

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

Trojan Nuclear Plant

DOCKET NUMBER (2)

0 5 0 0 0 3 4 4 1 OF 0 2

PAGE (3)

TITLE (4)

Valve Packing Leakage Exceeded FSAR Assumed Leakage

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)						
0	5	0	9	8	7	8	7	0	1	8	0	5	0	0	0		
									NA		0	5	0	0	0		
											0	5	0	0	0		
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)														
6			20.402(b)			20.406(e)			80.73(e)(2)(iv)			73.71(b)					
POWER LEVEL (10)			20.406(a)(1)(i)			80.36(e)(1)			80.73(e)(2)(v)			73.71(e)					
0 1 0 1 0			20.406(a)(1)(ii)			80.36(e)(2)			80.73(e)(2)(vi)			OTHER (Specify in Abstract below and in Text, NRC Form 306A)					
			20.406(a)(1)(iii)			80.73(e)(2)(i)			80.73(e)(2)(vii)(A)								
			20.406(a)(1)(iv)			X 80.73(e)(2)(ii)			80.73(e)(2)(vii)(B)								
			20.406(a)(1)(v)			80.73(a)(2)(iii)			80.73(e)(2)(x)								

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Bill Kershul, Plant Review Board Engineer	5 0 3 5 5 6 - 3 7 1 3

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
X	B	E	V	A	3	9	1	Yes	
X	B	P	V	C	3	6	5	Yes	

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	X				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During local leak rate testing (LLRT), on May 9, 1987, the Containment Spray and Residual Heat Removal recirculation suction valves outside containment (MO2052B and MO-8811B) exhibited packing leaks. The leakage exceeded the 1580 cubic centimeters per hour assumed in the Final Safety Analysis Report for post-accident recirculation leakage.

The cause of the valve packing leaks was attributed to normal packing degradation.

The valve packings were tightened and tested satisfactorily. The valve packing program for valves in the recirculation flow path will be reviewed for changes necessary to prevent recurrence.

This event had no effect on public health and safety. The LLRT of this penetration was performed with air at 60 psig. Actual leakage in the event of operation of the recirculation sump flow path would have been significantly less because: (1) the leakage would have been liquid instead of gas, and (2) recirculation does not commence until post-accident Containment pressures have decreased significantly below 60 psig.

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S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Trojan Nuclear Plant	0 5 0 0 0 3 4 4	8 7	— 0 1 8	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Description of Event

On May 9, 1987, the plant was in Mode 6 (refueling) with Reactor Coolant System temperature at 81°F. Local Leak Rate Testing (LLRT) of Penetration P-50 (Containment Sump Recirculation) was in progress. At about 1030 hr, packing leaks were observed from valves MO-2052B (Containment spray recirculation suction-outside containment) and MO-8811B (Residual Heat Removal recirculation suction-outside containment). The total leakage from both valves was approximately 150 cubic centimeters (cc)/minute (9000 cc/hour). Final Safety Analysis Report (FSAR) Section 15.6.5.6 specifies an assumed Containment Spray System leakage of 192 cc/hr and a total Emergency Core Cooling System leakage of 1580 cc/hr. The actual leakage exceeded these FSAR values.

An immediate notification of this event was made in accordance with 10 CFR 50.72(b)(1)(ii) on May 9, 1987. Further assessment of this event led to the recommendation that it was not reportable because the leak testing was done with gas (system would be operated with water only) and the test pressure of 60 psig was significantly greater than the system pressure during post-accident recirculation. Following review of this event on July 1, 1987 by the Plant Review Board, it was concluded that this event is reportable in accordance with 10 CFR 50.73(a)(2)(ii) due to the potential for this leakage exceeding the FSAR design basis with regard to post-accident doses to control room operators.

Cause of Occurrence

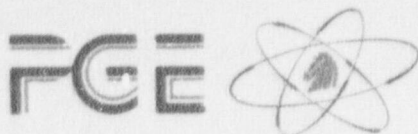
The cause of this event was leakage from the packing of valves MO-8811B and MO-2052B. The packing leakage was attributed to normal packing degradation.

Corrective Action

The valve packings were tightened and leak tested satisfactorily. An evaluation of the valve packing program for valves in the recirculation flow path will be performed by December 31, 1987 to identify changes necessary to prevent recurrence of this type of leakage.

Significance of Occurrence

This event had no effect on public health and safety. There was no event that required operation of the sump recirculation flow paths. The LLRT of this penetration was performed with air at 60 psig. Actual leakage in the event of operation of the recirculation sump flow path would have been significantly less because: (1) the leakage would have been liquid instead of gas, and (2) recirculation does not commence until post-accident Containment pressures have decreased significantly below 60 psig.



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July 31, 1987
CAO-265-87

US Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

Licensee Event Report No. 87-18 is attached. This report discusses an event in which valve packing leakage exceeded the values assumed in the Final Safety Analysis Report.

Sincerely,

C. A. Olmstead
General Manager
Trojan Nuclear Plant

c: Mr. John B. Martin
Regional Administrator
US Nuclear Regulatory Commission

Mr. David Kish, Acting Director
State of Oregon
Department of Energy

Mr. R. C. Barr
USNRC Resident Inspector
Trojan Nuclear Plant

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