

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Limerick Generating Station - Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 5 2										PAGE (3) 1 OF 0 4	
TITLE (4) Failure to Perform Surveillance Test Within the Allotted Time as Required by Technical Specifications Due to a Personnel Error.																					
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)							
0 9	2 7	8 7	8 7	0 5	4	0 0	1 0	2 8	8 7						0 5 0 0 0						
OPERATING MODE (9) 1			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																		
POWER LEVEL (10) 1 2			20.402(b)			20.406(e)			50.73(a)(2)(iv)			73.71(b)									
			20.406(a)(1)(i)			50.36(e)(1)			50.73(a)(2)(v)			73.71(a)									
			20.406(a)(1)(ii)			50.36(e)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 350A)									
			20.406(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)												
			20.406(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)												
			20.406(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)												
LICENSEE CONTACT FOR THIS LER (12)												TELEPHONE NUMBER									
NAME Charles A. Mengers, Senior Engineer, Licensing Section												AREA CODE 2 1 5 8 4 1 - 5 1 8 4									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS											
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)									
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO									

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract: 87-054

On September 28, 1987 a review of the surveillance test (ST) schedule determined that a surveillance test had not been performed within the required time period. The Instrument and Control ST Coordinator had assigned an incorrect date for the performance of the ST. Therefore, the monthly functional test of the 'E' channel for the control rod block for the scram discharge volume was not completed within the required time period. The ST (ST-2-047-614-1 - CRD Block - Scram Discharge Volume Water Level-High; Channel 'E' Calibration/Functional Test - LSH-47-1N013E) was 17 hours overdue when it was satisfactorily completed on September 28, 1987 at 1455 hours. There was no release of radioactive material or adverse consequences associated with this event. The weekly ST performance schedule list will be more closely monitored to assure that the STs are performed on schedule.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/86

FACILITY NAME (1)  Elmerick Generating Station Unit 1	DOCKET NUMBER (2)  0 5 0 0 0 3 5 2	LER NUMBER (3)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 7	0 5 4	0 0	0 2	OF 0 4

TEXT (If more space is required, use additional NRC Form 386A) (1/1)

Unit Conditions Prior to the Event:

Operating Condition 1 (Power Operation)

Reactor Power 12%

Description of the Event:

On September 28, 1987 the Instrument and Control (I&C) Surveillance Test (ST) Coordinator determined that a surveillance test (ST-2-047-614-1) was not performed within the required time period. Surveillance test ST-2-047-614-1 (CRD Block - Scram Discharge Volume Water Level-High; Channel 'E' Calibration/Functional Test - LSH-47-1N013E), intended to check the operability of the control rod block instrumentation, was to be completed by September 27, 1987 at 2155 hours. While manually entering dates on STs to be performed on a weekend, from a master ST schedule, the ST Coordinator inadvertently entered an incorrect date of September 30, 1987 for the performance of the ST. Therefore, the surveillance test was not performed within the required time period. Technical Specifications require that the instrument channel be demonstrated operable on a monthly basis. Failure to comply with this requirement requires that the Technical Specification Action Requirements 3.3.6 Table 3.3.6-1 be implemented. This action statement requires: "With the number of OPERABLE channels less than required by the Minimum OPERABLE channels per Trip Function requirement, place the inoperable channel in the tripped condition within one hour". The surveillance test (ST-2-047-614-1) was 17 hours overdue when it was satisfactorily completed on September 28 at 1455 hours. From this, it can be concluded that the 'E' channel control rod block scram discharge volume high water level instrumentation would have met its intended purpose since the last surveillance test.

The EIIS Code for the affected system is AA (Control Rod Drive System).



## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)  Limerick Generating Station Unit 1	DOCKET NUMBER (2)  0500035287	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		87	054	00	03	OF	04

TEXT (If more space is required, use additional NRC Form 365A's) (17)

Consequences of the Event:

There was no release of radioactive material or adverse consequences associated with this event. Although the control rod block scram discharge volume high water level instrumentation was not in compliance with Technical Specification surveillance requirements, the instrumentation was proven operable and would have performed its intended function in the event a scram discharge volume high water level was sensed.

Cause of the Event:

The surveillance test (ST-2-047-614-1) was not performed within the required time period due to a personnel error. The I&C Surveillance Test Coordinator inadvertently entered an incorrect due date of September 30, 1987 rather than September 27, 1987 on the first page of ST-2-047-614-1. As a result, the surveillance was not performed on schedule.

Corrective Actions:

Surveillance Test (ST-2-047-614-1) was completed satisfactorily on September 28, 1987 at 1455 hours. The control rod block scram discharge volume high water level instrumentation performed as designed and the surveillance requirements restored. The I&C ST Coordinator reviewed previously performed STs to ensure that the due dates entered met the surveillance requirements. No further deficiencies were identified.

Actions Taken to Prevent Recurrence:

The I&C personnel responsible for scheduling the performance of surveillance tests have prepared and are maintaining on a daily basis, a special tracking list. This list is prepared from the master surveillance test schedule and consists of tests close to expiration of their surveillance intervals. This list will aid in ensuring that STs are performed on time.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)  Limerick Generating Station  Unit 1	DOCKET NUMBER (2)  0 5 0 0 0 3 5 2 8 7 -- 0 5 4 -- 0 0 0 4 OF 0 4	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Previous Similar Occurrences:

CODE: A - Personnel Error  
A7 - Failure to properly communicate

Limerick LER's 87-030, 86-032, and 85-069 reported a failure to perform a surveillance test within the specified time due to a personnel error.

PHILADELPHIA ELECTRIC COMPANY

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PHILADELPHIA, PA. 19101

October 28, 1987

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Docket No. 50-352

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Washington, DC 20555

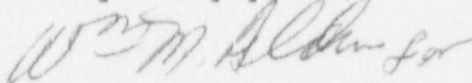
SUBJECT: Licensee Event Report  
Limerick Generating Station - Unit 1

This LER concerns a personnel error that resulted in a failure to perform a surveillance test within the allotted time as required by Technical Specifications.

Reference:	Docket No. 50-352
Report Number:	87-054
Revision Number:	00
Event Date:	September 27, 1987
Discovery Date:	September 28, 1987
Report Date:	October 28, 1987
Facility:	Limerick Generating Station P.O. Box A, Sanatoga, PA 19464

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(i)(B).

Very truly yours,



R. H. Logue  
Assistant to the Manager  
Nuclear Support Department

cc: W. T. Russell, Administrator, Region I, USNRC  
E. M. Kelly, Senior Resident Site Inspector

LEE  
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