inc.	1.5" Valve	70-29-1	
Anchor/Darling Valve Co.	10" Valve	94-13755	

TEXT

VALVE/PENETRATION	AS FOUND	AS LEFT	CAUSE	CORRECTIVE ACTIONS
2FC115, Reactor Well Drain Upstream Isolation Valve	19.8	1.2	Foreign matter on seat & seat irregularities.	Disassembled valve & lapped seating surface.
2FC086, Reactor Well Drain Downstream Isolation Valve	235.7	1.2	Foreign matter on seat & seat irregularities.	Disassembled valve & lapped seating surface.
2B21-F032A, Outboard Feedwater Check Valve	85.9	51.7	Actuator cylinder piston seals blown. Could not hold disc closed for test.	Rebuilt actuator cylinders.
2B21-F010A, Inboard Feedwater Check Valve	2100	43.4	Soft seat material degraded possibly due to flow erosion and/or temperature effects	Soft seat material removed from disc. Metal seat was lapped.
2B21-F032B, Outboard Feedwater Check Valve	Not Measurable	14.7	Actuator cylinder piston seals blown. Could not hold disc closed for test.	Rebuilt actuator cylinders.
2B21-F010B, Inboard Feedwater Check Valve	Not Measurable	3.1	Soft seat material degraded possibly due to flow erosion and/or temperature effects.	Soft seat material removed from disc. Metal seat was lapped.
2RF012, Floor Drain Sump Inboard Isolation Valve	Not Measurable	.5	Foreign matter on valve seat and actuator spring inadequately set.	Disassemble valve, lapped seat, and replaced actuator spring.
2RE025, Equipment Drain Sump Cooling Inboard Isolation Valve	19.0	.05	Actuator spring inadequately set.	Advised actuator spring and set to maximum compression setting.
2RE024, Equipment Drain Sump Pump Suction Outboard Isolation Valve	Not Measurable	.5	Actuator spring inadequately set.	Readjusted actuator spring to maximum compression setting.
2G33-F001 Reactor Water Cleanup Suction Inboard Isolation	19.4	6.3	Slight irregularities noted on valve seat.	Disassembled valve and lapped seat.
E-12, Electrical Penetration	62.5	.96	Loose union fitting.	Tightened fitting.
2E51-F068, RCIC Turbine Exhaust Isolation Valve	49.9	2.9	Limit switch contacts opening too soon & torque switch contacts dirty.	Reset limits and cleaned torque switch.

TEXT

VALVE/PENETRATION	AS FOUND	AS LEFT	CAUSE	CORRECTIVE ACTION
2HG006A, Hydrogen Recombiner Outlet Valve	108.7	.3	Torque switch contacts dirty, not allowing full closure.	Torque switch contacts cleaned.
2B21-F019, Main Steam Line Drain Outboard Isolation	2.6	.6	Stem leakage through packing.	Tightened packing.
2VP053A, Drywell Cooler Coils Outboard Isolation Valve	152.2	3.0	Torque switch setting set low.	Adjusted torque switch setting.
2IN075, Drywell Pneumatic Dryer Purge Valves	10.8	3.2	Slight irregularities noted on valve seat.	Disassembled valve and lapped seat.
2E51-F063, 76, 64, 08, 91 LTS-100-24	252.6	1.2	Foreign matter on seat surface.	Pressurized volume and cycled valves to blow matter off seats.

Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

ember 4, 1987

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Licensee Event Report #85-011-01, Docket #050-374 is being submitted to your office to supercede previously submitted Licensee Event Report 85-011-00 to update report with additional information.

for R. D. Bishop
G. J. Diederich
Station Manager
LaSalle County Station

GJD/PSW/kg

Enclosure

xc: Nuclear Licensing Administrator
NRC Resident Inspector
NRC Region III Administrator
INPO - Records Center

IE22
1/1